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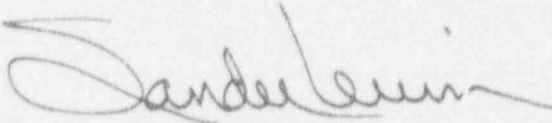
U. S. Nuclear Regulatory Commission
Att: Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Inservice Inspection Program; 10 Year Update
Supplemental Response to Request for Additional Information

By letter dated April 16, 1992, GPU Nuclear submitted the Inservice Inspection Program for the third 10 year interval at the Oyster Creek Nuclear Generating Station. The USNRC subsequently evaluated this submittal and by letters dated March 1, 1993, and July 6, 1993, requested additional information. GPUN responded to these requests by letters dated May 7, 1993, and August 23, 1993. Attachment I to this letter contains supplemental information, as committed in the earlier GPUN letters.

If any additional information or assistance is required, please contact Mr. John Rogers of my staff at 609.971.4893.


for John J. Barton
Vice President and Director
Oyster Creek

JJB/JJR
Attachments
Enclosure

cc: Oyster Creek NRC Project Manager
Administrator, Region I
Senior Resident Inspector

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9404190363 940407
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Q PDR

ACT 1/1

ATTACHMENT I

NRC REQUEST:

Provide additional information relative to Branch Technical Position MEB 3-1. (BTP MEB 3-1 addressed pipe breaks outside of primary containment.)

GPUN RESPONSE:

The Oyster Creek Nuclear Generating Station was part of the USNRC Systematic Evaluation Program (SEP). SEP topic III-5.B related to pipe breaks outside of containment. Branch Technical Position MEB 3-1 was addressed by the Integrated Plant Safety Assessment Report (IPSAR), Section 4.10. The USNRC review divided pipe breaks outside containment into two groups: 1) Section 4.10(1) which discussed the Main Steam and Reactor Water Cleanup systems; and 2) Section 4.10(2) which discussed the Isolation Condenser systems.

In NUREG-1382, "Safety Evaluation Report related to the full term operating license for Oyster Creek Nuclear Generating Station", docketed in January 1991, the NRC stated in part:

"In IPSAR Section 4.10(1), the staff stated that the concerns pertaining to a loss of coolant accident outside the containment had been resolved."

Therefore, no further actions or inspections on the Main Steam or Reactor Water Cleanup Systems were required.

Additionally, NUREG-1382 addressed the Isolation Condenser systems by stating:

"In IPSAR Section 4.10(2), the staff identified concerns associated with emergency condenser isolation. In IPSAR Supplement I, the staff indicated that the licensee would submit information on this matter for review. In a letter dated July 27, 1988, the licensee described plans to replace all four isolation condenser penetrations. Additionally, all isolation condenser piping on the 75-foot elevation and penetration material will be replaced with Nuclear grade 316 stainless steel piping. To provide time for design review, equipment procurement, and logistical optimization of implementation, the licensee has proposed a deferment in the schedule (from Cycle 12 refueling outage to the Cycle 13 refueling outage) for the resolution of this issue. The staff finds this change in schedule acceptable. It will review the details of the licensee's final design and justifying analyses when they are submitted."

The referenced modifications to the Isolation Condenser systems were installed, tested and accepted during the 13R refueling outage. The post installation testing included an ultrasonic examination of all newly installed welds. This information was submitted to the USNRC by letter dated October 28, 1991.

Therefore: 1) the concerns addressed by Branch Technical Position MEB 3-1 were closed in January 1983 for all systems except the Isolation Condenser systems; and 2) the concerns addressed by Branch Technical Position MEB 3-1 in the Isolation Condenser system were eliminated by hardware modifications performed during the Cycle 13R refueling outage.

As the welds addressed by MEB 3-1 are already included in the examinations performed to meet Generic Letter 88-01, and in light of the fact that these examinations have been previously determined by the NRC staff to be acceptable, no additional examinations to address BTP MEB 3-1 would contribute to the overall safe operation of the Oyster Creek Nuclear Generating Station.