

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

April 15, 1994
LIC-94-0069

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: March 1994 Monthly Operating Report (MOR)

Enclosed is the March 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/mah

Enclosures

- c: LeBoeuf, Lamb, Greene & MacRae
- L. J. Callan, NRC Regional Administrator, Region IV
- S. D. Bloom, NRC Project Manager
- R. P. Mullikin, NRC Senior Resident Inspector
- R. T. Pearce, Combustion Engineering
- R. J. Simon, Westinghouse
- Office of Management & Program Analysis (2)
- INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

MARCH 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power level through the month of March. Uniform Rating of Generating Equipment (URGE) testing was successfully completed to demonstrate generating capacity for Midcontinent Area Power Pool (MAPP) annual accreditation.

Normal plant surveillance and maintenance activities were conducted during the month. Maintenance activities on the Circulating Water System included complete overhaul of Traveling Screens CW-2E and CW-2F.

Erosion/corrosion holes in Bearing Water Cooler Heat Exchanger CW-6B were repaired. Wall thickness checks are being performed on the other bearing water cooler heat exchanger to identify potential failure points.

Main Feedwater Pump FW-4B was out of service at the end of the month after its motor was observed to be sparking and experiencing excessive vibration. The motor required a motor bearing housing alignment and a magnetic center alignment.

Work on the radiation monitor upgrade modification continued through the month.

Preparations are in progress for the upcoming reracking of the Spent Fuel Pool (SFP) later in 1994. The new racks will maintain sufficient storage capacity to the year 2007.

The following NRC inspections were completed during this reporting period:

<u>IER No.</u>	<u>Description</u>
94-05	Motor Operated Valves
94-07	Monthly Resident Inspection
94-08	Special Inspection Regarding ESF Supervisory Relay Failure (LER 94-001)
94-10	Security

The following LER was submitted during this reporting period:

LER No. Description

94-001 Safeguards Actuation and Subsequent Reactor Trip Due to a Relay Failure

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of March, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Besides changes observed during normal plant transients and periodic increases from charging pump packing leaks, the Reactor Coolant System leak rate remained steady throughout the month of March at a nominal 0.10 gpm. No degrading trends were noted.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. Description

16i The amendment revised the pressure-temperature limits in Technical Specification 2.1.2 and made the limits valid up to 20 effective full power years of operation. The amendment also changed the minimum requirements for starting a non-operating reactor coolant pump and the requirements for disabling high-pressure safety injection pumps. Additionally, the amendment revised the operation of the PORVs consistent with Generic Letter 90-06.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE

- Replaced a defective circuit board for Power Range Control Channel 9 (AI-31E-AW1).
- Completely overhauled Charging Pump CH-1C.
- Installed a new breaker (MCC-4A2-004) for Boric Acid Pump CH-4B.
- Replaced Pressure Control Switch PCS-2947 for the suction valve for Low Pressure Safety Injection Pump SI-1A.
- Installed a new EGS Power Supply (PQ-309) for the High Pressure Safety Injection (HPSI) discharge header pressure loop.
- Installed a new EGS Power Supply (PQ-310) for the alternate HPSI header pressure loop.

- Removed the orifice plate upstream of Raw Water Outlet Valve HCV-2881B on Component Cooling Water Heat Exchanger AC-1B.
- Replaced Raw Water Pump AC-10B.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE APRIL 05, 1994
COMPLETED BY M. A. HOWMAN
TELEPHONE 402-533-6939

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: MARCH 1994
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A
9. Power Level to which restricted, if any (Net MWe): N/A
10. Reasons for restrictions, if any:
N/A

NOTES

	THIS MONTH	YR-TO-DATE	CUMULATIVE
	-----	-----	-----
11. Hours in Reporting Period.....	744.0	2160.0	179834.0
12. Number of Hours Reactor was Critical	744.0	2126.2	139817.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	2111.1	138174.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113268.0	3049290.4	182335090.0
17. Gross Elec. Energy Generated (MWH)..	378706.0	1033914.0	60121938.2
18. Net Elec. Energy Generated (MWH)....	362150.9	986689.8	57359752.7
19. Unit Service Factor.....	100.0	97.7	76.8
20. Unit Availability Factor.....	100.0	97.7	76.8
21. Unit Capacity Factor (using MDC Net)	101.8	95.6	69.1
22. Unit Capacity Factor (using DER Net)	101.8	95.6	67.4
23. Unit Forced Outage Rate.....	.0	2.3	4.2

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE APRIL 05, 1994
COMPLETED BY M. A. HOWMAN
TELEPHONE 402-533-6939

MONTH MARCH 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	487	17	486
2	487	18	486
3	486	19	486
4	486	20	488
5	487	21	487
6	487	22	487
7	487	23	487
8	487	24	487
9	487	25	487
10	487	26	487
11	487	27	487
12	487	28	487
13	487	29	487
14	487	30	487
15	486	31	487
16	485		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE April 8, 1994
COMPLETED BY M. A. Howman
TELEPHONE (402) 533-6939

REPORT MONTH March 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None									During March 1994, the plant operated at a nominal 100% power.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

⁴
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending March 31, 1994

- | | |
|---|-----------------------------|
| 1. Scheduled date for next refueling shutdown. | <u>March 11, 1995</u> |
| 2. Scheduled date for restart following refueling. | <u>April 29, 1995</u> |
| 3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? | <u>No</u> |
| a. If answer is yes, what, in general, will these be? | <u>N/A</u> |
| b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. | <u>No</u> |
| c. If no such review has taken place, when is it scheduled? | <u>Prior to April 1995</u> |
| 4. Scheduled date(s) for submitting proposed licensing action and support information. | <u>No submittal planned</u> |
| 5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. | <u>**</u> |
| 6. The number of fuel assemblies: | |
| a) in the core | <u>133 Assemblies</u> |
| b) in the spent fuel pool | <u>570 Assemblies</u> |
| c) spent fuel pool storage capacity | <u>729 Assemblies</u> |
| d) planned spent fuel pool storage capacity | <u>1083 Assemblies</u> |
| 7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. | <u>1995 Outage*</u> |

* Capability of full core offload of 133 assemblies lost. Reracking began in March and is scheduled for completion in August 1994.

** OPPD is planning to utilize CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by Keri Holth Date 4-11-94