



Public Service Company of Colorado

5909 East 38th Avenue, Denver, Colorado 80207

September 30, 1982  
Fort St. Vrain  
Unit No. 1  
P-82430  
FSV-63

Mr. Robert A. Clark  
Chief, Operating Reactors Branch 3  
Division of Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20533

DOCKET NO. 50-267

SUBJECT: Fort St. Vrain Inservice Inspection  
and Testing - Additional  
Surveillance Requirements

- REFERENCE: 1) PSC Letter, J.K. Fuller to  
James Miller, P-80064 dated  
March 31, 1980
- 2) PSC Letter, J.K. Fuller to  
Steven A. Varga, P-80034, dated  
March 3, 1980

Gentlemen:

PSC has submitted for your review a preliminary update of the inservice inspection and testing program for all Fort St. Vrain systems and equipment identified as priority category I. Following an independent review under the direction of Los Alamos National Laboratory, it was agreed at the meeting held on July 29, 1982 at Fort St. Vrain that PSC would further investigate the feasibility of additional examinations and tests for:

- the PCRV penetrations secondary boundaries, including flow restrictors, restraints and limit stops; and

8210060273 820930  
PDR ADOCK 05000267  
P PDR

A047

- the steam generator bimetallic welds.

Enclosed for your review are the results of these additional investigations, together with draft Technical Specifications that reflect proposed additional examinations and tests.

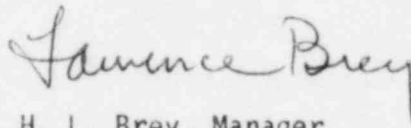
Enclosure (1) contains an evaluation of the additional surveillance requirements for the PCRV penetrations and is a supplement to PSC Letter P-80034 dated March 3, 1980. An attachment to this enclosure provides a draft for a new proposed Technical Specification SR5.2.28 - PCRV Penetrations and Closures Surveillance - which incorporates the examination requirements for the refueling penetration holddown plates, previously submitted as SR5.2.24 in letter P-80034.

Enclosure (2) contains an evaluation of the additional surveillance requirements for the steam generator bimetallic welds and is a supplement to PSC Letter P-80064 dated March 31, 1980. An attachment to this enclosure provides a draft for a new proposed Technical Specification SR5.3.11 - Steam Generator Bimetallic Welds Surveillance.

These additional surveillance requirements will be included in the forthcoming formal submittal for changes to the Technical Specifications to update the Fort St. Vrain inservice inspection and testing program and complete the PSC commitment included in the Safety Evaluation Report of January 20, 1972.

Please direct any questions or comments you may have concerning the submittals or PSC ISI program to Mr. M. H. Holmes, (303) 571-6711.

Very truly yours,



H. L. Brey, Manager  
Nuclear Engineering Division

HLB/JRJ:pa

Attachment