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MICHIGAN'S PROGRESS**

Big Rock Point Nuclear Plant, 10260 US-31 North, Charlevoix, MI 49720

December 3, 1990

William L Beckman
Plant Manager

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of November 1, 1990 through November 30, 1990.

Sincerely,

W L Beckman
Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission
URHahn, Department of Public Health
RCallen, Michigan Public Service Commission
SHall, Michigan Department of Labor
PDKrippner, American Nuclear Insurers
INPO Record Center
NRC Resident Inspector
Document Control, Big Rock Point, 740/22*35*10
DPHoffman, P24-117B
KWBerry, P24-614B
File

9012110080 901130
PDR ADOCK 05000155
R PC

MI0186-0667A-BT01

A CMS ENERGY COMPANY

IE24



NUCLEAR OPERATIONS DEPARTMENT
Unit Shutdowns and Power Reductions

Report Month November 1990	Docket Number 55 - 150	Unit Big Rock Point Plant	Date December 3, 1990	Completed by JR Johnston	Telephone (616) 547-6537 ext 223
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Number	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report Number	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
90-06	09/21/90 thru 11/28/90	S	663.6 Hrs	C	1	-----	----	-----	1990 Normal Refueling Outage (1,624.9 Hrs Total) * * Correction to October outage hrs - should have been 745 hrs instead of 744 hrs due to day light savings time.

¹F = Forced
S = Scheduled

²Reason:
A = Equipment Failure (Explain)
B = Maintenance of Test
C = Refueling
D = Regulatory Restriction
E = Operator Training and License Examination Administrative
G = Operational Error (Explain)
H = Other (Explain)

³Method:
1 = Manual
2 = Manual Scram
3 = Automatic Scram
4 = Other (Explain)

⁴Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155

DATE: 12 / 4 / 90

BY: JR. JOHNSTON

PHONE: 616-547-6537 EXT 223

OPERATING STATUS

1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT
2. REPORTING PERIOD: 11 / 90
3. LICENSED THERMAL POWER (MWT): 240
4. NAMEPLATE RATING (GROSS MWE): 75
5. DESIGN ELECTRICAL RATING (NET MWE): 72
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

NOTES:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	8016.0	242611.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	66.0	6015.0	175164.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	56.4	5980.9	172269.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	6932.0	1258685.0	32822145.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	2206.0	402924.0	10420572.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	2020.0	382082.4	9856516.0
19. UNIT SERVICE FACTOR	7.8%	74.6%	71.0%
20. UNIT AVAILABILITY FACTOR	7.8%	74.6%	71.0%
21. UNIT CAPACITY FACTOR (USING HDC NET)	4.2%	71.1%	60.3%
22. UNIT CAPACITY FACTOR (USING DER NET)	3.9%	66.2%	56.4%
23. UNIT FORCED OUTAGE RATE	0.0%	6.4%	12.3%

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH):

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

DAY AVERAGE DAILY POWER (MWT) (MWEN) (11/90) - CYCLE 25

1	0.0	0.0
2	0.0	0.0
3	0.0	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.0	0.0
8	0.0	0.0
9	0.0	0.0
10	0.0	0.0
11	0.0	0.0
12	0.0	0.0
13	0.0	0.0
14	0.0	0.0
15	0.0	0.0
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0
19	0.0	0.0
20	0.0	0.0
21	0.0	0.0
22	0.0	0.0
23	0.0	0.0
24	0.0	0.0
25	0.0	0.0
26	0.0	0.0
27	0.0	0.0
28	22.08	2.62
29	105.50	31.83
30	161.25	49.71

Refueling Information Request

1. Facility name: Big Rock Point Plant
2. Scheduled date for next refueling shutdown: November, 1991
3. Scheduled date for restart following shutdown: January, 1992
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? No

If no review has taken place, when is it scheduled?

The PRC review will be completed before reload commences.

5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
7. Number of fuel assemblies in: core 84; spent fuel storage pool 274; new fuel storage 0.
8. Present licensed spent fuel storage capacity: 441
Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: 1995