

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

December 3, 1990

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION DOCKET NO. 50-354 UNIT NO. 1 LICENSEE EVENT REPORT 90-025-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(i).

Sincerely,

aga

J.J. Hagan General Manager -Hope Creek Operations

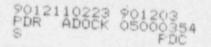
TE22

15 /1 25M 12 85

RAR/

Attachment SORC Mtg. 90-110

C Distribution



The Energy People

	4												1	ICE	NSE	E EV	ENI	REPO	R.														
FACILITY NAME (1) HOPE CREEK GENERATING STATION													DOCKET NUMBER (2) PAG 0 5 0 0 0 3 5 4 1 0										GE (3) OF   1										
ALTTI	(4)	R	ECTR	CULAT	ION	SYS	TEM	INST	RUN	(EN	Γ1	IN	E C	RAC!																			
EVENT DATE (5) LER NUMBER (6)									REPOR					E (7)				OTHER FACILLTIES IN									VOLVED (8)						
MONTH DAY YEAR		EAR	YEAR	*	* INU	NUMBER **			REV		MONTS						FACI	LITY	'N	NAME(S)						11	00	KET	N	MEE	R(S)	5	
1   1	0	4	9 0	9	0	. 0	2	5	-	0	0	1	2	0	3	9	0																
	RAT.					PORT D2 (b		SUH	UT	ED			JAN1 05 (c		TH	E RE		REMEN 0.73(					((			ONE .71			ORE	B₽	1.0%	) (1	-
OKER	Contract and a second second part of the second									50.36(c)(1) 50 50.36(c)(2) 50							0.73(	73(a)(2)(v) 73.71(								(c)	(c) (Specify in						
				2	20.405(a)(1)(iv) 50									).73(a)(2)(ii) 50						50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B) 50.73(a)(2)(x)								Abstract below and in Text)					
/////	111	111	1111	1 12	0.40	95 (a	)(1)	(v)	-		150			Cr. double line				0.73( OR TH			and the second second	2)					-		-	-	-		-
NAME		alasa a sa				est person			-					and the second				NO. 41		201	-34		1	7	FL	EPH	ONE	N	UMB	ER			1
			Robi	n Rit	znar	n, B	ngin	eer	- 7	ect	nni	cal											6	0	19	3	3	3	9	3	7	3	7
			C	MPLE	TE	NE	LINE	FOR	EA	CH	00	MPC	NEN	T F	AIL	URE 1	TOP	ED IN	THI	IS I	REF	ORI	1 (1	3)	and the second	a codere			-			a sunds	1
			OMPON	FONENT MANUFAC- TURER									111111 CAUS			SYST	TEM   COMPONEN		NI	T MANUFA				10 States			PORTABLE ONPRDS?						
X	a dore a day o	ND.	-	PSX		U000			N		to Alter a data in a second de la seconda			111111								_								_			
SUPPLE	MEN	IAL .	KEPO	KL EX	PEC	ED?	(14	0 1	ES	XX	N	01	+	DAT	EE	XPEC	TED	(15)	-	-		design of the local distance of the local di	DA 1	CARGO CONTRACTOR							111		

ABSTRACT (16)

On 11/4/90 at 1515, with the Reactor in Operational Condition 3 (Hot Shutdown), and while conducting an investigation to determine the source of unidentified Drywell leakage, a leak was discovered at a weld on a Reactor Recirculation instrument line.

Subsequent to the discovery of the leak, the subject instrument line was cut out and replaced with a new section of piping. The failed line was forwarded to an independent laboratory for failure analysis of the joint weld. Preliminary results of the analysis have been received which indicate that the cause of the leaking weld appears to be vibration induced fatigue. A final report is expected prior to the end of December, 1990. As an interim measure, other similar piping and connections in the Reactor Recirculation System were visually inspected and a sampling of similar welds were non-destructively examined to ensure pressure boundary integrity. No further problems were noted. Additionally, one design change was implemented to install instrumentation to monitor vibration levels of the Reactor Recirculation instrument lines and another design change was implemented to reduce the potential for fatigue induced failure due to vibration by adding pipe supports and modifying some existing supports.

PSE&G will provide a supplement to this report when the failure analysis and independent evaluation of the subject piping is completed.