



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1
DOCKET NO. 50-445
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 23
License No. NPF-87

- I. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated May 21, 1993, as supplemented by letter dated February 23, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

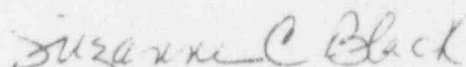
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 23, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 6, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 2
DOCKET NO. 50-446
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. NPF-89

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated May 21, 1993, as supplemented by letter dated February 23, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. TU Electric shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 6, 1994

ATTACHMENT TO LICENSE AMENDMENT NOS. 23 AND 9

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

DOCKET NOS. 50-445 AND 50-446

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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PLANT SYSTEMS

3/4.7.6 FLOOD PROTECTION

LIMITING CONDITION FOR OPERATION

3.7.6 Flood protection shall be provided for all safety-related systems, components, and structures when the water level of the Squaw Creek Reservoir (SCR) exceeds 777.5 feet Mean Sea Level, USGS datum.

APPLICABILITY: At all times.

ACTION: (Units 1 and 2)

With the water level of SCR above elevation 777.5 feet Mean Sea Level, USGS datum, initiate and complete within 2 hours, the flood protection measures verifying that any equipment which is to be opened or is opened for maintenance is isolated from the SCR by isolation valves, or stop gates, or is at an elevation above 790 feet.

SURVEILLANCE REQUIREMENTS

4.7.6 The water level of SCR shall be determined to be within the limits by:

- a. Measurement at least once per 24 hours when the water level is below elevation 776 feet Mean Sea Level, USGS datum,
- b. Measurement at least once per 2 hours when the water level is equal to or above elevation 776 feet Mean Sea Level, USGS datum, and
- c. With the water level of SCR above 777.0 feet Mean Sea Level, USGS datum, verify flood protection measures are in effect by verifying once per 12 hours that flow paths from the SCR which are open for maintenance are isolated from the SCR by isolation valves, or stop gates, or are at an elevation above 790 feet.

PLANT SYSTEMS

3/4.7.7 CONTROL ROOM HVAC SYSTEM

CONTROL ROOM EMERGENCY FILTRATION/PRESSURIZATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.7.1 Two Control Room Emergency Filtration/Pressurization System trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, 4, 5, 6, and during movement of irradiated fuel assemblies.

ACTION:

MODES 1, 2, 3, and 4:

With one Control Room Emergency Filtration/Pressurization System train inoperable, restore the inoperable train to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5, 6 and during movement of irradiated fuel assemblies:

- a. With one Control Room Emergency Filtration/Pressurization System train inoperable, restore the inoperable train to OPERABLE status within 7 days or immediately place the OPERABLE Control Room Emergency Filtration/Pressurization System train in the emergency recirculation mode or immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.
- b. With two Control Room Emergency Filtration/Pressurization System trains inoperable immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.

SURVEILLANCE REQUIREMENTS

4.7.7.1 Each Control Room Emergency Filtration/Pressurization System train shall be demonstrated OPERABLE:

- a. At least once per 31 days by operating each Control Room Emergency Filtration/Pressurization System train for ≥ 10 continuous hours with the heaters operating.
- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire, or chemical release in any ventilation zone communicating with the system by:

PLANT SYSTEMS

3/4.7.7 CONTROL ROOM HVAC SYSTEM

CONTROL ROOM EMERGENCY FILTRATION/PRESSURIZATION SYSTEM (Continued)

SURVEILLANCE REQUIREMENTS (Continued)

- (1) Verifying that the filtration unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% by using the test procedure guidance in Regulatory Position C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978*, and the emergency filtration unit flow rate is 8000 cfm \pm 10%, and the emergency pressurization unit flow rate is 800 cfm \pm 10%;
 - (2) Verifying within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a Regulatory Guide 1.52, Revision 2, March 1978*, for a methyl iodide penetration of less than 0.2%; and
 - (3) Verifying an emergency filtration unit flow rate of 8000 cfm \pm 10% and an emergency pressurization unit flow rate of 800 cfm \pm 10% during system operation when tested in accordance with ANSI N510-1980.
- c. After every 720 hours of charcoal adsorber operation, by verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*, for a methyl iodide penetration of less than 0.2%;
- d. At least once per 18 months by:
- (1) Verifying that the total pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 8.0 inches water gauge while operating the emergency filtration unit at a flow rate of 8000 cfm \pm 10%, and is less than 9.5 inches water gauge while operating the emergency pressurization unit at a flow rate of 800 cfm \pm 10%; and
 - (2) Verifying that the heaters in the emergency pressurization units dissipate 10 ± 1 kW when tested in accordance with ANSI N510-1980;

*ANSI N510-1980 and ANSI N509-1980 shall be used in place of ANSI N510-1975 and ANSI N509-1976, respectively.

PLANT SYSTEMS

3/4.7.7 CONTROL ROOM HVAC SYSTEM

CONTROL ROOM EMERGENCY FILTRATION/PRESSURIZATION SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- e. After each complete or partial replacement of a HEPA filter bank in the emergency filtration unit(s), by verifying that the unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% in accordance with ANSI N510-1980 for a DOP test aerosol while operating the unit at a flow rate of 8000 cfm \pm 10%;
- f. After each complete or partial replacement of a charcoal adsorber bank in the emergency filtration unit(s), by verifying that the unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% in accordance with ANSI N510-1980 for a halogenated hydrocarbon refrigerant test gas while operating the unit at a flow rate of 8000 cfm \pm 10%;
- g. After each complete or partial replacement of a HEPA filter bank in the emergency pressurization unit(s), by verifying that the unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% in accordance with ANSI N510-1980 for a DOP test aerosol while operating the unit at a flow rate of 800 cfm \pm 10%.
- h. After each complete or partial replacement of a charcoal adsorber bank in the emergency pressurization unit(s), by verifying that the unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% in accordance with ANSI N510-1980 for a halogenated hydrocarbon refrigerant test gas while operating the unit at a flow rate of 800 cfm \pm 10%;
- i. At least once per 18 months by verifying that each Control Room Emergency Filtration/Pressurization System train actuates on an actual or simulated Safety Injection, Loss-of-Offsite Power, or Intake Vent-High Radiation Signal; and
- j. At least once per 18 months by verifying that each Control Room Emergency Filtration/Pressurization System train can maintain a positive pressure of \geq 0.125 inches water gauge, relative to the adjacent areas during the pressurization mode of operation at a makeup flow rate of \leq 800 cfm.

PLANT SYSTEMS

CONTROL ROOM HVAC SYSTEM

CONTROL ROOM AIR CONDITIONING SYSTEM (CRACS)

LIMITING CONDITION FOR OPERATION

3.7.7.2 Two CRACS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, 4, 5, 6, and during movement of irradiated fuel assemblies.

ACTION:

MODES 1, 2, 3 and 4:

- a. With one CRACS train inoperable, restore the inoperable train to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With two CRACS trains inoperable and at least 100% of the required heat removal capability equivalent to a single OPERABLE CRACS train available, restore the inoperable trains to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5, 6 and during movement of irradiated fuel assemblies:

- a. With one CRACS train inoperable, restore the inoperable train to OPERABLE status within 30 days or immediately place the OPERABLE CRACS train in operation or immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.
- b. With two CRACS trains inoperable and at least 100% of the required heat removal capability equivalent to a single OPERABLE CRACS train available, restore the inoperable trains to OPERABLE status within 30 days or immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.
- c. With two CRACS trains inoperable and with b. above not applicable, immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.

SURVEILLANCE REQUIREMENTS

4.7.7.2 At least once per 18 months verify each CRACS train has the capability to remove the assumed head load.

PLANT SYSTEMS

3/4.7.8 PRIMARY PLANT VENTILATION SYSTEM - ESF FILTRATION UNITS

LIMITING CONDITION FOR OPERATION

3.7.8 Two independent ESF Filtration Trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

- a. With one ESF Filtration Train inoperable, restore the inoperable ESF Filtration Train to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With the inability to reach and maintain a negative pressure in the negative pressure envelope of the Auxiliary, Safeguards, and Fuel Buildings greater than or equal to 0.05 inch water gauge, restore the PRIMARY PLANT VENTILATION SYSTEM to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With the inability to reach and maintain a negative pressure in the negative pressure envelope of the Auxiliary, Safeguards, and Fuel Buildings greater than or equal to 0.01 inch water gauge, restore the PRIMARY PLANT VENTILATION SYSTEM'S ability to maintain a negative pressure of greater than or equal to 0.01 inch water gauge within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.8 Each ESF Filtration Train shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that each ESF Filtration Train operates for at least 10 continuous hours with the heaters operating;
- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire, or chemical release in any ventilation zone communicating with the system by:
 - 1) Verifying that each ESF Filtration Unit satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 1.0% by using the test procedure guidance in Regulatory Positions C.5.a, C.5.c, and C.5.d of Regulatory Guide 1.52,

PLANT SYSTEMS

BASES

3/4.7.5 ULTIMATE HEAT SINK

The limitations on the ultimate heat sink level and temperature ensure that sufficient cooling capacity is available to either: (1) provide normal cooldown of the facility or (2) mitigate the effects of accident conditions within acceptable limits.

The limitations on minimum water level is based on providing a 30-day cooling water supply to safety-related equipment without exceeding its design basis temperature and is consistent with the recommendations of Regulatory Guide 1.27, "Ultimate Heat Sink for Nuclear Plants," Rev. 2 (January 1976). The limitation on maximum temperature is based on the maximum allowable component temperatures in the Service Water and Component Cooling Water Systems, and the requirements for cooldown. The limitation on average sediment depth is based on the possible excessive sediment buildup in the service water intake channel.

3/4.7.6 FLOOD PROTECTION

The limitation of flood protection ensures that facility protective actions will be taken in the event of flood conditions. The only credible flood condition that endangers safety related equipment is from water entry into the turbine building via the circulating water system from Squaw Creek Reservoir and then only if the level is above 778 feet Mean Sea Level. This corresponds to the elevation at which water could enter the electrical and control building endangering the safety chilled water system. The surveillance requirements are designed to implement level monitoring of Squaw Creek Reservoir should it reach an abnormally high level above 776 feet. The Limiting Condition for Operation is designed to implement flood protection, by ensuring no open flow path via the Circulating Water System exists, prior to reaching the postulated flood level.

3/4.7.7 CONTROL ROOM HVAC SYSTEM

The control room emergency filtration/pressurization system consists of two independent, redundant trains that recirculate and filter the control room air, and two independent, redundant trains that pressurize the control room.

The OPERABILITY of the Control Room HVAC System ensures that: (1) the control room ambient air temperature does not exceed the allowable temperature per 3/4.7.10 for continuous-duty rating for the equipment and instrumentation cooled by this system, and (2) the control room will remain habitable including temperature for operations personnel during and following all credible accident conditions.

A Control Room Air Conditioning System (CRACS) consists of two independent and redundant trains that provide cooling and heating of recirculated control room air. Each Control Room Air Conditioning (CRAC) train includes two heating and cooling units, instrumentation, and controls to provide for control room temperature control. Each cooling unit provides 50% of the heat removal capability for its respective train.

A CRAC train is inoperable if it is not capable of removing the required heat load for plant conditions. The required heat load includes normal and post-accident conditions. The actual heat load and the heat removal

PLANT SYSTEMS

BASES

3/4.7.7 CONTROL ROOM HVAC SYSTEM (Continued)

capability needed to adequately cool the control room is dependent upon factors such as outdoor air temperature and safe shutdown impoundment water temperature. OPERABILITY determinations are based upon design basis conditions unless a specific evaluation has been performed which identifies the required heat load for actual conditions. Individual components are inoperable if they are not capable of performing their safety functions.

Due to the redundancy of trains, the diversity of components, and the annual variations in outdoor conditions, the inoperability of one component in a train or two components in different trains does not necessarily result in a loss of safety function for the CRACS. The intent of this condition is to maintain a combination of equipment such that 100% of the required heat removal capability of a single OPERABLE CRAC train remains available.

With one or more CRAC trains inoperable and at least 100% of the required heat removal capability equivalent to a single OPERABLE CRAC train available, the inoperable trains must be returned to OPERABLE status within 30 days. This Allowed Outage Time is based on the low probability of complete loss of cooling due to the redundancy of the support systems (electrical and cooling water), the capability of the OPERABLE train/components to provide the required cooling, the potential that plant staff actions can restore or mitigate the effects of component failures, and the time available to respond as loss of cooling does not have an immediate, irreversible impact.

While in MODES 5, 6 or during movement of irradiated fuel assemblies, if both trains cannot be restored to OPERABLE status within 30 days, an OPERABLE train (A or B) must be placed in operation immediately; otherwise, immediately suspend CORE ALTERATIONS and movement of irradiated fuel assemblies.

Operation of the system with the heaters operating to maintain low humidity using automatic control for at least 10 continuous hours in a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rems or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of 10 CFR 50 Appendix A. ANSI N510-1980 and ANSI N509-1980 will be used as a procedural guide for surveillance testing.

PLANT SYSTEMS

BASES

3/4.7.8 PRIMARY PLANT VENTILATION SYSTEM - ESF FILTRATION UNITS

The OPERABILITY of the ESF Filtration Units ensures that radioactive materials leaking from the ECCS equipment within the safeguards and auxiliary buildings following a LOCA are filtered prior to reaching the environment. These filtration units also ensure that radioactive materials leakage from within the fuel building are filtered prior to reaching the environment. Operation of the ESF filtration units with the heaters operating to maintain low humidity using automatic control for at least 10 continuous hours in a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters. The operation of the ESF filtration units and the resultant effect on offsite dosage calculations was assumed in the safety analyses. ANSI N510-1980 and ANSI N509-1980 will be used as a procedural guide for surveillance testing.

The negative pressure envelope of the Auxiliary, Safeguards and Fuel Buildings is the portions of these buildings which is exhausted post accident to ensure that potential ECCS leakage is filtered.

3/4.7.9 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the Reactor Coolant System and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads.

Snubbers are classified and grouped by design and manufacturer but not by size. For example, mechanical snubbers utilizing the same design features of the 2-kip, 10-kip and 100-kip capacity manufactured by Company "A" are of the same type. The same design mechanical snubbers manufactured by Company "B" for the purposes of this Technical Specification would be of a different type, as would hydraulic snubbers from either manufacturer.

A list of individual snubbers with detailed information of snubber location and size and of system affected shall be available at the plant in accordance with 10 CFR 50.71(c). The accessibility of each snubber shall be determined and approved by the Station Operation Review Committee (SORC). The determination shall be based upon the existing radiation levels and the expected time to perform a visual inspection in each snubber location as well as other factors associated with accessibility during plant operations (e.g., temperature, atmosphere, location, etc.), and the recommendations of Regulatory Guides 8.8 and 8.10. The addition or deletion of any hydraulic or mechanical snubber shall be made in accordance with 10 CFR 50.59.

Surveillance to demonstrate OPERABILITY is by performance of the requirements of an approved inservice inspection program.