CONNECTICUT VANKEE ATOMIC POWER COMPANY



HADDAM NECK PLANT 362 INJUN HOLLOW ROAD • EAST HAMPTON, CT 06424-3099

April 15, 1994 Re: Technical Specification 6.9.1.8 Docket No. 50-213

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 94-03 covering operations for the period March 1, 1994 to March 31, 1994 is hereby forwarded.

Very truly yours,

John P. Stetz Vice President Haddam Neck Station

JPS/va

- cc: (1) Regional Administrator, Region 1 U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406
 - (2) William J. Raymond Sr. Resident Inspector Connecticut Yankee

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Connecticut Yankee Atomic Power Company

a.

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 94-03

For The Month of

March 1994

Plant Operations Summary - March 1994

The following is a Summary of Plant Operations for March 1994.

On March 1st, at 0000 hours, the plant was in Mode 5, Cold Shutdown at 0% load.

On March 25th at 2258 hours, the plant entered Mode 4, Hot Shutdown.

On March 26th at 1655 hours, the plant entered Mode 3, Hot Standby.

On March 28th at 1003 hours, the plant commenced a critical approach and entered Mode 2, Start Up. At 2122 hours, the generator was phased to the grid and the plant entered Mode 1, Power Operation. At 2227 hours, an automatic trip occurred while transferring the reactor coolant pumps to the 1-1A and 1-1B buses. The plant was placed in Mode 3, Hot Standby.

On March 29th at 1037 hours, the problem with reactor coolant pumps was resolved and the plant commenced a critical approach and entered Mode 2, Start Up. At 1818 hours, the generator was phased to the grid and the plant entered Mode 1, Power Operation. At 1852 hours, the plant was at 10% load for a chemistry hold. At 2015 hours, the plant commenced a load increase.

On March 30th at 0444 hours, the plant was at 30% load for a chemistry hold. At 0916 hours, chemistry was within specifications and power ascension resumed. At 1413 hours, the plant was at 70% load and holding for the control rod alignment function. At 1624 hours, the plant commenced power ascension.

On March 31st at 0048 hours, the plant was at 100% rated power.

The plant continued to operate at 100% for the remainder of the month.

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213

2. Reporting Period: 03/94 Outage + On-line Kours: 689.2 + 54.8 = 744.0

3. Utility Contact: W.M. Herwig (203) 267-3198

4. Licensed Thermal Power (MWt): 1825

5. Nameplate Rating (Gross MWe): $667 \times 0.9 = 600.3$

6. Design Electrical Rating (Net MWe): 582

7. Maximum Dependable Capacity (Gross MWe): 586.9 B. Maximum Dependable Capacity (Net MWe): 560.1

9. If changes occur above since last report, reasons are: NONE 10. Power level to which restricted, if any (Net MWe): N/A

11. Reasons for restriction, if any: N/A

		MONTH	YEAR-TO-DATE	CUMULATIVE
12	Report period hours:	744.0	2,160.0	230,088.0
13	. Hours reactor critical:	69.4	1,092.7	183,045.0
14	. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15	. Hours generator on-line:	54.8	1,076.6	176,279.4
16	. Unit reserve shutdown hours:	0.0	0.0	398.0
17	. Gross thermal energy generated (MwtH):	70,558.0	1,923,980.0	304,744,248.0 *
18	Gross electrical energy generated (MWeH):	22,881.0	645,207.0	99,908,126.0 *
19	. Net electrical energy generated (MWeR):	17,403.3	609,889.B	94,911,163.8 *
20	. Unit service factor:	7.4	49.8	76.6
21	. Unit availability factor:	7.4	49.8	76.8
22	. Unit capacity factor using MDC net:	4.2	50.4	74.8
23	. Unit capacity factor using DER net:	4.0	48.5	70.9
24	. Unit forced outage rate:	92.6	47,3	6.0
25	. Forced outage hours:	689.2	967.0	11,202.2

26. Shutdowns scheduled over next 6 months (type,date, duration): NONE

27. If currently shutdown, estimated startup date: N/A

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

AVERAGE DAILY UNIT POWER LEVEL

Docket No: 50-213

Unit: Connecticut Yankee Haddam Neck

Bate: April 15, 1994

Completed By: K. Emmons/M. Bigalbal

Telephone: (203) 267-3654

DAY	AVERAGE POWER LEVEL (MWe-Net)	DAY	AVERAGE POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	Q
3	Q	19	<u>0</u>
4	<u>0</u>	2 0	Q
5	Q	21	<u>0</u>
6	<u>0</u>	2 2	<u>0</u>
7	<u>0</u>	2 3	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	2 5	Q
10	Q	2 6	Q
11	<u>0</u>	2 7	Q
12	Q	28	<u>0</u>
13	<u>0</u>	29	Q
14	Q	30	300
15	Q	31	<u>581</u>
16	<u>0</u>		

Month: March 1994

UNIT SHUTDOWNS AND POWER REDUCTION

Docket No: 50-213 Unit Name: Connecticut Yankee Date: April 15, 1994 Completed By: Kathy Emmons Telephone: (203) 267-3654

Report Month: March 1994

No.	Date	Туре	Duration (Hours)	Reason	Method of Shutting down Reactor	Report		Component Code	Cause and Corrective Action to Prevent Recurrence
94-02	3/01/94	F	669.37	A	1	94-001	BI	N/A	Pin hole leak on service water system (SWS) supply piping (Continued from previous month)
94-03	3/28/94	F	19.85	A/F	3	94-009	JC	BKR	While shifting reactor coolant pump busses, power went above 10 Procedure revision and repair/reinstallment of reactor coolant pump bus breaker should prevent recurrence

TYPE

- F Forced S Scheduled
- REASON
- A Equipment Failure
- B Maintenance or Test C Refueling
 - D Regulatory Restriction
 - E Operator Training
 - F Administrative
 - G Operator Error

 - H Other (Explain)

METHOD

- 1 Manual
- 2 Manual Scram
- 3 Automatic Scram
- 4 Continued
- 5 Reduced Load
- 9 Other

IEEE Standard 805-1984 and/or NUREG-0161 Exhibit F

SYSTEM

COMPONENT

IEEE Standard 803A-1983 and/or NUREG-0161 Exhibit H

Refueling Information Request

1. Name of facility

Haddam Neck

- 2. Scheduled date for next refueling shutdown.
 - January 14, 1995
- Scheduled date for restart following refueling.

March 6, 1995

4.

. . . .

(a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

(b) If answer is yes, what, in general, will these be?

changes to linear heat generation rate uncertainties
necessary changes to the Design Features, Section 5 to support new fuel design
changes to support storage of new and spent fuel with higher enrichments

(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?

n/a

(d) If no such review has taken place, when is it scheduled?

n/a

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

April 1994

 Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Change in fuel vendor from B&W Fuel Co. to Westinghouse Electric Corp., and change in fuel assembly design.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 157 (b) 809

 The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planeed, in number of fuel assemblies.

1168

 The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1998