

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3 REACTIVITY CONTROLApplicability:

Applies to the operational status of the control rod system.

Objective:

To assure the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

A sufficient number of control rods shall be operable so that the core could be made subcritical in the most reactive condition during the operating cycle with the strongest control rod fully withdrawn and all other operable control rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Control rods which cannot be moved with control rod drive pressure shall be considered inoperable.

If a partially or fully withdrawn control rod drive cannot be moved with drive or scram pressure the reactor shall be brought to a shutdown condition within 48 hours unless

4.3 REACTIVITY CONTROLApplicability:

Applies to the surveillance requirements of the control rod system.

Objective:

To verify the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

Sufficient control rods shall be withdrawn following a refueling outage when core alterations were performed to demonstrate with a margin of $0.38\% \Delta k/k$ that the core can be made subcritical at any time in the subsequent fuel cycle with the analytically determined strongest operable control rod fully withdrawn and all other operable rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Each partially or fully withdrawn operable control rod shall be exercised one notch at least once each week when operating above the RWM low power setpoint. Each partially or fully withdrawn operable control rod shall be exercised at least one notch within 24 hours when operating above the RWM low power setpoint if there are three

Unit 2

PBAPS

LIMITING CONDITIONS FOR OPERATION

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

B. Normal System Requirements

At all times when the Standby Liquid Control System is required to be operable, the following conditions shall be met:

1. At least 162.7 lbm Boron-10 must be stored in the Standby Liquid Control Solution Tank and be available for injection.
2. The sodium pentaborate solution concentration must be equal to or less than 9.82% weight.

SURVEILLANCE REQUIREMENTS

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

2. Manually initiate one of the Standby Liquid Control System Pumps and pump demineralized water into the reactor vessel from the test tank.

This test checks explosion of the charge associated with the tested loop, proper operation of the explosive valves, and pump operability. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired.

3. Both systems, including both explosive valves, shall be tested in the course of two operating cycles.

B. Normal System Requirements

1. Boron-10 Quantity: At least once per month calculate and record the quantity of Boron-10 stored in the Standby Liquid Control Solution Tank.
2. Concentration: At least once per month check and record. Also, check concentration anytime water or boron is added to the solution.

Unit 2

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

3. The Standby Liquid Control System conditions must satisfy the following equation:

$$\left(\frac{C}{13\% \text{ wt.}}\right) \left(\frac{Q}{86 \text{ gpm}}\right) \left(\frac{E}{19.8\% \text{ atom}}\right) \geq 1$$

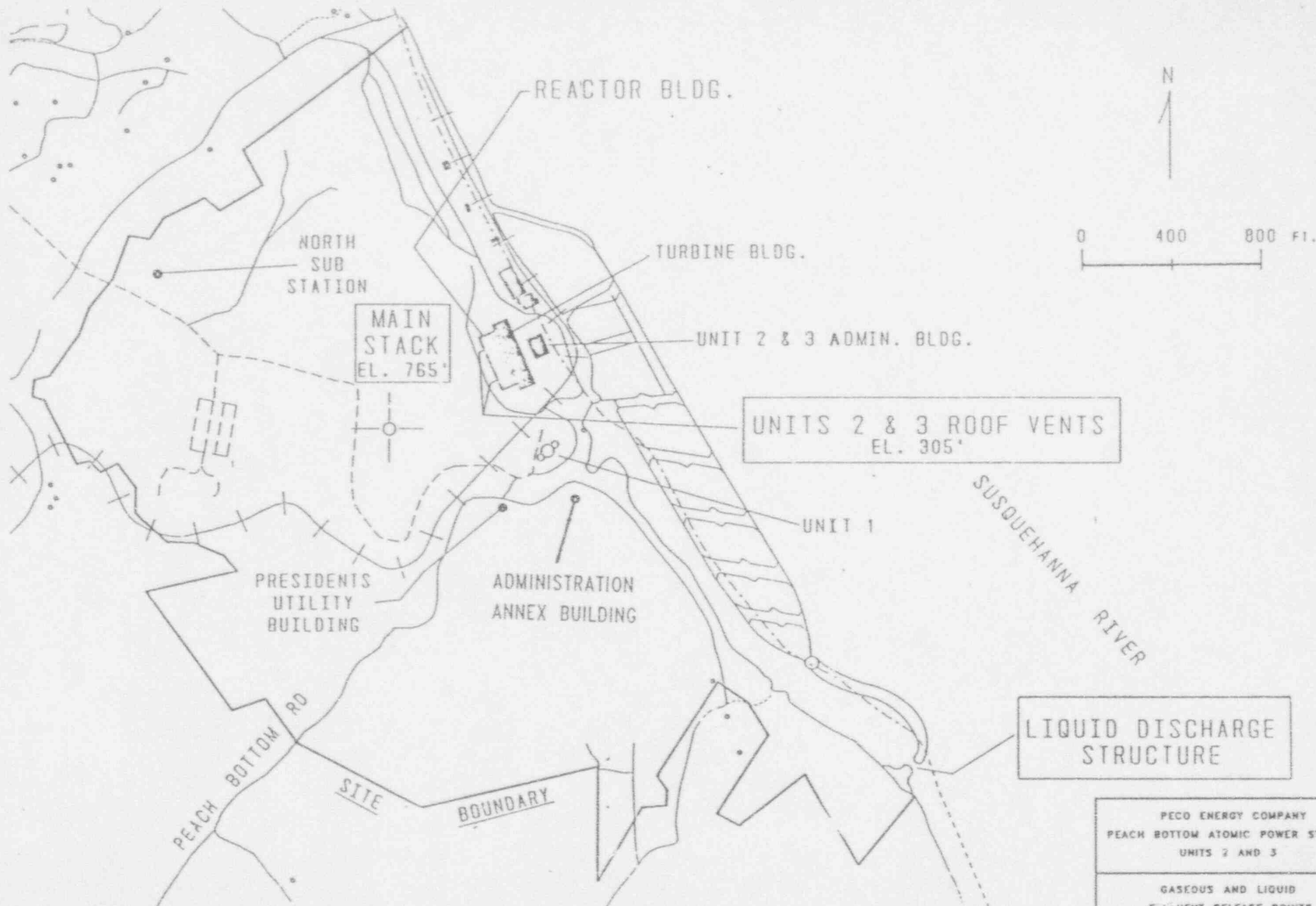
where,

C = Sodium Pentaborate Solution Concentration (% weight)

Q = Pump Flow Rate (gpm) against a system head of 1225 psig.

E = Boron-10 Enrichment (% atom Boron-10)

3. Pump Flow Rate: At least once per 92 days each pump loop shall be functionally tested by pumping boron solution to the test tank. At least once per quarter check and record pump flow rate against a system head of 1225 psig.
4. Enrichment: Following each addition of boron to the solution tank, calculate enrichment within 8 hours. Verify results by analysis within 30 days.
5. Solution Volume: At least once per day check and record.



PECO ENERGY COMPANY
 PEACH BOTTOM ATOMIC POWER STATION
 UNITS 2 AND 3

GASEOUS AND LIQUID
 EFFLUENT RELEASE POINTS
 FIGURE 3.B.1

Unit 2

-216e-

PBAPS

6.5.2 Nuclear Review Board Function

6.5.2.1 The Nuclear Review Board (NRB) shall function to provide independent review and audit of designated activities in the area of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

The members of the NRB will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance organization available to them.

Organization

6.5.2.2 The Chairman, members and alternate members of the NRB shall be appointed in writing by the Executive Vice President - Nuclear, and shall have an academic degree in an engineering or physical science field and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

The NRB shall be composed of at least five members who are PECO Energy Company (PECO) employees and at least two outside members who are not PECO employees.

PBAPS

6.5.2 Nuclear Review Board Function

6.5.2.1 The Nuclear Review Board (NRB) shall function to provide independent review and audit of designated activities in the area of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

The members of the NRB will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance organization available to them.

Organization

6.5.2.2 The Chairman, members and alternate members of the NRB shall be appointed in writing by the Executive Vice President - Nuclear, and shall have an academic degree in an engineering or physical science field and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

The NRB shall be composed of at least five members who are PECO Energy Company (PECO) employees and at least two outside members who are not PECO employees.

PBAPS

6.9.1 Routine Reports (cont'd)c. Annual Safety/Relief Valve Report

Describe all challenges to the primary coolant system safety and relief valves. Challenges are defined as the automatic opening of the primary coolant safety or relief valves in response to high reactor pressure.

d. Monthly Operating Report

Routine reports of operating statistics and shutdown experience and a narrative summary of the operating experience shall be submitted on a monthly basis. Each report shall be submitted no later than the 15th of the month following the calendar month covered by the report.

e. Core Operating Limits Report

- (1) Core operating limits shall be established and shall be documented in the CORE OPERATING LIMITS REPORT prior to each Operating Cycle, or prior to any remaining portion of an Operating Cycle, for the following:
 - a. The APLHGR for Specification 3.5.I,
 - b. The MCPR for Specification 3.5.K,
 - c. The core flow and power adjustment factors for Specification 3.5.K and 3.5.I,
 - d. The LHGR for Specification 3.5.J,
 - e. The upscale power biased Rod Block Monitor setpoints and corresponding power levels.
- (2) The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents as amended and approved:
 - a. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version)
 - b. "Maximum Extended Load Line Limit and ARTS Improvement Program Analyses for Peach Bottom Atomic Power Station Units 2 and 3," NEDC-32162P, Revision 1, February, 1993
 - c. PECO Energy Company Methodologies as described in:
 - (1) PECO-FMS-0001-A, "Steady-State Thermal Hydraulic Analysis of Peach Bottom Units 2 and 3 using the FIBWR Computer Code"

PBAPS

6.0 Environmental Surveillance and Special Study Programs

Studies described in this section will be terminated upon agreement by PECO Energy Company and the Nuclear Regulatory Commission that the intended purpose of the study has been satisfied.

6.1 (Deleted)

6.2 (Deleted)

6.3 (Deleted)

UNITED STATES
ATOMIC ENERGY COMMISSION
Washington, DC 20545

PECO ENERGY COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY
DOCKET NO. 50-278
PEACH BOTTOM ATOMIC POWER STATION, UNIT 3
FACILITY OPERATING LICENSE

License No. DPR-56

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for license filed by PECO Energy Company, formerly Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Peach Bottom Atomic Power Station, Unit 3 (the facility) has been substantially completed in conformity with Construction Permit No. CPPER-38 and the application, as amended, the provisions of the Act and the rules and regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - D. There is reasonable assurance: (1) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (2) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - E. PECO Energy Company is technically qualified and the licensees are financially qualified to engage in the activities authorized by this amended operating license in accordance with the rules and regulations of the Commission;
 - F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this operating license will not be inimical to the common defense and security or to the health and safety of the public;

- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental costs and considering available alternatives, the issuance of Facility Operating License No. DPR-56 is in accordance with 10 CFR Part 50, Appendix D, of the Commission's regulations and all applicable requirements of said Appendix D have been satisfied; and
- I. The receipt, possession, and use of by-product and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR Section 30.33, 40.32, and 70.23 and 70.31.
2. Facility Operating License No. DPR-56 is hereby issued to the PECO Energy Company, formerly the Philadelphia Electric Company, Public Service Electric and Gas Company (PSE&G), Delmarva Power and Light Company (DP&LC), and Atlantic City Electric Company (ACEC), is hereby amended in its entirety to read as follows:
- A. This license applies to the Peach Bottom Atomic Power Station, Unit 3, a direct cycle, forced circulation, boiling water nuclear reactor and associated equipment (the facility), owned by the licensees and operated by PECO Energy Company. The facility is located in Peach Bottom, York County, Pennsylvania and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 31) and the Environmental Report as supplemented and amended (Supplements 1 through 6).
- B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
- (1) PECO Energy Company, pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility and PSE&G, DP&LC, and ACEC to possess the facility at the designated location in Peach Bottom, York County, Pennsylvania in accordance with the procedures and limitations set forth in this license;
 - (2) PECO Energy Company, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended, as of December 15, 1975;
 - (3) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or when associated with radioactive apparatus or components;

- (5) PECO Energy Company, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

- (1) Maximum Power Level

PECO Energy Company is authorized to operate the Peach Bottom Atomic Power Station, Unit 3, at steady state reactor core power levels not to exceed 3293 megawatts thermal.

- (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. are hereby incorporated in the license. PECO Energy Company shall operate the facility in accordance with the Technical Specifications.

- (3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Peach Bottom Atomic Power Station, Units 2 and 3, Physical Security Plan," with revisions submitted through December 16, 1987; "Peach Bottom Atomic Power Station, Units 2 and 3 Plant Security Personnel Training and Qualification Plan," with revisions submitted through July 9, 1986; and "Peach Bottom Atomic Power Station, Units 2 and 3 Safeguards Contingency Plan," with revisions submitted through March 10, 1981. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- (4) The licensee may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.16 of the NRC's Fire Protection Safety Evaluation (SE), dated May 23, 1979 for the facility. These modifications will be completed in accordance with the schedule in Table 3.1 of the SE and supplements thereto. In addition, the licensee shall submit the additional information identified in Table 3.2 of this SE in accordance with the schedule contained therein.

- (5) Operation beyond the end-of-cycle (all rods out condition) thermal power is limited to seventy (70) percent minimum. Increasing core power level via reduced feedwater heating, once operation in the coastdown mode has begun, is not permitted unless the licensee has performed an analysis of this operating condition that confirms that this condition is bounded by the analysis for the particular cycle of operation.

3. This license is subject to the following conditions for the protection of the environment:

- A. If the actual milk sample measurements taken at the nearby farms in accordance with the Technical Specifications, predict a dose to a child's thyroid, based on actual combined operation of Unit 2 and 3, that exceeds 15 mrem/year, licensees shall install for operation no later than the next refueling cycle for each unit the necessary equipment to reduce the projected dose to such levels.
- B. To the extent matters related to thermal discharges are treated therein, operation of Peach Bottom Atomic Power Station Unit 2 will be governed by NPDES Permit No. PA 0009733, as now in effect and as hereafter amended. Questions pertaining to conformance thereto shall be referred to and shall be determined by the NPDES Permit issuing or enforcement authority, as appropriate.
- C. In the event of any modification of the NPDES Permit related to thermal discharges or the establishment (or amendment) of alternative effluent limitations established pursuant to Section 316 of the Federal Water Pollution Control Act, the licensees shall inform the NRC and analyze any associated changes in or to the Station, its components, its operation or in the discharge of effluents therefrom. If such change would entail any modification to this license, or any technical specifications which are part of this license, or present an unreviewed safety question or involve an environmental impact different than analyzed in the Final Environmental Statement, the licensees shall file with the NRC, as applicable, an appropriate analysis of any such change on facility safety, and/or an analysis of any such change on the environmental impacts and on the overall cost-benefit balance for facility operation set forth in the Final Environmental Statement and a request for an amendment to the operating license, if required by the Commission's regulations. As used in this Condition 3.(d), Final Environmental Statement means the NRC Staff Final Environmental Statement related to Operation of Peach Bottom Atomic Power Station Units Nos. 2 and 3 dated April 1973, as modified by (1) the Initial Decision of the Atomic Safety and Licensing Board dated September 14, 1973, (2) the Supplemental Initial Decision of the Atomic Safety and Licensing Board dated June 14, 1974, (3) the Decision of the Atomic Safety and Licensing Appeal Board dated July 5, 1974, (4) the Memorandum and Order of the Commission dated August 8, 1974, (5) any further modification resulting from further review by the Appeal Board and by the Commission, if any, and (6) any Environmental Impact Appraisal which has been or may be issued by the NRC since the FES was published in April 1973.

4. This license is effective as the date of issuance and shall expire at midnight on July 2, 2014.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by Roger Boyd, for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachments:

Amended pages to Appendices A and B
DPR-44 & DPR-56 Technical
Specifications

Date of Issuance: July 2, 1974

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3 REACTIVITY CONTROLApplicability:

Applies to the operational status of the control rod system.

Objective:

To assure the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

A sufficient number of control rods shall be operable so that the core could be made subcritical in the most reactive condition during the operating cycle with the strongest control rod fully withdrawn and all other operable control rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Control rods which cannot be moved with control rod drive pressure shall be considered inoperable.

If a partially or fully withdrawn control rod drive cannot be moved with drive or scram pressure the reactor shall be brought to a shutdown condition within 48 hours unless

4.3 REACTIVITY CONTROLApplicability:

Applies to the surveillance requirements of the control rod system.

Objective:

To verify the ability of the control rod system to control reactivity.

Specification:A. Reactivity Limitations1. Reactivity margin - core loading

Sufficient control rods shall be withdrawn following a refueling outage when core alterations were performed to demonstrate with a margin of $0.38\% \Delta k/k$ that the core can be made subcritical at any time in the subsequent fuel cycle with the analytically determined strongest operable control rod fully withdrawn and all other operable rods fully inserted.

2. Reactivity margin - inoperable control rods

- a. Each partially or fully withdrawn operable control rod shall be exercised one notch at least once each week when operating above the RWM low power setpoint. Each partially or fully withdrawn operable control rod shall be exercised at least one notch within 24 hours when operating above the RWM low power setpoint if there are three

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.3.A Reactivity Limitations
(Cont'd)4.3.A Reactivity Limitations
(Cont'd)

- f. Inoperable control rods shall be positioned such that specification 3.3.A.1 is met. In addition, during reactor power operation, no more than one control rod in any 5 x 5 array may be inoperable (at least 4 operable control rods must separate any 2 inoperable ones). If this Specification cannot be met the reactor shall not be started, or if at power, the reactor shall be brought to a cold shutdown condition within 24 hours.

B. Control Rods

1. Each control rod shall be coupled to its drive or completely inserted and the control rod directional control valves disarmed electrically except as in 3.3.B.1.a. This requirement does not apply in the refuel condition when the reactor is vented. Two control rod drives may be removed as long as Specification 3.3.A.1 is met.

- a. For control rod 54-35, for the remainder of cycle 10 (to be completed before 10/30/95).

If coupling cannot be accomplished, the uncoupled control rod may be withdrawn when $\geq 10\%$ of rated thermal power only if all the following conditions are satisfied:

- 1) no other uncoupled control rod is withdrawn;
- 2) the uncoupled control rod may not be withdrawn past notch position 46.

B. Control Rods

1. The coupling integrity shall be verified for each withdrawn control rod as follows:
 - a. When a rod is withdrawn the first time after each refueling outage or after maintenance, observe discernible response of the nuclear instrumentation and rod position indication for the "full-in" and "full-out" position. However, for initial rods when response is not discernible, subsequent exercising of these rods after the reactor is above the Rod Worth Minimizer low power setpoint shall be performed to verify instrumentation response.
 - b. When the rod is fully withdrawn the first time after each refueling outage or after maintenance observe that the drive does not go to the overtravel position.

LIMITING CONDITIONS FOR OPERATION3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)B. Normal System Requirements

At all times when the Standby Liquid Control System is required to be operable, the following conditions shall be met:

1. At least 162.7 lbm Boron-10 must be stored in the Standby Liquid Control Solution Tank and be available for injection.
2. The sodium pentaborate solution concentration must be equal to or less than 9.82% weight.

SURVEILLANCE REQUIREMENTS4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

2. Manually initiate one of the Standby Liquid Control System Pumps and pump demineralized water into the reactor vessel from the test tank.

This test checks explosion of the charge associated with the tested loop, proper operation of the explosive valves, and pump operability. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of the batch successfully fired.

3. Both systems, including both explosive valves, shall be tested in the course of two operating cycles.

B. Normal System Requirements

1. Boron-10 Quantity: At least once per month calculate and record the quantity of Boron-10 stored in the Standby Liquid Control Solution Tank.
2. Concentration: At least once per month check and record. Also, check concentration anytime water or boron is added to the solution.

Unit 3

PBAPS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

4.4 STANDBY LIQUID CONTROL SYSTEM
(Cont'd)

3. The Standby Liquid Control System conditions must satisfy the following equation:

$$\left(\frac{C}{13\% \text{ wt.}}\right) \left(\frac{Q}{86 \text{ gpm}}\right) \left(\frac{E}{19.8\% \text{ atom}}\right) \geq 1$$

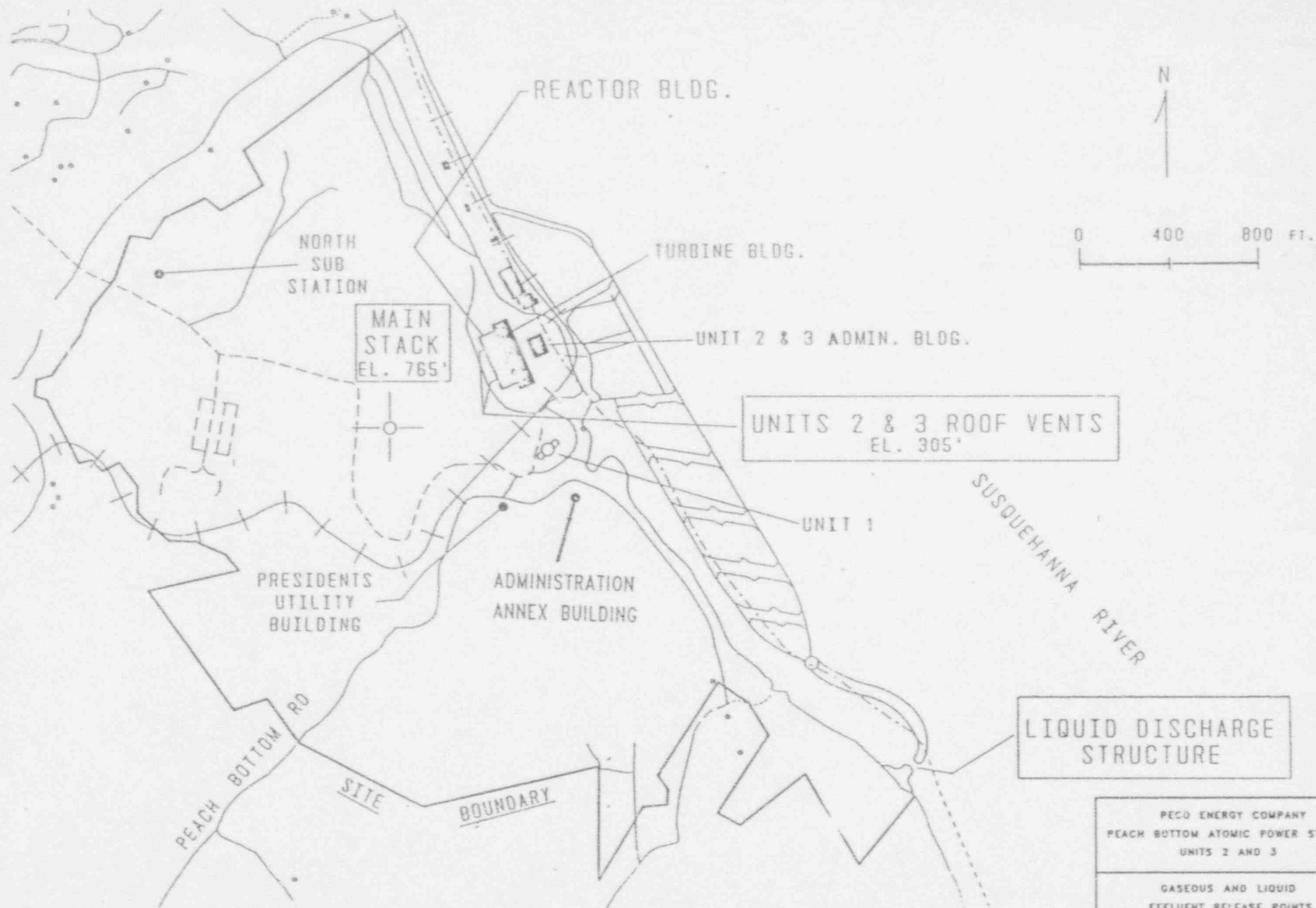
where,

C = Sodium Pentaborate Solution Concentration (% weight)

Q = Pump Flow Rate (gpm) against a system head of 1225 psig.

E = Boron-10 Enrichment (% atom Boron-10)

3. Pump Flow Rate: At least once per 92 days each pump loop shall be functionally tested by pumping boron solution to the test tank. At least once per quarter check and record pump flow rate against a system head of 1225 psig.
4. Enrichment: Following each addition of boron to the solution tank, calculate enrichment within 8 hours. Verify results by analysis within 30 days.
5. Solution Volume: At least once per day check and record.



PECO ENERGY COMPANY
 PEACH BOTTOM ATOMIC POWER STATION
 UNITS 2 AND 3
 GASEOUS AND LIQUID
 EFFLUENT RELEASE POINTS
 FIGURE J.B.1

-216e-

PBAPS

6.5.2 Nuclear Review Board Function

6.5.2.1 The Nuclear Review Board (NRB) shall function to provide independent review and audit of designated activities in the area of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

The members of the NRB will be competent in the area of quality assurance practice and cognizant of the Quality Assurance requirements of 10 CFR 50, Appendix B. Additionally, they will be cognizant of the corporate Quality Assurance Program and will have the corporate Quality Assurance organization available to them.

Organization

6.5.2.2 The Chairman, members and alternate members of the NRB shall be appointed in writing by the Executive Vice President - Nuclear, and shall have an academic degree in an engineering or physical science field and in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in 6.5.2.1.

The NRB shall be composed of at least five members who are PECO Energy Company (PECO) employees and at least two outside members who are not PECO employees.

6.9.1 Routine Reports (cont'd)c. Annual Safety/Relief Valve Report

Describe all challenges to the primary coolant system safety and relief valves. Challenges are defined as the automatic opening of the primary coolant safety or relief valves in response to high reactor pressure.

d. Monthly Operating Report

Routine reports of operating statistics and shutdown experience and a narrative summary of the operating experience shall be submitted on a monthly basis. Each report shall be submitted no later than the 15th of the month following the calendar month covered by the report.

e. Core Operating Limits Report

(1) Core operating limits shall be established and shall be documented in the CORE OPERATING LIMITS REPORT prior to each Operating Cycle, or prior to any remaining portion of an Operating Cycle, for the following:

- a. The APLHGR for Specification 3.5.I,
- b. The MCPR for Specification 3.5.K,
- c. The core flow and power adjustment factors for Specification 3.5.K and 3.5.I,
- d. The LHGR for Specification 3.5.J,
- e. The upscale power biased Rod Block Monitor setpoints and corresponding power levels.

(2) The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents as amended and approved:

- a. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version)
- b. "Maximum Extended Load Line Limit and ARTS Improvement Program Analyses for Peach Bottom Atomic Power Station Units 2 and 3," NEDC-32162P, Revision 1, February, 1993
- c. PECO Energy Company Methodologies as described in:

- (1) PECO-FMS-0001-A, "Steady-State Thermal Hydraulic Analysis of Peach Bottom Units 2 and 3 using the FIBWR Computer Code"

PBAPS

6.0 Environmental Surveillance and Special Study Programs

Studies described in this section will be terminated upon agreement by PECO Energy Company and the Nuclear Regulatory Commission that the intended purpose of the study has been satisfied.

6.1 (Deleted)

6.2 (Deleted)

6.3 (Deleted)