



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038  
Hope Creek Generating Station

December 3, 1990

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION  
DOCKET NO. 50-354  
UNIT NO. 1  
LICENSEE EVENT REPORT 90-025-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(i).

Sincerely,

J.J. Hagan  
General Manager -  
Hope Creek Operations

RAR/

Attachment  
SORC Mtg. 90-110

C Distribution

9012110223 901203  
PDR ADOCK 05000354  
S FDC

The Energy People

IE22

1/1  
95-11-25M 12-88

LICENSEE EVENT REPORT

FACILITY NAME (1) HOPE CREEK GENERATING STATION										DOCKET NUMBER (2) 0   5   0   0   0   3   5   4					PAGE (3) 1   OF   1	
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TITLE (4): RECIRCULATION SYSTEM INSTRUMENT LINE CRACK

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	**	NUMBER	**	REV	MONTH	DAY	YEAR	FACILITY NAME(S)		DOCKET NUMBER(S)										
1	1	0	4	9	0	9	0	-	0	2	5	-	0	0	1	2	0	3	9	0			

OPERATING MODE (9)		3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR: (CHECK ONE OR MORE BELOW) (11)											
POWER LEVEL	0	0	0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		OTHER (Specify in Abstract below and in Text)		
				20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)				
				20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)						
				20.405(a)(1)(iii)	XX	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
				20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Robin Ritzman, Engineer - Technical										TELEPHONE NUMBER 6   0   9   3   3   9   3   7   3   7				
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE NOTED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS?	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS?
X	AD	PSX	UOOO	N					

SUPPLEMENTAL REPORT EXPECTED? (14)		YES	XX	NO	DATE EXPECTED (15)			MONTH	DAY	YEAR	
								0	1	1	6

ABSTRACT (16)

On 11/4/90 at 1515, with the Reactor in Operational Condition 3 (Hot Shutdown), and while conducting an investigation to determine the source of unidentified Drywell leakage, a leak was discovered at a weld on a Reactor Recirculation instrument line.

Subsequent to the discovery of the leak, the subject instrument line was cut out and replaced with a new section of piping. The failed line was forwarded to an independent laboratory for failure analysis of the joint weld. Preliminary results of the analysis have been received which indicate that the cause of the leaking weld appears to be vibration induced fatigue. A final report is expected prior to the end of December, 1990. As an interim measure, other similar piping and connections in the Reactor Recirculation System were visually inspected and a sampling of similar welds were non-destructively examined to ensure pressure boundary integrity. No further problems were noted. Additionally, one design change was implemented to install instrumentation to monitor vibration levels of the Reactor Recirculation instrument lines and another design change was implemented to reduce the potential for fatigue induced failure due to vibration by adding pipe supports and modifying some existing supports.

PSE&G will provide a supplement to this report when the failure analysis and independent evaluation of the subject piping is completed.