

PROCEDURE REQUEST FORM

1. Procedure Number FNP-0-ERP-8 Revision Number 24
 Procedure Title Notification Roster

- Safety Related Non-Safety Related
- New Procedure Request
- Procedure Revision, New Revision Number _____
Change of Intent
- Temporary Procedure Change, Effective until next permanent change, TCN 24A
- Temporary Procedure Change, Req'd. by Plant Conditions, TCN _____
- Temporary Procedure Change, One Time Use

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2. Change Summary

2.1 Procedure Page Numbers Affected by Change
Appendix 2

2.2 Description of Changes
to change reference from site emergency to site area emergency

2.3 Reason for Change
to implement changes required by the emergency plan

3. Prepared By Elverson Dennis, Chief Technician, 9-8-82
 Signature Title Date

4. Reviewed By William G. Simpson, Section Supervisor, 9-11-82
 Signature Title Date

5. Cross-Disciplinary/PORC Review

Group	Signature	Title	Date
<u>PORC</u>	<u>W. A. Hunt III</u>	<u>PM</u>	<u>9-21-82</u>
_____	_____	_____	_____
_____	_____	_____	_____

6. Temporary Change Approval (Signature/Date)

- Member Group Staff W. B. Bayne 19-11-82
- Shift Foreman
- Senior Reactor Operator C. D. ... 19-14-82
- Plant Manager W. A. Hunt III 19-21-82

7. Final Approval (Signature/Date, required within 60 days of temporary approval)

- Group Supervisor _____ / _____
- Plant Superintendent 8210190073 821001 _____ / _____
PDR ADDCK 05000348
- MSAER F PDR _____ / _____
- General Manager-Nuclear Generation _____ / _____
- _____ / _____
- Plant Manager W. S. Hunt III 19-21-82

FARLEY NUCLEAR PLANT
 NUCLEAR SAFETY EVALUATION CHECK LIST

- (1) UNIT Shared
- (2) CHECK LIST APPLICABLE TO: FNP-0-EP2 Revision 24 TCN 24A
- (3) SAFETY EVALUATION - PART A

The procedure, procedure change or modification to which this evaluation is applicable represents:

- (3.1) Yes No A change to the plant as described in the FSAR?
- (3.2) Yes No A change to procedures as described in the FSAR?
- (3.3) Yes No A test or experiment not described in the FSAR?
- (3.4) Yes No A change to the Technical Specifications?

If the answer to any of the above questions is "Yes," complete Item (4) and attach a 10CFR50.59 evaluation. If the answer to all of the above is "No," omit Item (4) and Item (9).

(4) SAFETY EVALUATION - PART B

- (4.1) Yes No Will the probability of an accident previously evaluated in the FSAR be increased?
- (4.2) Yes No Will the consequences of an accident previously evaluated in the FSAR be increased?
- (4.3) Yes No May the possibility of an accident which is different than any already evaluated in the FSAR be created?
- (4.4) Yes No Will the probability of a malfunction of equipment important to safety previously evaluated in the FSAR be increased?
- (4.5) Yes No Will the consequences of a malfunction of equipment important to safety different than any already evaluated in the FSAR be increased?
- (4.6) Yes No May the possibility of a malfunction of equipment important to safety different than any already evaluated in the FSAR be created?
- (4.7) Yes No Will the margin of safety as defined in the basis to any Technical Specification be reduced?

If the answer to any of the above questions is "Yes," an unreviewed safety question is involved. Explain the basis for each answer provided in Section 4.

(5) REMARKS: (Attach additional pages if necessary) _____

(6) PREPARED BY: Ely Don DATE 9-8-82
 (7) REVIEWED BY: William H. Sipes DATE 9-11-82
 (8) PORC REVIEW: W. S. Burt DATE 9-21-82
 (9) MORE REVIEW: _____ DATE _____

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APPENDIX 2

INITIAL MESSAGE

This is _____, the Emergency Director at Farley Nuclear Plant.

(Name)

1. This is to inform you that an emergency classified as:

- Notification of Unusual Event
- Alert *Area*
- Site Emergency
- General Emergency

TCN
24A

has occurred involving Unit(s) _____.

- 2. A release is not in progress.
- A release may be in progress.
- A liquid release is in progress.
- An atmospheric release is in progress.

3. <u>Atmospheric</u>	<u>Liquid</u>
Release Point _____	Release Point _____
Wind direction 35': (from) _____ ⁰ ; (to) _____ ⁰	Magnitude _____
Wind direction 150': (from) _____ ⁰ ; (to) _____ ⁰	
Wind speed 35': _____ mph ÷ 2 _____ meter/sec	
Wind speed 150': _____ mph ÷ 2 _____ meter/sec	

- 4. On-site situation (circle):
 - a. Evacuation of on-site personnel: Yes No Some
 - b. Recommended protective actions: None Shelter Evacuate
 - c. Assistance needed: Fire Police Ambulance Other
 - d. Prognosis of situation: Terminated Stable Worsening Other

Further information will be transmitted as soon as it is available.

PROCEDURE REQUEST FORM

1. Procedure Number FNP-0-EP-9 Revision Number 9
 Procedure Title Radiation exposure estimation and classification of emergencies

- Safety Related Non-Safety Related
- New Procedure Request
- Procedure Revision, New Revision Number _____
- Change of Intent
- Temporary Procedure Change, Effective until next permanent change, TCN 9A
- Temporary Procedure Change, Req'd. by Plant Conditions, TCN _____
- Temporary Procedure Change, One Time Use

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2. Change Summary

2.1 Procedure Page Numbers Affected by Change
3, Tab 3 p. 1 of 1

2.2 Description of Changes
to change from site emergency to site area emergency

2.3 Reason for Change
to implement changes required by the emergency plan

3. Prepared By Elmer Dorn, Top Technician, 9-8-82
 Signature Title Date

4. Reviewed By William G. Cipriotti, Sector Supervisor, 9-11-82
 Signature Title Date

5. Cross-Disciplinary/PORC Review

Group	Signature	Title	Date
<u>PORC</u>	<u>W.A. Hunt III</u>	<u>PM</u>	<u>9-21-82</u>

6. Temporary Change Approval (Signature/Date)

- Member Group Staff W.B. Bayne 19-11-82
- Shift Foreman
- Senior Reactor Operator C.D. Newell 19-14-82
- Plant Manager W.A. Hunt III 19-21-82

7. Final Approval (Signature/Date, required within 60 days of temporary approval)

- Group Supervisor _____
- Plant Superintendent _____
- MSAER _____
- General Manager-Nuclear Generation _____
- _____
- Plant Manager W.A. Hunt III 19-21-82

FARLEY NUCLEAR PLANT
 NUCLEAR SAFETY EVALUATION CHECK LIST

- (1) UNIT shared
 (2) CHECK LIST APPLICABLE TO: FNP-0-EP-9 Revision 9/ TCN 9A
 (3) SAFETY EVALUATION - PART A

The procedure, procedure change or modification to which this evaluation is applicable represents:

- (3.1) Yes No A change to the plant as described in the FSAR?
 (3.2) Yes No A change to procedures as described in the FSAR?
 (3.3) Yes No A test or experiment not described in the FSAR?
 (3.4) Yes No A change to the Technical Specifications?

If the answer to any of the above questions is "Yes," complete item (4) and attach a 10CFR50.59 evaluation. If the answer to all of the above is "No," omit Item (4) and Item (9).

(4) SAFETY EVALUATION - PART B

- (4.1) Yes No Will the probability of an accident previously evaluated in the FSAR be increased?
 (4.2) Yes No Will the consequences of an accident previously evaluated in the FSAR be increased?
 (4.3) Yes No May the possibility of an accident which is different than any already evaluated in the FSAR be created?
 (4.4) Yes No Will the probability of a malfunction of equipment important to safety previously evaluated in the FSAR be increased?
 (4.5) Yes No Will the consequences of a malfunction of equipment important to safety different than any already evaluated in the FSAR be increased?
 (4.6) Yes No May the possibility of a malfunction of equipment important to safety different than any already evaluated in the FSAR be created?
 (4.7) Yes No Will the margin of safety as defined in the basis to any Technical Specification be reduced?

If the answer to any of the above questions is "Yes," an unreviewed safety question is involved. Explain the basis for each answer provided in Section 4.

(5) REMARKS: (Attach additional pages if necessary) _____

(6) PREPARED BY: [Signature] DATE 9-8-82
 (7) REVIEWED BY: [Signature] DATE 9-11-82
 (8) PORC REVIEW: [Signature] DATE 9-21-82
 (9) MORE REVIEW: _____ DATE _____

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4.2 Emergency Classification Based on Dose Projections

4.2.1 General Emergency

4.2.1.1 Criteria: Based on lower limit of projected individual exposure at site boundary.

5 rem whole body exposure, or
10 rem thyroid exposure

4.2.1.2 Refer to EIP-19, General Emergency.

4.2.2 Site Emergency

4.2.2.1 Criteria: Based on lower limit of projected individual exposure at site boundary.

1.0 rem whole body exposure,
or 2.5 rem thyroid exposure

4.2.2.2 Refer to EIP-18, Site Emergency.

4.2.3 Alert

4.2.3.1 Criteria: 1 mr/hr at site boundary.

4.2.3.2 Refer to EIP-12, Alert

4.3 Emergency Classification Based on Plant Condition.

Refer to the Tab 3 for general criteria for classifying plant conditions. Refer to the indicated EIP for the exact criteria:

EIP-17, Notification of Unusual Event

EIP-12, Alert *Area*

EIP-18, Site Emergency

EIP-19, General Emergency

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4.4 Repeat steps 4.1 - 4.3 as necessary every hour; following any significant change in release rate; or if sample results indicate a significant change in dose factors (refer to RCP-25) or until the release is terminated.

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	Loss of Reactor Coolant	Loss of Secondary Coolant	S/G Tube Rupture	Degraded Core	Effluent	Security	Loss of Electrical Power
Notification of Unusual Event EIP-11		Outside Cmt. with ECCS activation		Core < 10°F subcooled	Radiological T.S. limits exceeded	Attempted sabotage or unauthorized entry	Both trains of AC or all diesel
Alert EIP-12	50gpm leakage (unmaintainable)	Inside Cmt. or outside Cmt. with MSIV failure or outside Cmt. with 10 gpm S/G tube leakage.	With ECCS activa- tion or >10gpm leak with steam break outside Cmt.	Clad damage indi- cated by RCS acti- vity. >100 $\mu\text{Ci/gm}$ equiv. 1-131.	>10 times radiological T.S. exceeded or either R-14, R-21, or R-22 reading offscale (sampling equilibria) or boundary	Actual or imminent threat of sabotage	LOSP and loss of all die- sels for <15 min. or loss of Aux. bldg. .C. for < 15 min.
Site Area EIP-13	With Cmt. press. >27 psig or rupture of a control rod housing.	Outside Cmt. with >50gpm tube leak- age and RCS acti- vity T.S. limit.	With LOSP and ECCS activation.	RCS AT> 64° and increasing or cor- e exit temp. >1200°F.	Projected offsite dose 1.0 Rem W.B. or 2.5 Rem Thyroid	Imminent takeover of plant	LOSP and loss of all die- sels for >15 min. or loss of both trains of Aux. bldg. .C. for >15 min.
General EIP-14	With fuel damage and potential loss of Cmt. or loss of Cmt integ. and potential fuel damage			With LOCA and po- tential loss of Cmt. integ. or with loss of Cmt integrity and po- tential LOCA	Projected offsite dose > 5 Rem WB or 10 Rem Thyroid	Loss of physical con- trol of plant.	
	Loss of Control Room Indication	Hi RCS Activity	RSE Equip. Failure	Fuel Damage/Inad- vertent Loading	Natural Emergencies	Hazards	Miscellaneous
Notification of Unusual Event EIP-17	Loss of NCB indication or annunciation to an extent requiring shut- down.	Exceeds Tech. Spec		Inadvertent load- ing of fuel caus- ing Pq to exceed tech. spec. limit	Any of following which affect site: Earthquake Tornado Hurricane Unusual River level	Any of following on site or <1 mile from site plant. effects ops Aircraft crash Exp. or fire Toxic gas Flamm. gas	Cmt. integ. tech. spec. exceeded. Loss of forced flow-3 loops. ECCS actuated. Safety or PORV fail to close (ps or S/G). Contain. indiv. transported.
Alert EIP-12	Loss of all NCB annunciators	>100 $\mu\text{Ci/gm}$ equiv. 1-131	Loss of both trains of either APV, BHR, SW, CCM or fail- ure of MSPs to initiate & com- plete trip.	Fuel damage with either R-2, R-11, R-12, R25 A or B reading off scale	Earthquake > OBE. Tornado striking fac. Hurricane winds near 115 mph. Unusual river level affecting ops.	Any of following effec- ting ops: Aircraft crash Toxic gas Flamm. gas or fire potentially affecting ECCS.	Rod ejection at power.
Site Area EIP-16	Loss of all NCB annu- ciators for >15 min. when either not in CSB or a significant transi- ent is in progress.	Fission product activity in RCS > 100 $\mu\text{Ci/gm}$ with po- tential loss of RCS or Cmt. integ.	Loss of functions required to achieve MSB	Fuel damage with projected dose 1.0 Rem WB or 2.5 Rem Thyroid	Earthquake > MSR Winds > 115 mph River level > or < design basis.	Any of following with plant not in CSB: Aircraft crash affect- ing vital structures. Toxic or flamm. gas into vital areas. Fire or exp. affecting ECCS or MSR equip.	Evacuation of Control Room
General EIP-19		>100 $\mu\text{Ci/gm}$ with LOCA & poten. loss of cmt. integ. or loss of cmt. int. & potential LOCA.					

TAB 3

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PROCEDURE REQUEST FORM

1. Procedure Number FNP-0-EIP-13 Revision Number 6
Procedure Title Fire Emergencies

- Safety Related Non-Safety Related
- New Procedure Request
- Procedure Revision, New Revision Number _____
Change of Intent
- Temporary Procedure Change, Effective until next permanent
change, TCN 6A
- Temporary Procedure Change, Req'd. by Plant Conditions, TCN _____
- Temporary Procedure Change, One Time Use

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2. Change Summary

2.1 Procedure Page Numbers Affected by Change
6, 8

2.2 Description of Changes
To change references from site emergency to site area emergency

2.3 Reason for Change
To implement changes required by the emergency plan

3. Prepared By Elwyn Danks, Shift Technician, 9-8-82
Signature Title Date

4. Reviewed By William J. Dipertoy, Sector Supervisor, 9-11-82
Signature Title Date

5. Cross-Disciplinary/PORC Review

Group	Signature	Title	Date
<u>PORC</u>	<u>W. J. Danks</u>	<u>PM</u>	<u>9-21-82</u>
_____	_____	_____	_____
_____	_____	_____	_____

6. Temporary Change Approval (Signature/Date)

- Member Group Staff W. J. Danks 9-11-82
- Shift Foreman
- Senior Reactor Operator C. D. ... 9-14-82
- Plant Manager W. J. Danks 9-21-82

7. Final Approval (Signature/Date, required within 60 day of temporary approval)

- Group Supervisor _____ / _____
- Plant Superintendent _____ / _____
- MSAER _____ / _____
- General Manager-Nuclear Generation _____ / _____
- _____ / _____
- Plant Manager W. J. Danks 9-21-82

FARLEY NUCLEAR PLANT
 NUCLEAR SAFETY EVALUATION CHECK LIST

- (1) UNIT Shared
- (2) CHECK LIST APPLICABLE TO: FNP-0-EIP-13 Revision 6 TCN 6A
- (3) SAFETY EVALUATION - PART A

The procedure, procedure change or modification to which this evaluation is applicable represents:

- (3.1) Yes No A change to the plant as described in the FSAR?
- (3.2) Yes No A change to procedures as described in the FSAR?
- (3.3) Yes No A test or experiment not described in the FSAR?
- (3.4) Yes No A change to the Technical Specifications?

If the answer to any of the above questions is "Yes," complete item (4) and attach a 10CFR50.59 evaluation. If the answer to all of the above is "No," omit item (4) and item (9).

(4) SAFETY EVALUATION - PART B

- (4.1) Yes No Will the probability of an accident previously evaluated in the FSAR be increased?
- (4.2) Yes No Will the consequences of an accident previously evaluated in the FSAR be increased?
- (4.3) Yes No May the possibility of an accident which is different than any already evaluated in the FSAR be created?
- (4.4) Yes No Will the probability of a malfunction of equipment important to safety previously evaluated in the FSAR be increased?
- (4.5) Yes No Will the consequences of a malfunction of equipment important to safety different than any already evaluated in the FSAR be increased?
- (4.6) Yes No May the possibility of a malfunction of equipment important to safety different than any already evaluated in the FSAR be created?
- (4.7) Yes No Will the margin of safety as defined in the basis to any Technical Specification be reduced?

If the answer to any of the above questions is "Yes," an unreviewed safety question is involved. Explain the basis for each answer provided in Section 4.

(5) REMARKS: (Attach additional pages if necessary) _____

(6) PREPARED BY:	<u>Sharon Lewis</u>	DATE	<u>9-8-82</u>
(7) REVIEWED BY:	<u>William H. Hixson</u>	DATE	<u>9-11-82</u>
(8) PORC REVIEW:	<u>W.A. Hunt</u>	DATE	<u>9-21-82</u>
(9) MORE REVIEW:		DATE	

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FIRE EMERGENCIES

1.0 Purpose

This procedure establishes the initial and subsequent action to be taken by personnel on the plant site in the event of fire in the Controlled Area or fire which could affect operations in the Controlled Area.

2.0 References

- 2.1 Joseph M. Farley Nuclear Plant Emergency Plan.
- 2.2 FNP-0-EIP-1, Duties of an Individual Who Discovers an Emergency Condition.
- 2.3 FNP-0-EIP-2, Duties of the Shift Supervisor.
- 2.4 FNP-0-EIP-3, Duties of the Emergency Director.
- 2.5 FNP-0-EIP-7, Security Support to the Emergency Plan.
- 2.6 FNP-0-EIP-8, Notification Roster.
- 2.7 FNP-0-EIP-9, Radiation Exposure Estimation and Classification of Emergencies.
- 2.8 FNP-0-EIP-10, Evacuation and Personnel Accountability.
- 2.9 FNP-0-AP-37, Fire Brigade Organization.
- 2.10 FNP-0-EIP-26, Offsite Notification
- 2.11 FNP-0-EIP-12, Alert
- 2.12 FNP-0-EIP-18, Site ^{Area} Emergency

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6A3.0 General

- 3.1 A Controlled Area evacuation will be effected by the Shift Supervisor or Emergency Director, as necessary, to protect the health and safety of personnel occupying the Controlled Area.
- 3.2 Any fire that occurs in a Radiation Controlled Area (RCA) shall be considered to involve radioactive material and monitoring by the Radiation Monitoring Team shall be required.
- 3.3 Except in unusual circumstances, the threat to life and property from fire exceeds that from radiation exposure. Radiation control should be considered, but should not be permitted to interfere with the fire fighting effort unless the radiological hazard is significant.

4.2.3 The APCo Fire Marshal shall:

4.2.3.1 Investigate the cause of the fire and the extent of damage.

4.2.3.2 Report findings to the Emergency Director.

4.2.3.3 Supervise recharging or replacing of all firefighting equipment and supplies.

4.2.4 The Emergency Director shall:

Be guided by the following procedures:

FNP-0-EIP-3, Duties of the Emergency Director

FNP-0-EIP-8, Notification Roster

FNP-0-EIP-9, Radiation Exposure Estimation and Classification of Emergencies

FNP-0-EIP-12, Alert (if applicable)

FNP-0-EIP-18, Site ^{Area} Emergency (if applicable)

FNP-0-EIP-26, Offsite Notification

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*Implemented if the fire involves a radiation hazard.

~~Rev. 6~~
TCN 6A

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-11

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HANDLING OF INJURED PERSONNEL

Approved:

W. J. Hunt
Plant Manager

Date Issued: 9-21-82

Date of Implementation: 9-21-82

Diskette EIP-4

List of Effective Pages

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HANDLING OF INJURED PERSONNEL

1.0 Purpose

This procedure provides guidelines for actions to be taken in the event of serious injury or in the event of any injury where radioactive contamination may exist.

2.0 References

- 2.1 Joseph M. Farley Nuclear Plant Emergency Plan
- 2.2 FNP-0-EIP-8, Notification Roster
- 2.3 Title 10, Code of Federal Regulations, Part 20
- 2.4 FNP-0-EIP-17, Notification of Unusual Event
- 2.5 FNP-0-EIP-19, Offsite Notification

3.0 General

- 3.1 Plant medical treatment of injured personnel shall consist of simple first aid rendered by qualified individuals.
- 3.2 At least one person on each operating crew of each unit shall be qualified to perform first aid.
- 3.3 Casualty decontamination should not take precedence over first aid in those cases requiring urgent medical treatment.
- 3.4 Decontamination will normally consist of removing contaminated clothing, washing of superficial abrasions, flushing of the eyes, etc.
- 3.5 Casualties with serious injuries such as concussions, fractures, lacerations, stretcher cases, etc., should be transferred to the hospital immediately.
- 3.6 Chemistry & Health Physics shall provide supervision as soon as possible after casualties have been sent to Southeast Alabama Medical Center, to provide assistance in the coordination of the receipt of casualties.
- 3.7 Any individual who administers first aid shall subsequently complete applicable sections of APCo Form Number 5-41273, Employee Injury Report, (Figure 1) and forward the completed form to the Chemistry and Health Physics Supervisor.

- 3.8 Notifications which could be required are shown in Figure 1A and are listed in EIP-26. Telephone numbers are listed in FNP-0-EIP-8.
- 3.9 Personnel contamination associated with medical care is considered a personnel emergency. On each occurrence individuals performing personnel decontamination shall complete CHP Form 202, Personnel Decontamination Record (Figure 2), and forward to the Chemistry and Health Physics Supervisor.
- 3.10 Actual or suspected radiation exposure casualties shall be grouped into the following classifications according to the listed criteria. For the purpose of these classifications trauma is defined as an injury or wound violently produced or the abnormal emotional condition resulting from the injury or wound. Action guidelines are listed for the disposition of such casualties:
- 3.10.1 Class 1 Radiation Exposure Casualty
- 3.10.1.1 Exposure criteria:
- a. Estimated radiation dose greater than applicable 10CFR20 limits but less than 5 rem to whole body (including eyes, gonads, and bloodforming organs); or
 - b. Estimated radiation dose to the skin of the whole body greater than the 10CFR20 limit but less than 30 rem; or
 - c. Estimated radiation dose to the feet, ankles, hands, or forearms greater than the 10CFR20 limit but less than 75 rem.
- 3.10.1.2 Action guidelines:
- a. Without trauma-send to Southeast Alabama Medical Center (SAMC) for evaluation after clearance by Health Physics for contamination.

- b. With trauma-appropriate first aid then send to SAMC for evaluation.

3.10.2 Class 2 Radiation Exposure Casualty

3.10.2.1 Exposure criteria:

- a. Estimated radiation dose to the whole body (including eyes, gonads, and blood-forming organs) greater than 5 rem but less than 25 rem; or
- b. Estimated radiation dose to the skin of the whole body greater than 30 rem but less than 150 rem; or
- c. Estimated radiation dose to the feet, ankles, hands, or forearms greater than 75 rem but less than 375 rem.

3.10.2.2 Action guidelines:

- a. Without trauma - sent to SAMC for evaluation after clearance by Health Physics for contamination.
- b. With trauma - appropriate first aid, then send to SAMC for evaluation.

3.10.3 Class 3 Radiation Exposure Casualty

3.10.3.1 Exposure criteria:

- a. Estimated radiation dose to the whole body (including eyes, gonads, and blood-forming organs) of 25 rem or more; or
- b. Estimated radiation dose to the skin of the whole body of 150 rem or more; or

- c. Estimated radiation dose to the feet, ankles, hands, or forearms of 375 rem or more.
- d. Internal radiation exposure estimated to be significant.

3.10.3.2 Action guidelines:

- a. Without trauma - after proper decontamination by Health Physics, send directly to the Radiation Casualty Treatment Facility (RCTF) in Birmingham.
- b. With trauma - appropriate first aid, decontamination, and if necessary for life-saving surgical problems, local hospital treatment, then transfer to RCTF.

3.10.4 Monitoring and decontamination is desirable prior to sending a casualty to a hospital.

3.11 The following modes of transportation are available for transporting casualties to medical treatment facilities:

3.11.1 Plant to SAMC, RCTF, Oak Ridge Associated Universities (ORAU):

3.11.1.1 Plant Emergency Vehicle (PEV).

3.11.1.2 Any other Alabama Power Company (APCo) vehicle.

3.11.1.3 Ambulance Service Company, Dothan, Alabama.

3.11.2 SAMC to RCTF or Birmingham Municipal Airport:

3.11.2.1 Ambulance Service Company, Dothan, Alabama.

3.11.2.2 U. S. Army Aviation Center, Fort Rucker, Alabama.

- 3.11.3 Birmingham Municipal Airport to RCTF:
 - 3.11.3.1 A & A Ambulance Service,
Birmingham, Alabama.
- 3.11.4 SAMC to ORAU:
 - 3.11.4.1 Ambulance Service Company,
Dothan, Alabama.
 - 3.11.4.2 U. S. Army Aviation Center,
Fort Rucker, Alabama (Fixed
wing transportation from
Dothan Airport).
- 3.11.5 RCTF to ORAU:
 - 3.11.5.1 A & A Ambulance Service,
Birmingham, Alabama.

4.0 Procedure

- 4.1 The individual who discovers an injured person or witnesses an injury shall:
 - 4.1.1 Render any assistance and first aid which you are qualified to perform.
 - 4.1.2 Notify the Control Room giving your name, the location and number of injured personnel, extent of injuries if known and any other pertinent information which could affect plant operations. Notification of the Control Room may occur before 4.1.1 above based on the judgement of the individual.
- 4.2 The Shift Supervisor shall consider the following actions in conjunction with initiating the Personnel Emergency Checklist (FNP-0-EIP-11A):
 - 4.2.1 Announce the emergency on the PA system and dispatch a qualified individual(s) to perform first aid.
 - 4.2.2 Implement EIP-17, Notification of Unusual Event if a contaminated, injured individual is transported to an offsite facility.
 - 4.2.3 Dispatch a Radiation Monitoring Team to survey if radiation exposure or contamination is possibly involved.

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- 4.2.4 Summon a local ambulance if the Plant Emergency Vehicle is not available giving the number of injured personnel and whether or not radioactive materials are involved.
- 4.2.5 Inform Central Security Control (CSC) to escort the ambulance when it arrives at the plant site to the location of the injured personnel.
- 4.2.6 If contamination is involved, direct a Chemistry and Health Physics technician or another individual deemed competent in radiological monitoring to accompany the injured person(s) to SAMC.

NOTE: It may be desirable to notify a C & HP technician at home to meet the emergency vehicle at SAMC.

- 4.2.7 Notify SAMC and the company doctor on call giving the following information:
 - 4.2.7.1 Number of casualties.
 - 4.2.7.2 Whether or not radioactive material is involved.
 - 4.2.7.3 Level of contamination, if known.
 - 4.2.7.4 Nature of injury.
 - 4.2.7.5 Estimated time of arrival.
 - 4.2.7.6 Any other pertinent information.
 - 4.2.7.7 After information is given, the hospital will call back to FNP to verify an actual emergency exists.
- 4.2.8 Ensure that the Emergency Director is notified.

- 4.3 The Emergency Director shall, if not previously performed, provide for the following in conjunction with, completing the Personnel Emergency Checklist (FNP-0-EIP-11A):

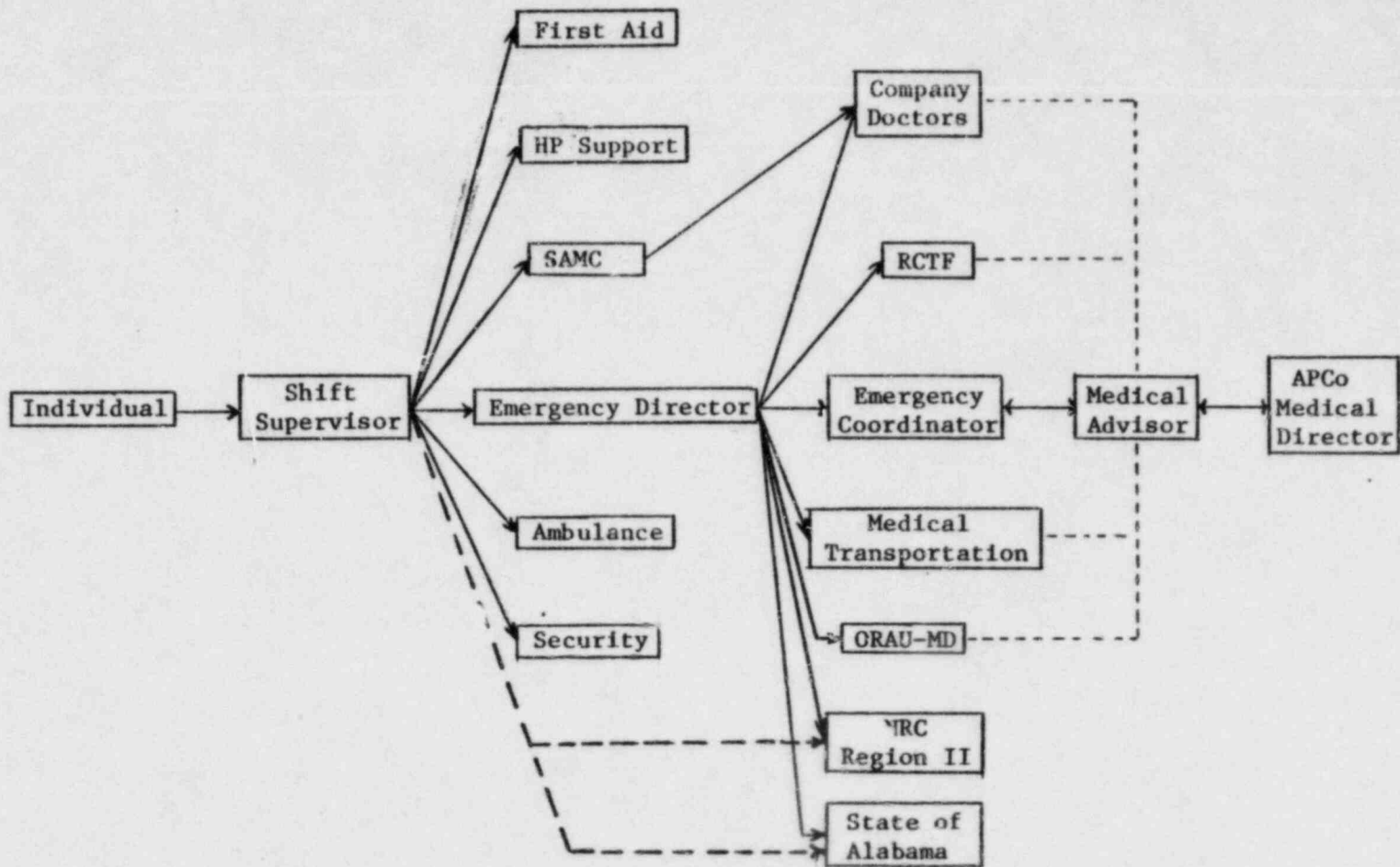
Rev. 4

- 4.3.1 Removal of personnel from a hazardous area (high radiation or contamination).
- 4.3.2 Administration of first aid for severe physical injuries.
- 4.3.3 Personnel decontamination.
- 4.3.4 Evaluation of radiation exposures.
- 4.3.5 In the event of mass casualties, determine which casualties will be sent to SAMC, RCTF, or directly to REACTS.
- 4.3.6 Notification per EIP-26 in the event of personnel contamination or overexposure.
- 4.4 Person(s) assigned to administer first aid to a casualty shall:
 - 4.4.1 Report to the accident with the first aid kit which is available in both First Aid Rooms and Emergency Cabinets.
 - 4.4.2 Render first aid to the casualty.
 - 4.4.3 If the casualty is to be sent to the hospital, attach a hospital wrist band with casualty's name and I.D. badge number and if radioactive contamination is NOT involved, remove the casualty's I.D. badge and personnel dosimetry devices.
 - 4.4.4 Subsequently, complete applicable sections of APCo Form Number 5-41273, Employee Injury Report, and forward the completed form to the Chemistry and Health Physics Supervisor.
- 4.5 If Health Physics support is required, the Radiation Monitoring Team shall:
 - 4.5.1 Establish the degree of contamination and exposure of the patient.
 - 4.5.2 Establish protective clothing requirements for first aid personnel, and/or ambulance personnel.
 - 4.5.3 Decontaminate casualty if appropriate and/or minimize the spread of contamination.

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- 4.5.4 Read the casualty's personnel dosimetry devices as soon as possible to determine the radiation casualty classification, if a radiation injury is involved. If the casualty is highly contaminated and injuries require immediate transfer to the hospital, place an unexposed TLD in a plastic packet on the contaminated area.
 - 4.5.5 Prepare the casualty for transportation to a medical treatment facility if necessary.
 - 4.5.6 Assist the hospital staff as required.
 - 4.5.7 Periodically inform the Emergency Director as to the casualty's disposition.
 - 4.5.8 Detain ambulance and its attendants at hospital until properly monitored and decontaminated.
 - 4.5.9 Obtain personnel dosimetry devices and other APCo property from ambulance personnel when they are released.
 - 4.5.10 If a vehicular accident should occur enroute to SAMC and the Plant Emergency Vehicle or ambulance were to remain unattended, lock the vehicle and if radioactive materials are involved, placard all four sides with the sign "RADIOACTIVE".
- 4.6 Off-site ambulance personnel shall:
- 4.6.1 Meet guard at the APCo gate and follow to the CSC Building.
 - 4.6.2 Obtain ambulance kit and appropriate personnel dosimetry devices at the CSC Building.
 - 4.6.3 Proceed with guard to the location of the casualty.
 - 4.6.4 Transport casualty to the medical treatment facility indicated by APCo personnel who are with the casualty.
 - 4.6.5 Remain at the medical facility with the ambulance until monitored and released by APCo Health Physics personnel.

(Figure 1A)



PERSONNEL EMERGENCY CHECKLIST

Handling of Injured Personnel

<u>Action</u>	<u>Initials</u>
1. Announce emergency and dispatch individual(s) to render first aid.	_____
2. Investigate situation for possible higher level emergency.	_____
3. Dispatch Radiation Monitoring Team if radiation exposure or contamination is possible involved.	_____
4. Summon Plant Emergency Vehicle or local ambulance service giving the number of injured personnel and whether or not radioactive materials are involved.	_____
5. Inform Central Security Control to escort the ambulance.	_____
6. If contamination is involved, send a C & HP Technician or other individual competent in radiological monitoring to accompany the injured person(s) to Southeast Alabama Medical Center (SAMC). Implement EIP-17	_____
7. Notify SAMC and the company doctor on call giving the following information.	_____
a. Number of casualties.	_____
b. Whether or not radioactive materials are involved.	_____
c. Level of contamination.	_____
d. Nature of injury.	_____
e. Estimated time of arrival.	_____
f. Other pertinent information.	_____
8. In the event of mass casualties decide which casualties will be sent to SAMC, RCTF, or directly to REACTS.	_____
9. Notify agencies per EIP-26	_____

EMPLOYEE INJURY REPORT

This report is for reporting work related injuries when it is known that an injury did occur. If there is doubt that the injury is a company responsibility, make a report in letter form to the Safety Department. If subsequent facts define it as a company responsibility, the letter will be supplemented by reporting on this form.

Case or File No. _____

Report No. _____
(SAFETY DEPT. USE ONLY)

IF IT IS BELIEVED INJURY WILL INVOLVE LOST TIME, ADVISE SAFETY DEPARTMENT IMMEDIATELY.

Forward Original To Safety Department

WHO WAS INJURED?	Employee No. _____ Social Security No. _____ PAYROLL NAME _____ Home Address _____ Classification _____ Length of Service _____ Age _____ Sex _____ This information required only in lost-time injuries. Married? _____ Living with spouse? _____ No. of Children _____ UNDER 18 YRS Other Dependents _____ RELATION _____
TIME AND PLACE OF INJURY	Date of Injury _____ Time of Injury _____ A.M. P.M. Division, Plant, Organization _____ Department _____ Location _____ Give further detail such as place in plant, substation, or location on line where injury occurred _____
NATURE OF INJURY	Nature of Injury _____ Specific Location of Injury on Body _____
WHAT WAS DONE FOR INJURED?	Was First Aid given? _____ If so what? _____ Name and location of Doctor _____ Date sent to Doctor _____ Name and location of Hospital if applicable _____ Did Doctor permit injured to return to work? _____ When? _____ Is work slip attached, dated & signed? _____ Restricted Duty? _____ Unrestricted Duty? _____ (Send all Return To Work slips to Safety Department after signing and dating.)
THIS SECTION OF REPORT TO BE COMPLETED BY FOREMAN OR SUPERVISOR OF INJURED <small>(Do not delay sending report if injured is not physically able to complete his/her part.)</small>	Name the machine, tool, equipment or substance involved _____ Was above defective in any way? _____ If so, state in what way _____ Cause(s) of injury _____ Describe circumstances, including names of witnesses, surrounding injury based on your investigation. Attach additional statement and sketches if necessary _____ Suggested corrective action _____ Date of Report _____ Signed _____ FOREMAN OR SUPERVISOR TITLE _____
INJURED'S DESCRIPTION OF INCIDENT RESULTING IN INJURY <small>(DO NOT SAY "See above for Description")</small>	Give a clear description of how you received injury, including names of witnesses. Use other side of report if more space is needed. _____ _____ _____ _____ Date _____ Signed _____ INJURED EMPLOYEE

JOSEPH M. FARLEY
NUCLEAR PLANT

PERSONNEL DECONTAMINATION RECORD

Name _____ Date _____

Social Security NO. _____ Time _____

Security Badge No. _____ Plant Group _____

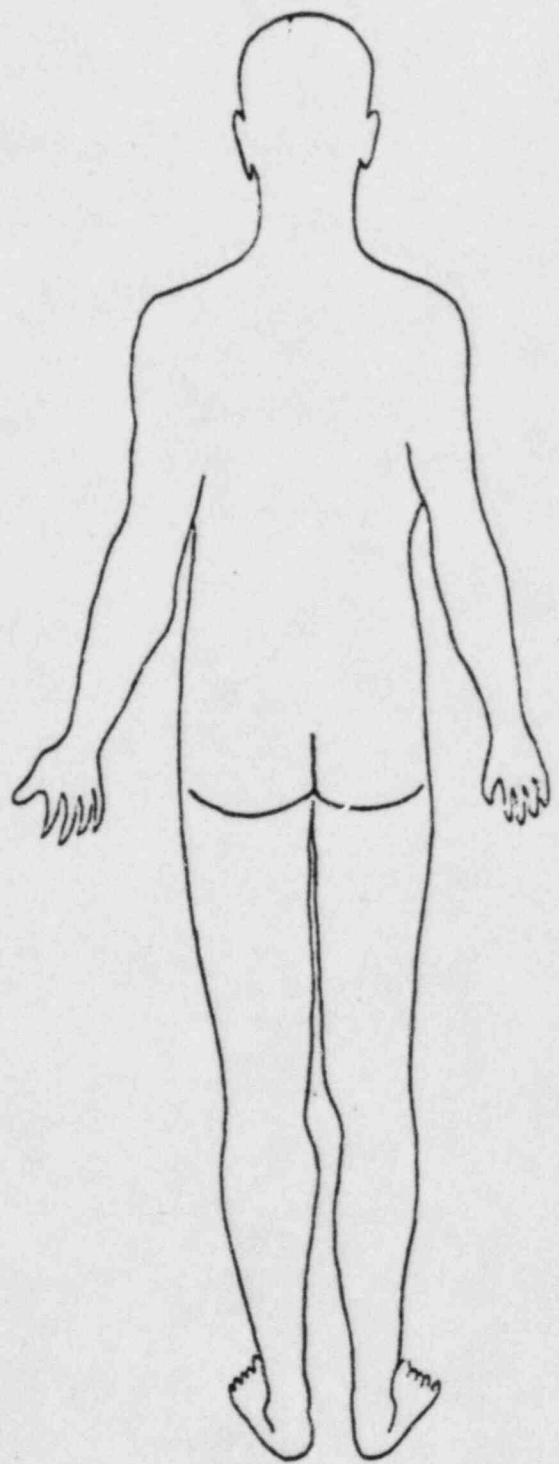
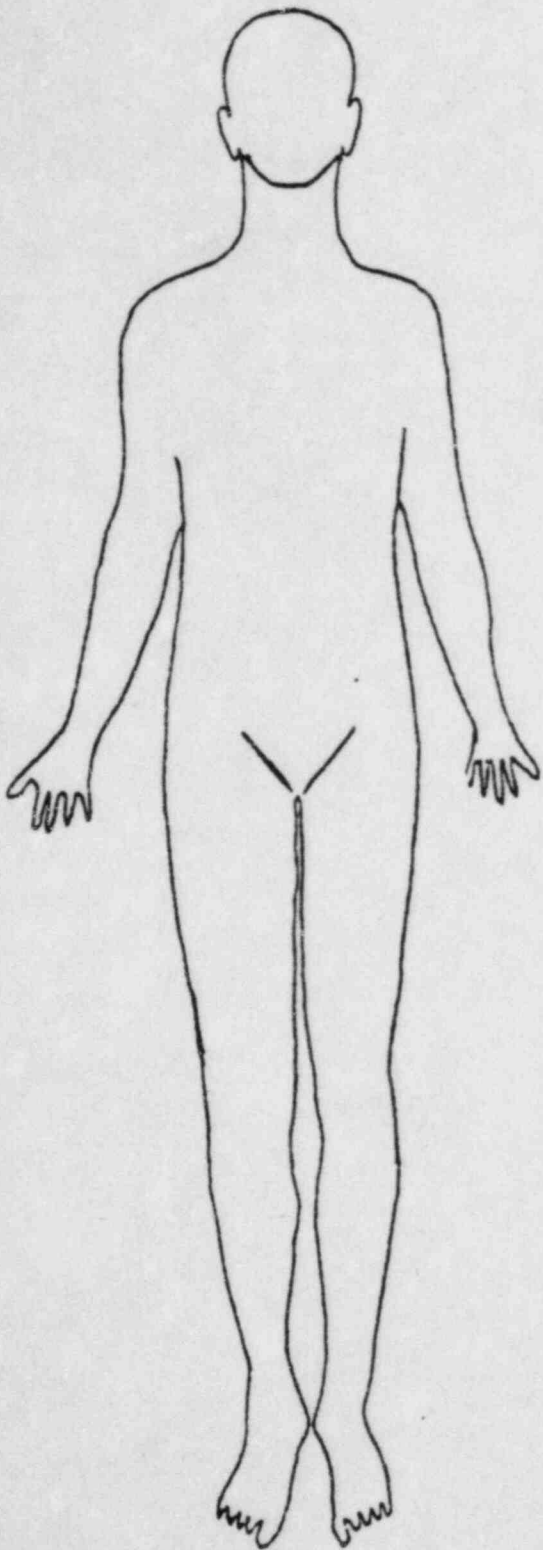
Area Where Contamination Occurred: _____

<u>Body Location</u>	<u>Instrument</u>	<u>Reading</u>	<u>Decon Agent</u>	<u>Skin Condition</u>	<u>Reading</u>
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
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_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____

Remarks: _____

Decontamination Performed By: _____

Title: _____



Location of Contamination

(Figure 2)
Back

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-18

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SITE AREA EMERGENCY

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Approved:

W. S. Hunt
Plant Manager

Date Issued: 9-21-82

Date of Implementation: 9-21-82

Diskette #EIP-1

<u>List of Effective Pages</u>	
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SITE AREA EMERGENCY

1.0 Purpose

This procedure defines the criteria for classifying an emergency as a Site Area Emergency, delineates personnel and organizations who may be notified and lists actions which may be taken to mitigate the effects of the emergency.

2.0 References

- 2.1 Joseph M. Farley Nuclear Plant Emergency Plan.
- 2.2 FNP-0-EIP-8, Notification Roster.
- 2.3 FNP-0-EIP-9, Radiation Exposure Estimation and Classification of Emergencies.
- 2.4 FNP-0-EIP-10, Evacuation and Personnel Accountability.
- 2.5 FNP-0-EIP-13, Fire Emergencies.
- 2.6 FNP-0-EIP-14, Re-entry Procedures.
- 2.7 FNP-0-EIP-26, Offsite Notification

3.0 General

3.1 Description

The classification of Site Area Emergency applies to those events which are in progress or have occurred that involve actual or likely major failures of plant functions needed for protection of the public from radiation or contamination. The potential for release of radioactive material for the Site Area Emergency classification is up to 1000 Ci of I-131 equivalent or 10^4 to 10^6 Ci of Xe-133 equivalent. The purpose of the declaration of a Site Area Emergency is to:

- (a) Assure that response centers are manned,
- (b) Assure that monitoring teams are dispatched,
- (c) Assure that personnel involved in an evacuation effort of near-site areas are at their duty stations if the situation worsens, and,
- (d) Provide current information for and consultation with offsite authorities and the public.

3.2 Criteria

A Site Area Emergency would be declared for plant conditions that warrant activation of emergency centers and monitoring teams. Specifically a Site Area Emergency would be declared for any of the following:

3.2.1 A major loss of primary coolant as indicated by:

- (a) Decreasing pressurizer pressure and possible level, AND
- (b) Near normal steam pressure in all steam generators accompanied by,
 - (1) Containment pressure reaching 27 psig, AND
 - (2) High containment radiation (R-2, R-11, and R-12 reaching their alarm setpoint), AND
 - (3) High containment sump (recirculation) level AND
 - (4) High containment humidity.

3.2.2 Degraded core conditions with possible loss of core geometry as indicated by:

- (a) ΔT between RCS wide range hot leg and cold leg temperature $>64^{\circ}\text{F}$ and core exit temperature (in-core thermocouples) reading greater than 800°F and increasing, OR
- (b) Core exit temperature (in-core thermocouples) $> 1200^{\circ}\text{F}$.

3.2.3 A loss of offsite power and a steam generator tube rupture as indicated by:

- (a) ECCS actuation, AND

Rev. 5

- (b) High secondary coolant activity (R-15 or R-19 reach full scale)
- 3.2.4 Greater than 50 gpm primary to secondary leak, fuel damage as evidenced by a reactor coolant activity greater than technical specifications, and a steam line break outside containment as indicated by:
- (a) Abnormally low steam pressure on one or all steam generators with one or more of the following:
- (1) Steam line high flow,
 - (2) Steam line high differential pressure,
 - (3) Steam flow greater than feed flow

AND

- (b) No abnormal temperature or humidity increase in containment,
- 3.2.5 Loss of offsite power with a failure of all emergency AC power for more than 15 minutes.
- 3.2.6 Loss of both trains of auxiliary building DC power for more than 15 minutes.
- 3.2.7 Loss of functions for achieving hot standby.
- 3.2.8 Spent fuel handling accident for which sampling or radiation monitors indicate a projected lower limit of offsite individual exposure to be:

1.0 Rem - Whole Body or

2.5 Rem Thyroid

as a result of one of the following:

- (a) Dropped spent fuel assembly,
OR

- (b) An object is dropped onto a spent fuel assembly, OR
 - (c) A cask containing a spent fuel assembly is dropped exposing the assembly, OR
 - (d) A spent fuel assembly is deformed as a result of any manipulation, OR
 - (e) Spent fuel pool water level below top of assemblies.
- 3.2.9 A fire affecting ECCS.
- 3.2.10 Loss of all main control board annunciator capability for more than 15 minutes while:
- (a) Plant is not in cold shutdown, OR
 - (b) Significant plant transient is initiated while all alarms lost.
- 3.2.11 Imminent loss of physical control of the plant (i.e., takeover by terrorists, anti-nuclear factions, etc.).
- 3.2.12 Severe natural phenomena being experienced or projected with plant not in cold shutdown:
- (a) Earthquake greater than SSE levels
 - (b) Flood, low river water, or hurricane surge greater than design levels.
 - (c) Winds in excess of 115 mph.
- 3.2.13 Other hazards being experienced with the plant not in cold shutdown as follows:
- (a) Aircraft crash affecting vital structures by fire or impact, OR
 - (b) Severe damage to safe shutdown equipment from missiles or explosion, OR

- (c) Entry of toxic or flammable gases into vital areas.
- 3.2.14 Evacuation of the control room.
- 3.2.15 Any event such as a waste gas decay tank rupture for which sampling or radiation monitors indicate a projected lower limit of offsite individual exposure to be:
 - 1.0 Rem - Whole Body OR
 - 2.5 Rem - Thyroid
- 3.2.16 Rupture of a control rod mechanism housing as indicated by the following:
 - (a) Rod position indication, AND
 - (b) High RCS pressure surge, AND
 - (c) Momentary nuclear power surge, AND
 - (d) Subsequent behavior indicating a loss of primary coolant.

4.0 Procedure

- 4.1 The Shift Supervisor shall perform the following:
 - 4.1.1 Sound the Plant Emergency Alarm and announce the condition and give evacuation instructions over the plant public address system.
 - 4.1.2 Implement EIP-26, Offsite Notification
 - 4.1.3 If emergency is a fire, also refer to EIP-13, Fire Emergency
 - 4.1.4 The Shift Supervisor shall perform the duties of the Emergency Director until his arrival and assumption of duties.
- 4.2 The Emergency Director shall perform the following:
 - 4.2.1 Upon receiving notification of an emergency, provide instructions for the Administrative Aide to notify the Emergency Coordinator and those portions of the plant call list for the Emergency Organization as

directed by the Emergency Director, to include the Technical Support Center, Operations Support Centers, and Emergency Operations Facility, as necessary.

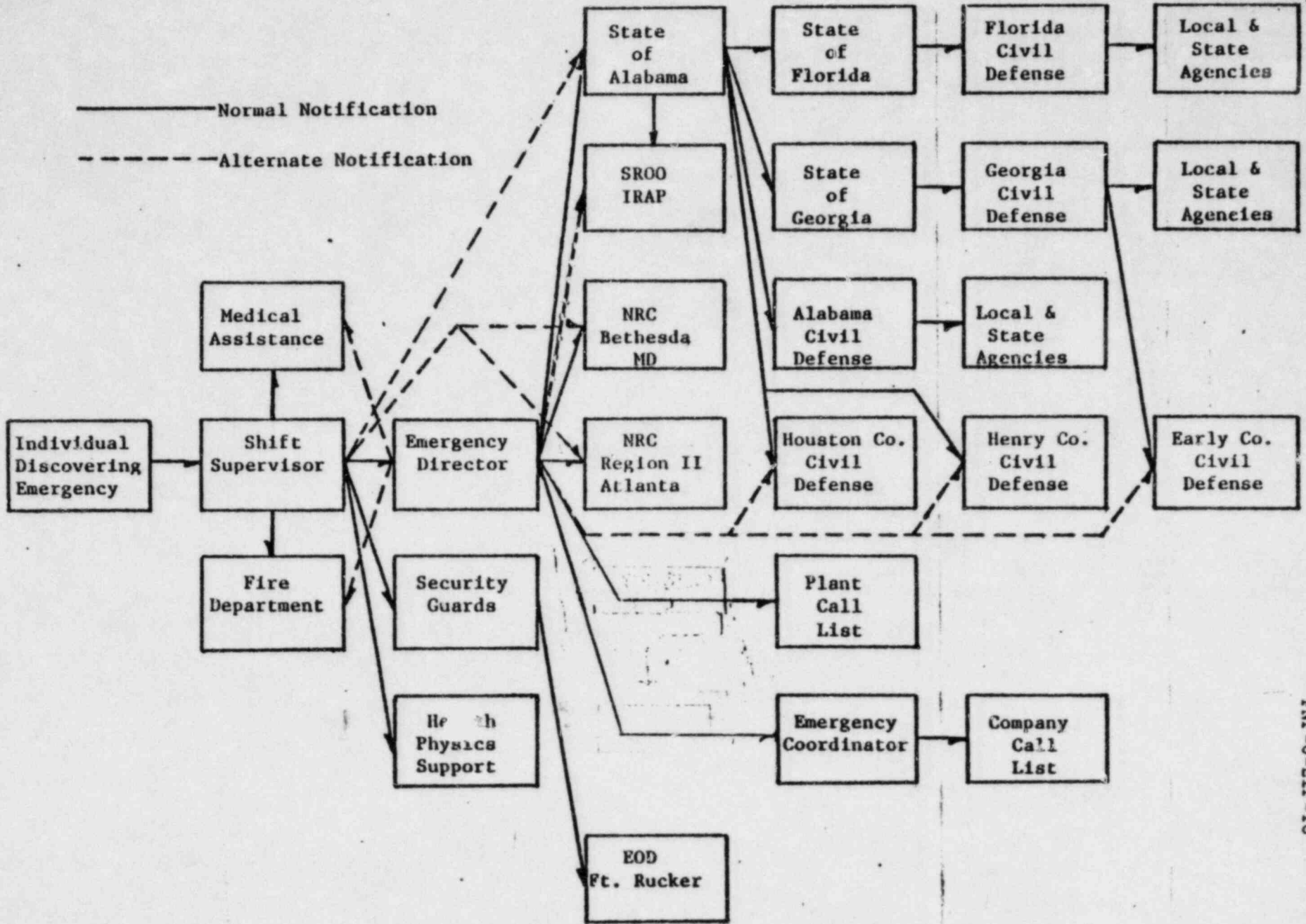
- 4.2.2 Implement notifications per EIP-26.
- 4.2.3 Activate the Technical Support Center and Operations Support Centers
- 4.2.4 Ensure personnel accountability (EIP-10).
- 4.2.5 Plan and initiate re-entries per EIP-14.
- 4.2.6 Dispatch Radiation Monitoring Teams. If additional support is required refer to EIP-8.
- 4.2.7 Provide periodic meteorological and dose estimates and release projections based on plant conditions and foreseeable contingencies to offsite authorities.
- 4.2.8 Coordinate with the Recovery Manager with respect to information to be released to the press and recovery planning.
- 4.2.9 Coordinate with the Recovery Manager to send a company representative to the Houston County Central Emergency Operations Center (CEOC).
- 4.2.10 Continually reassess the emergency condition to ensure that a higher classification does not exist.
- 4.2.11 Close out or recommend reduction in emergency class by briefing of offsite authorities and by phone followed by written report as required by technical specifications; or escalate to a General Emergency.

Rev. 3

SITE AREA EMERGENCY CHECKLIST

- | | <u>Initials</u> |
|--|-----------------|
| I. Shift Supervisor | |
| A. Sound PEA, if necessary announce condition and give evacuation instructions | _____ |
| B. Implement EIP-26 | _____ |
| II. Emergency Director | |
| A. Provide instructions for the Administrative Aide to notify TSC staff, and the Emergency Coordinator | _____ |
| B. Implement EIP-26 | _____ |
| C. Initiate environmental sampling | _____ |
| D. Ensure personnel accountability (EIP-10). | _____ |
| E. Plan and initiate re-entries (EIP-14). | _____ |
| F. Coordinate with Recovery Manager sending company representative to Houston County CEOC. | _____ |
| G. Reassess conditions for possible upgrade to General Emergency | _____ |

FIGURE 1



Notification Order - Site Area Emergency