M.S. Tuckman Vice President Naclear Operations (704)373-3851



DUKE POWER

December 3, 1990

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Subject: Oconee Nuclear Station

Docket No. 50-270

Second Ten Year Interval Request for Relief No. 90-06

Gentlemer.

Pursuant to 10CFR 50.55a, please find attached request for relief number 90-06 from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the impracticality of code repair being made during plant operation. This letter is a follow-up to prior approval from the NRR on November 30, 1990.

Very truly yours,

M. S. Tuckman

Mike S. Tuckman

LBJ/7

Attachment

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Mr. L. A. Wiens
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
One White Flint North, Mail Stop 9H3
Washington, D.C. 20555

W/O Diagram
Mr. S. D. Ebneter
Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta St., NW., Suite 2900
Atlanta, Georgia 30322

Mr. Heyward Shealy, Chief Bureau of Radiological Health S. C. Department of Health and Environmental Control 2600 Bull Street Columbia, S. C. 29201

Mr. P. H. Skinner NRC Resident Inspector Oconee Nuclear Station Duke Power Company Oconee Nuclear Station Second Ten Year Interval Request for Relief No. 90-06

Component for Which Relief is Requested:

(a) Name and Number:

Pinhole leak on the 1" socket weld pipe cap downstream of valve 2FDW-37.

- (b) Function: N/A
- (c) ISI Class/Duke Class:

ISI Class B/Duke Class F.

(d) IWV-2260 Valve Category:

N/A

(e) Design Pressure: 1275 psig Design Temp: 475°F

Material: 1-inch carbon steel

11. Reference Code Requirement that has been Determined to be Impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1980 Edition (with Addenda through Winter 1980) Article IWA-4400, which requires that repairs shall be made in accordance with this Article and IWC-4000 for Class 2 pressure retaining components.

III. Basis for Requesting Relief:

In order to perform a code repair to this component, Unit 2 would require an undesired shutdown/startup cycle. Generic Letter 90-05 dated June 15, 1990, addresses the applicability of temporary non-code repairs under these conditions with prior relief from the NRC.

- IV. Alternate Repair: The leak has been repaired temporarily by the use of a non-ASME code engineered mechanical clamp.
- V. Evaluation of Acceptability of Proposed Repair With Respect to the Level of Quality and Safety as Well as Public Health and Safety:

Generic Letter 90-05 addresses the applicability of a temporary non-code repair. The use of an engineered mechanical clamp is an acceptable temporary repair since an engineering evaluation indicates this type clamp is designed to meet the load-bearing requirements of the piping, assuming that the flaw is completely through the wall for 360°, that is, all around the pipe circumference, at the location of the flaw.

Page 2 Relief Request 90-06

VI. Implementation Schedule:

The temporary repair was implemented (per verbal authorization) on November 30, 1990. Permanent repair will be performed in compliance with Section XI of the ASME Boiler and Pressure Vessel Code, Article TWA-4000 during the next col nutdown of Unit 2 following the temporary repair of November 1, 1990.

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