The Light company

Houston Lighting & Power South Texas Project Electric Generating Station P. O. Box 289 Wadsworth, Texas 77483

April 15, 1994 ST-HL-AE-4774 File No.: G02 10CFR50.71

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

South Texas Project
Units 1 and 2
Docket Nos. STN 50-498, STN 50-499
Monthly Operating Reports for March 1994

Pursuant to 10CFR50.71(a) and South Texas Project Electric Generating Station (STPEGS) Technical Specification 6.9.1.5, attached are the Monthly Operating Reports for March 1994.

If you should have any questions on this matter, please contact Mr. S. M. Head at (512) 972-7136.

J. F. Groth Vice President, Nuclear Licensing

MKJ/esh

Attachments:

- 1) STPEGS Unit 1 Monthly Operating Report March 1994
- 2) STPEGS Unit 2 Monthly Operating Report March 1994

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C:

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U. S. Nuclear Regulatory Comm. Attn: Document Control Desk Washington, D. C. 20555

SOUTH TEXAS PROJECT

ELECTRIC GENERATING STATION

UNIT 1

MONTHLY OPERATING REPORT

MARCH 1994

HOUSTON LIGHTING AND POWER CO.

NRC DOCKET NO. 50-498

LICENSE NO. NPF-76

Reviewed By: a Chalut Engineering Manager

Approved by Why Plant Manager

Date

Monthly Summary

ATTACHMENT ST-HL-AE- 4774 PAGE OF 1

STPEGS Unit 1 began the reporting period in Mode 3. A root-cause analysis was being performed due to a manual reactor trip event that occurred on 2/28/94.

The unit was in the power ascension testing program, operating at 29% reactor power, when a manual reactor trip was initiated due to dropping water level in the 1D Steam Generator. The event was caused by a malfunctioning 1D Main Feedwater Regulating Valve (MFRV). This prevented the reactor operator from taking remote manual control of the level in the 1D Steam Generator.

The malfunctioning 1D MFRV was caused by a failed transformer coil in the torque motor in the current to pneumatic converter. This resulted in the closure of the 1D MFRV and subsequent loss of feedwater flow to 1D Steam Generator. The failed converter was replaced.

Following the unit's removal from service on 2/28/94, a small leak was discovered from the primary to the secondary side of the 1C Steam Generator. The unit was taken to Mode 5 on 3/4/94 to facilitate the location and repair of the leak.

The unit was returned to service on 3/22/94 at 1731 and concluded the reporting period operating at 77% reactor power with the power ascension testing program ongoing.

ATTACHMENT I ST-HL-AF- 4774 PAGE 3 OF

OPERATING DATA REPORT

DOCKET NO. 50-498

UNIT 1

DATE Apr. 5, 1994

COMPLETED BY R.L. Hill

TELEPHONE 512/972-7667

OPERATING STATUS

1. REPORTING PERIOD: 03/01/94-03/31/94 GROSS HOURS IN REPORTING PERIOD: 744

CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800 MAX.DEPEND.CAPACITY (MWe-Net): 1250.6 DESIGN ELECTRICAL RATING (MWe-Net): 1250.6

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None

REASONS FOR RESTRICTION (IF ANY): N/A 4.

	THIS MONTH	YR TO DATE	CUMULATIVE
5.	NUMBER OF HOURS REACTOR WAS CRITICAL 263.9	525.7	27398.3
6.	REACTOR RESERVE SHUTDOWN HOURS0	0	0
7.	HOURS GENERATOR ON LINE 222.5	309.5	26233.3
8.	UNIT RESERVE SHUTDOWN HOURS	0	0
9.	GROSS THERMAL ENERGY GENERATED (MWt) 463078	545892	95211400
10.	GROSS ELECTRICAL ENERGY GENERATED (MWH) 148780	161060	32175720
11.	NET ELECTRICAL ENERGY GENERATED (MWH) 136421	144881	30404425
12.	REACTOR SERVICE FACTOR	24.3%	55.8%
13.	REACTOR AVAILABILITY FACTOR	24.3%	55.8%
14.	UNIT SERVICE FACTOR	14.3%	53.4%
15.	UNIT AVAILABILITY FACTOR	14.3%	53.4%
16.	UNIT CAPACITY FACTOR (Using MDC) 14.7%	5.4%	49.5%
17.	UNIT CAPACITY FACTOR (Using Design MWe) 14.7%	5.4%	49.5%
18.	UNIT FORCED OUTAGE RATE	85.6%	35.1%
19.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE,	AND DURATIO	N OF EACH):
	N/A		

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

AVERAGE DAILY UNIT POWER LEVEL

ATTACHMENT T	TO SECURITY OF RESIDENCE
ST-HL-AE- 4774	14
PAGE 4 OF	-

DOCKET NO. 50-498

UNIT 1

DATE Apr. 5, 1994

COMPLETED BY R.L. Hill

TELEPHONE 512/972-7667

MONTH	N. A. W. W. W.	CORPORATE TO STATE OF THE PARTY
TATE STATES IN	N1.0-121	55400
A STATE AND ASS.		60 A.A.

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	10
7		23	267
8	0	24	500
9	0	25	516
10	0	26	515
11	0	27	521
12	0	28	594
13	0	29	854
14	0	30	948
15	0	31	960
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-498 UNIT 1 DATE Apr. 5, 1994

COMPLETED BY R.L. Hill TELEPHONE 512/972-7667

REPORT MONTH MARCH

No.	Date	Type	Duration (Hours)		Method of Shutting Down Reactor	Licensee Event Report #	System Code	Component 5 Code	Cause & Corrective Action to Prevent Recurrence
94-02	940228	F	521.5	A	4	1-94-009	JB	FCV	The unit was manually tripped from 29% reactor power due to dropping water level in the 1D Steam Generator. The event was caused by the malfunctioning of the 1D Main Feedwater Regulating Valve (MFRV). This prevented the reactor operator from taking remote manual control of the level in the 1D Steam Generator. The malfunctioning 1D MFRV was caused by a failed coil in the torque motor in the current to pneumatic converter. This resulted in the closure of the 1D MFRV and subsequent loss of feedwater flow to the 1D Steam Generator.
									The failed 1D MFRV current to pneumatic converter was replaced. The 1A MFRV converter had been previously replaced and can be expected to operate reliably.

F: Forced S: Scheduled Reason:

A-Equipment Failure (Explain)

E-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Exam

F-Administrative

G-Operational Error (Explain)

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Cont. of Existing

Outage

5-Reduction

9-Other

IEEE 805-1983

IEEE 803A-1983

UNIT SHUTDOWNS AND POWER PEDUCTIONS

DOCKET NO. 50-498 UNIT 1 DATE Apr. 5, 1994 COMPLETED BY R.L. Hill TELEPHONE 512/972-7667

REPORT MONTH MARCH

No.	Date	Type	Duration (Hours)	Method of Shutting Down Reactor	Licensee Event Report #	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
								The 1B and 1C MFRV converters were replaced since there was no history of previous replacements. The failed 1D converter and the associated controller driver card, that had failed prior to the event, were returned to their respective manufacturers for failure analysis. All four Unit 2 MFRV current to pneumatic converters will be replaced to improve the reliability of the valves. After Unit 1 was removed from service, due to the manual trip, a small leak was discovered from the primary to the secondary side of the 1C Steam Generator. Eddy current data indicated one repair was needed to a previously identified leaking tube plug.

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Exam

F-Administrative

G-Operational Error (Explain)

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Cont. of Existing

IEEE 805-1983

Outage

5-Reduction

9-Other

IEEE 803A-1983

PORVs and Safety Valves Summary

ATTACHMENT ST-HL-AE-4774
PAGE 1 OF 7

There were no PORV or Safety Valves challenged during the reporting period.

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PAGE 1 OF

SOUTH TEXAS PROJECT

ELECTRIC GENERATING STATION

UNIT 2

MONTHLY OPERATING REPORT

MARCH 1994

HOUSTON LIGHTING AND POWER CO.

NRC DOCKET NO. 50-499

LICENSE NO. NPF-80

Supervisor

Reviewed By: ac mount Engineering Manager

Approved By: 10 Parluy

Monthly Summary

ATTACHMENT 2 ST-HL-AE- 4974 PAGE 2 OF Le

STPEGS Unit 2 experienced an automatic reactor scram during February 1993 and subsequently remained shutdown due to auxiliary feedwater system component failures. While the unit was shutdown, corrective maintenance was performed and organizational and work process changes were implemented.

Upon the resolution of the auxiliary feedwater system component failures and the additional issues, efforts were resumed on the refueling and scheduled maintenance outage.

The energy losses for the unit are unplanned, except for the period between February 27 and May 23, 1993, which was the previously scheduled outage period for the unit.

The unit will not be taken critical until the self-assessment process as presented by the Operational Readiness Plan is completed. The self-assessment process will evaluate and determine the acceptability of continued operation at specific milestones.

ATTACHMENT 2 ST-HL-AE- 477 4 PAGE 3 OF Le

OPERATING DATA REPORT

DOCKET NO. 50-499 UNIT 2 DATE Apr. 5, 1994 COMPLETED BY R.L. Hill TELEPHONE 512/972-7667

OPERATING STATUS

- 1. REPORTING PERIOD: 03/01/94-03/31/94 GROSS HOURS IN REPORTING PERIOD: 744
- CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800 MAX.DEPEND.CAPACITY (MWe-Net): 1250.6 DESIGN ELECTRICAL RATING (MWe-Net): 1250.6
- 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
 4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS M	ONTH	YR TO DATE	CUMULATIVE
5.	NUMBER OF HOURS REACTOR WAS CRITICAL		0	0	24756.2
6.	REACTOR RESERVE SHUTDOWN HOURS		0	0	0
7.	HOURS GENERATOR ON LINE		0	0	23733.8
8.	UNIT RESERVE SHUTDOWN HOURS	i v	0	0	0
9.	GROSS THERMAL ENERGY GENERATED (MWt)	* *	0	0	86433521
10.	GROSS ELECTRICAL ENERGY GENERATED (MWH)		0	0	29204590
11.	NET ELECTRICAL ENERGY GENERATED (MWH)		0	0	27735279
12.	REACTOR SERVICE FACTOR	0	.0%	0,0%	59.0%
13.	REACTOR AVAILABILITY FACTOR	0	.0%	0.0%	59.0%
14.	UNIT SERVICE FACTOR	0	.0%	0.0%	56.6%
15.	UNIT AVAILABILITY FACTOR	0	.0%	0.0%	56.6%
16.	UNIT CAPACITY FACTOR (Using MDC)	0	,0%	0.0%	52.9%
17.	UNIT CAPACITY FACTOR (Using Design MWe)	0	.0%	0.0%	52.9%
18.	UNIT FORCED OUTAGE RATE	100	.0%	100.0%	32.1%
19.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS	(TYPE,	DATE	AND DURATIO	N OF EACH):
	N/A				

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 05/23/94

AVERAGE DAILY UNIT POWER LEVEL

ATTACHMENT 2 ST-HL-AE- 4974 PAGE 4 OF 6

DOCKET NO. 50-499

UNIT 2

DATE Apr. 5, 1994

COMPLETED BY R.L. Hill

TELEPHONE 512/972-7667

MONTH MARCH

AVI	RAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
	0	17	0
	0	18	0
	0	19	0
	0	20	0
Netron	0	21	0
	0	22	0
	0	23	0
	0	24	0
	0	25	0
	0	26	0
	0	27	0
T _	0	28	0
	0	29	0
	0	30	0
	0	31	0

UNIT SHITTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-499

UNIT 2

DATE Apr. 5, 1994

COMPLETED BY R.L. Hill

TELEPHONE 512/972-7667

REPORT MONTH MARCH

No.	Date	Type	Duration (Hours)		Method of Shutting Down Reactor	Licensee Event Report #	Code	Component Code	Cause & Corrective Action to Prevent Recurrence
93-07	930524	P	744.0	F	4	N/A	N/A	N/A	Upon the resolution of the auxiliary feedwater system corrective actions and additional issues, the refueling and scheduled maintenance outage was resumed.

F: Forced S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Exam

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Cont. of Existing

Outage

5-Reduction

9-Other

IEEE 805-1983

IEEE 803A-1983

PORVs and Safety Valves Summary

ATTACHMENT ST-HL-AE-4-114 PAGE OF 6

There were no PORV or Safety Valves challenged during the reporting period.