



Public Service Company of Colorado

5909 East 38th Avenue, Denver, Colorado 80207

October 6, 1982
Fort St. Vrain
Unit No. 1
P-82440
FSV-3A

50267

Mr. Thomas M. Novak
Assistant Director for Operating Reactors
Division of Licensing
United States Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: FSV RECA Verification

REFERENCE: PSC Letter from Mr. F.E. Swart
to Mr. J.E. Miller, P-81054,
dated February 18, 1981

Dear Mr. Novak:

In the February 10, 1981 meeting between PSC/GA/NRC, PSC committed to evaluate a plant transient from a "temperature redistribution" using the RECA 3 code. Since 1981 there has only been one transient suitable for this analysis. The results of this analysis show that the RECA 3 code does accurately predict plant transient behavior, even in a temperature redistribution configuration.

Enclosed are 40 copies of "Fort St. Vrain Accident Reanalysis Code (RECA 3) Verification Following A Region Outlet Temperature Redistribution."

Should you have any questions, please contact Mr. M.H. Holmes at (303) 571-6711.

Very truly yours,

Lawrence Brey
H. L. Brey, Manager
Nuclear Engineering Division

HLB/JCS:pa

Enclosures

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