



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-321
EDWIN I. HATCH NUCLEAR PLANT, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-57 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 15, 1994



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GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated September 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

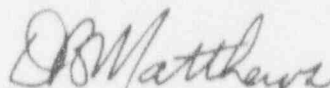
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 133, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 15, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

AND

TO LICENSE AMENDMENT NO. 133

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

	<u>Remove Pages</u>	<u>Insert Pages</u>
Unit 1	3.1-7 3.2-40	3.1-7 3.2-40
Unit 2	3/4 3-7 3/4 3-41	3/4 3-7 3/4 3-41

Table 4.1-1

Reactor Protection System (RPS) Instrumentation Functional Test, Functional Test Minimum Frequency, and Calibration Minimum Frequency

Scram Number (a)	Source of Scram Trip Signal	Group (b)	Instrument Check Minimum Frequency	Instrument Functional Test Minimum Frequency (c)	Instrument Calibration Minimum Frequency
1	Mode Switch in SHUTDOWN	A	NA	Once/Operating Cycle	Not Applicable
2	Manual Scram	A	NA	Once/week	Not Applicable
3	IRM High High Flux	C	D	Once/Week (e)(l)(n)	Once/Operating Cycle
	Inoperative	C	NA	Once/week (e)	NA
4	Reactor Vessel Steam Dome Pressure - High	D	S	Every 3 months	Once/Operating Cycle
5	Drywell Pressure - High	D	S	Every 3 months	Once/Operating Cycle
6	Reactor Vessel Water Level - Low (Level 3)	D	S	Every 3 months (g)	Once/Operating Cycle
7	Scram Discharge Volume High High Level				
	a. Float Switches	A	NA	Once/Operating Cycle	(h)
	b. Thermal Level Sensors	B	NA	Every 3 months	Once/Operating Cycle
8	APRM Fixed High-High Flux	B	S	Every 3 months (e)(l)	Once/Week (p), SA
	Inoperable	B	NA	Every 3 months (e)	NA
	Downscale	B	NA	Once/Week(e)	NA
	Flow Reference Simulated Thermal Power Monitor	B	S	Every 3 months (l)	Once/Week (p)(q), SA
	15% Flux	C	S	Once/Week during refueling (l)(m)(n)	Once/Week during refueling (l)(m)
9	LPRM	B	D	NA	Every 1000 Effective Full Power Hours

Table 4.2-7

Check, Functional Test, and Calibration Minimum Frequency for
Neutron Monitoring Instrumentation Which Initiates
Control Rod Blocks

Ref. No.	Instrument	Instrument Check Minimum Frequency (b)	Instrument Functional Test Minimum Frequency (c)	Instrument Calibration Minimum Frequency (d)
1	<u>SOURCE RANGE MONITORS</u>			
	a. Detector not full in	NA	S/U ^(ff) , W	NA
	b. Upscale	NA	S/U ^(ff) , W	R
	c. Inoperative	NA	S/U ^(ff) , W	NA
	d. Downscale	NA	S/U ^(ff) , W	R
2	<u>INTERMEDIATE RANGE MONITORS</u>			
	a. Detector not full in	NA	S/U ^(ff) , W ^(a)	NA
	b. Upscale	NA	S/U ^(ff) , W ^(a)	R
	c. Inoperative	NA	S/U ^(ff) , W ^(a)	NA
	d. Downscale	NA	S/U ^(ff) , W ^(a)	R
3	<u>APRM</u>			
	a. Flow Referenced Simulated Thermal Power-Upscale	NA	S/U ^(ff) , Q	R
	b. Inoperative	NA	S/U ^(ff) , Q	NA
	c. Downscale	NA	S/U ^(ff) , Q	R
	d. Neutron Flux - High, 12%	NA	S/U ^(ff) , Q	R
4	<u>ROD BLOCK MONITOR</u>			
	a. Upscale	NA	S/U ^(ff) , Q	R
	b. Inoperative	NA	S/U ^(ff) , Q	NA
	c. Downscale	NA	S/U ^(ff) , Q	R
5	<u>SCRAM DISCHARGE VOLUME</u>			
	a. Water Level-High	NA	R	R

Notes for Table 4.2-7

- a. The column titled "Ref. No." is only for convenience so that a one-to-one relationship can be established between items in Table 4.2-7 and items in Table 3.2-7.
- b. Deleted.

TABLE 4.8.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION ¹	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
1. Intermediate Range Monitors:				
a. Neutron Flux - High	D	S/U ^{2,3,4}	R	2
	D	W	R	3, 4, 5
b. Inoperative	NA	W	NA	2, 3, 4, 5
2. Average Power Range Monitor:				
a. Neutron Flux - Upscale, 15%	S	S/U ^{2,3,4} , W ^{4,5}	S/U ^{2,3} , W ^{4,5}	2
	S	W	W	5
b. Non Referenced Simulated Thermal Power - Upscale	S	S/U ^{2,3} , Q	W ^{4,5,6} , SA	1
c. Fixed Neutron Flux - Upscale, 118%	S	S/U ^{2,3} , Q	W ^{4,5} , SA	1
d. Inoperative	NA	Q	NA	1, 2, 5
e. Downscale	NA	W	NA	1
f. LPRM	D	NA	let	1, 2, 5
3. Reactor Vessel Steam Dome Pressure - High	S	Q	R	1, 2
4. Reactor Vessel Water Level - Low (Level 3)	S	Q	R	1, 2
5. Main Steam Line Isolation Valve - Closure	NA	Q	R	1
6. (Deleted)				
7. Drywell Pressure - High	S	Q	R	1, 2
8. Steam Discharge Volume Water Level - High				
a. Float Switches	NA	R	R ^{2,3}	1, 2, 5
b. Thermal Level Switches	NA	Q	R	1, 2, 5

TABLE 4.3.5-1

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION^(a)</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. APRM:				
a. Flow Referenced Simulated Thermal Power-Upscale	NA	S/U ^(b) , Q	R	1
b. Inoperative	NA	S/U ^(b) , Q	NA	1, 2, 5
c. Downscale	NA	S/U ^(b) , Q	R	1
d. Neutron Flux - High, 12%	NA	S/U ^(b) , Q	R	2, 5
2. Rod Block Monitor:				
a. Upscale	NA	S/U ^(b) , Q	R	1 ^(d)
b. Inoperative	NA	S/U ^(b) , Q	NA	1 ^(d)
c. Downscale	NA	S/U ^(b) , Q	R	1 ^(d)
3. Source Range Monitors:				
a. Detector not full in	NA	S/U ^(b) , W	NA	2, 5
b. Upscale	NA	S/U ^(b) , W	R	2, 5
c. Inoperative	NA	S/U ^(b) , W	NA	2, 5
d. Downscale	NA	S/U ^(b) , W	R	2, 5
4. Intermediate Range Monitors:				
a. Detector not full in	NA	S/U ^(b) , W ^(c)	NA	2, 5
b. Upscale	NA	S/U ^(b) , W ^(c)	R	2, 5
c. Inoperative	NA	S/U ^(b) , W ^(c)	NA	2, 5
d. Downscale	NA	S/U ^(b) , W ^(c)	R	2, 5
5. Scram Discharge Volume:				
a. Water Level-High	NA	R	R	1, 2, 5 ^(e)