

## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

April 12, 1994

Docket No. 52-002

APPLICANT: ABB-Combustion Engineering, Inc. (ABB-CE)

PROJECT: CE System 80+

SUBJECT: PUBLIC MEETING OF JANUARY 26, 1994, REGARDING IN-CONTAINMENT

REFUELING WATER STORAGE TANK (IRWST) PH CONTROL AND THE USE OF THE NEW SOURCE TERM FOR THE ABB-CE SYSTEM 80+ STANDARD PLANT DESIGN

On January 26, 1994, a public meeting was held at the U.S. Nuclear Regulatory Commission (NRC) headquarters offices in Rockville, Maryland, between representatives of the NRC, ABB-CE, and ABB-CE's contractors Duke Engineering and Services (DE&S), and Stone and Webster Engineering Corporation (SWEC). Enclosure 1 provides a list of attendees.

The purpose of the meeting was to discuss the mechanisms for post-accident control of pH in the IRWST and ABB-CE's use of the new source term in the System 80+ design. Presentations were made by ABB-CE, DE&S, and SWFC. Enclosure 2 provides a copy of the material presented at the meeting.

Regarding the IRWST pH control issue, ABB-CE indicated that acid generation from post-accident radiolysis of cable insulation was considered in the context of the System 80 design. The materials of construction for the System 80+ insulation have not been specified, however, the quantity of chloride-producing insulation material used in the System 80 design was agreed by all to bound the System 80+ design. ABB-CE committed to add the amount of chloride-producing cable insulation used in the System 80 design to the list of organic material in Table 6.1-4 of CESSAR-DC. They also committed to add a COL action item that would require the COL applicant to verify that the amount of chloride-producing cable insulation used is less than the limiting amount.

ABB-CE and their contractors stated, in their presentations, that a pH of 7.0 is reached in the IRWST within two and a half hours after an accident. The staff raised questions about the completeness of the analysis performed. After some discussion, it was agreed that ABB-CE should increase the amount of trisodium phosphate dodecahydrate stored inside containment to an amount sufficient to maintain a concentration of 2,000 ppm. ABB-CE committed to revise CESSAR-DC to reflect this. The staff committed to discuss this issue with experts at Oak Ridge National Laboratory and to provide feedback to ABB-CE as soon as possible.

NRC FILE CENTER COPY

Aerosol deposition in containment and containment spray removal were also discussed during the meeting. The staff had previously raised concerns about the appropriateness of the aerosol deposition values and the containment spray removal lambda values used by ABB-CE. In their presentations, ABB-CE and SWEC provided additional justification for the use of these values. The staff committed to reviewing the values with experts at Sandia National Laboratory and to provide feedback to ABB-CE as soon as possible.

### Original Stoned By:

Stewart L. Magruder, Project Manager Standardization Project Directorate Associate Directorate for Advanced Reactors and License Renewal Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/enclosures: See next page

DISTRIBUTION w/enclosures:

Docket File PDST R/F

PShea

ADrozd, 8H7

PDR

HWalker, 8D1

DCrutchfield JLee, 10D4 SMagruder

KEccleston, 10D4

Distribution w/o enclosures:

WRussell/FMiraglia TWambach

ACRS (11)

WTravers RArchitzel

WDean, EDO

RBorchardt JKudrick, 8H7 JMoore, 15B18

MFranovich TEssig, 10D4

OFC: LA: PDST: ADAR

NAME: PShea 335 DATC: 04/11/94

PM: PDST: ADAR SMagruder: t&LM

04/11/94

SC: PDST: ADAR RArchitzel | 04/12/94

OFFICIAL RECORD COPY:

DOCUMENT NAME: MSUM0126.94

cc: Mr. C. B. Brinkman, Acting Director Nuclear Systems Licensing ABB-Combustion Engineering, Inc. 1000 Prospect Hill Road Windsor, Connecticut 06095-0500

> Mr. C. B. Brinkman, Manager Washington Nuclear Operations ABB-Combustion Engineering, Inc. 12300 Twinbrook Parkway, Suite 330 Rockville, Maryland 20852

Mr. Stan Ritterbusch Nuclear Systems Licensing ABB-Combustion Engineering, Inc. 1000 Prospect Hill Road Post Office Box 500 Windsor, Connecticut 06095-0500

Mr. Sterling Franks U.S. Department of Energy NE-42 Washington, D.C. 20585

Mr. Steve Goldberg Budget Examiner 725 17th Street, N.W. Washington, D.C. 20503

Mr. Raymond Ng 1776 Eye Street, N.W. Suite 300 Washington, D.C. 20006

Joseph R. Egan, Esquire Shaw, Pittman, Potts & Trowbridge 2300 N Street, N.W. Washington, D.C. 20037-1128

Mr. Regis A. Matzie, Vice President Nuclear Systems Development ABB-Combustion Engineering, Inc. 1000 Prospect Hill Road Post Office Box 500 Windsor, Connecticut 06095-0500

Mr. Victor G. Snell, Director Safety and Licensing AECL Technologies 9210 Corporate Boulevard Suite 410 Rockville, Maryland 20850

# ABB-CE SYSTEM 80+ IRWST pH CONTROL/SOURCE TERM MEETING ATTENDEES January 26, 1994

#### Name Organization H. Walker NRC M. Jacob ABB-CE F. Carpentino ABB-CE T. Wambach NRC A. Drozd NRC J. Kudrick NRC L. Soffer NRC T. Essig NRC T. Rudek ABB-CE C. Brinkman ABB-CE K. Eccleston NRC J. Schaperow NRC J. Mazetis J. Hayes NRC NRC T. Crom DE&S S. Ritterbusch ABB-CE J. Metcalf SWEC S. Ferguson SWEC W. Peng SWEC

NRC

NRC

NRC

R. Architzel

S. Magruder

K. Parczewski

#### LONG TERM POST ACCIDENT PH CONTORL

LONG TERM pH MAINTAINED AT A MINIMUM OF 7.0 AND LESS THAN 8.5 BASED ON A REFERENCE TEMPERATURE OF 25 °C.

MAXIMUM BORIC ACID CONCENTRATION OF IRWST 4400 PPM

THE FOLLOWING WATER SOURCES ARE CONSIDERED (TOTAL = 708316 GALLONS)

IRWST (NORMAL OPERATING VOLUME)
REACTOR COOLANT SYSTEM, INCLUDING PRESSURIZER
SAFETY INJECTION TANKS
SAFETY INJECTION SYSTEM PIPING
CONTAINMENT SPRAY SYSTEM PIPING

ph control accomplished with 796 cubic feet (45,372 pounds) of trisodium phosphate Dodecahydrate stored in baskets located in the holdup volume

THIS PROVIDES A PHOSPHATE CONCENTRATION EQUAL TO 1719 PPM AS PG4

#### LONG TERM POST ACCIDENT PH CONTROL

PH CONTROL OF IRWST IS INITIATED WHEN WATER SPILLS BACK TO THE IRWST FROM THE HOLDUP VOLUME

59,100 GALLONS IN HOLDUP VOLUME BEFORE SPILL OVER

SPILL OVER OCCURS AT 11.8 MINUTES BASED ON 5000 GPM FROM A SINGLE CONTAINMENT SPRAY PUMP

PH WILL BE AT A MINIMUM OF 7.0 AFTER ALL OF THE ABOVE WATER VOLUME IS RECIRCULATED THROUGH THE HOLDUP VOLUME

2.5 HOURS CONSIDERING FLOW FROM A SINGLE CONTAINMENT SPRAY PUMP OF 5000 GPM

#### LONG TERM POST ACCIDENT PH CONTORL

### TYPICAL COMPARISON

LONGTERM PH CONTROL ACCOMPLISHED WITH TRISODIUM PHOSPHATE DODECAHYDRATE STORED IN BASKETS LOCATED IN THE CONTAINMENT SUMP

NINE BASKETS 2 FT X 2 FT X 2 FT EIGHT BASKET 4 FT X 4 FT X 4 FT

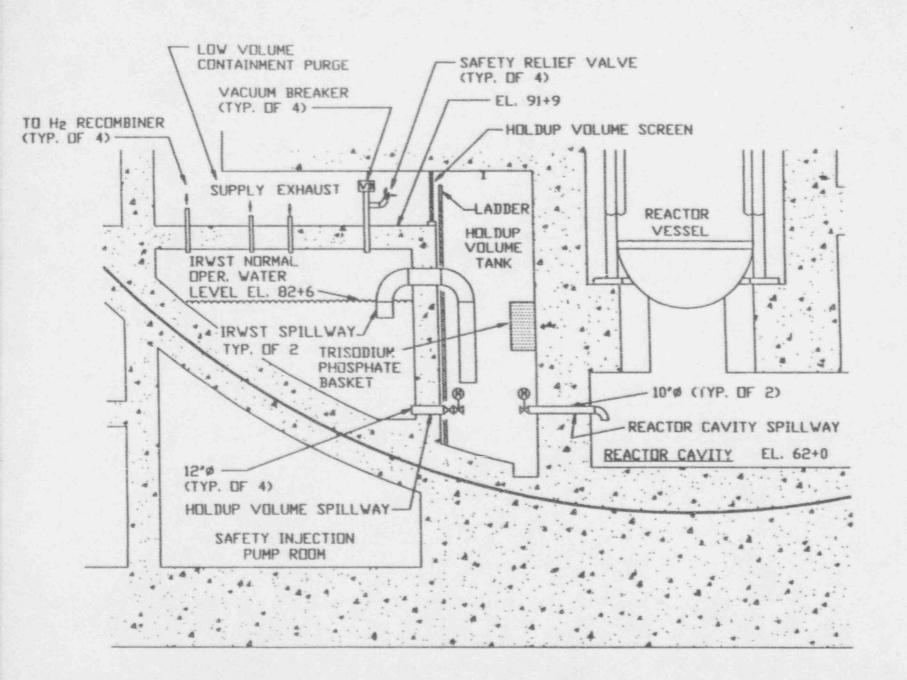
PH MANTAINED BETWEEN 7.0 AND 8.5

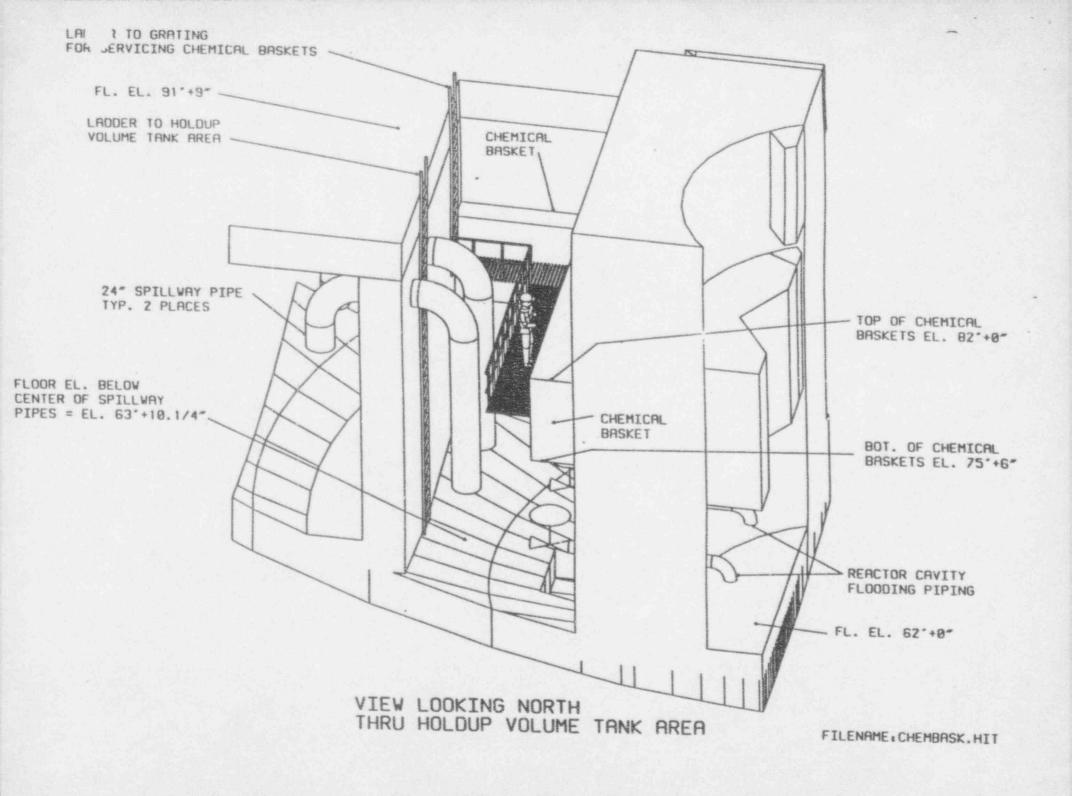
464 CUBIC FEET OF TRISODIUM PHOSPHATE DODECAHYDRATE (26,000 POUNDS)

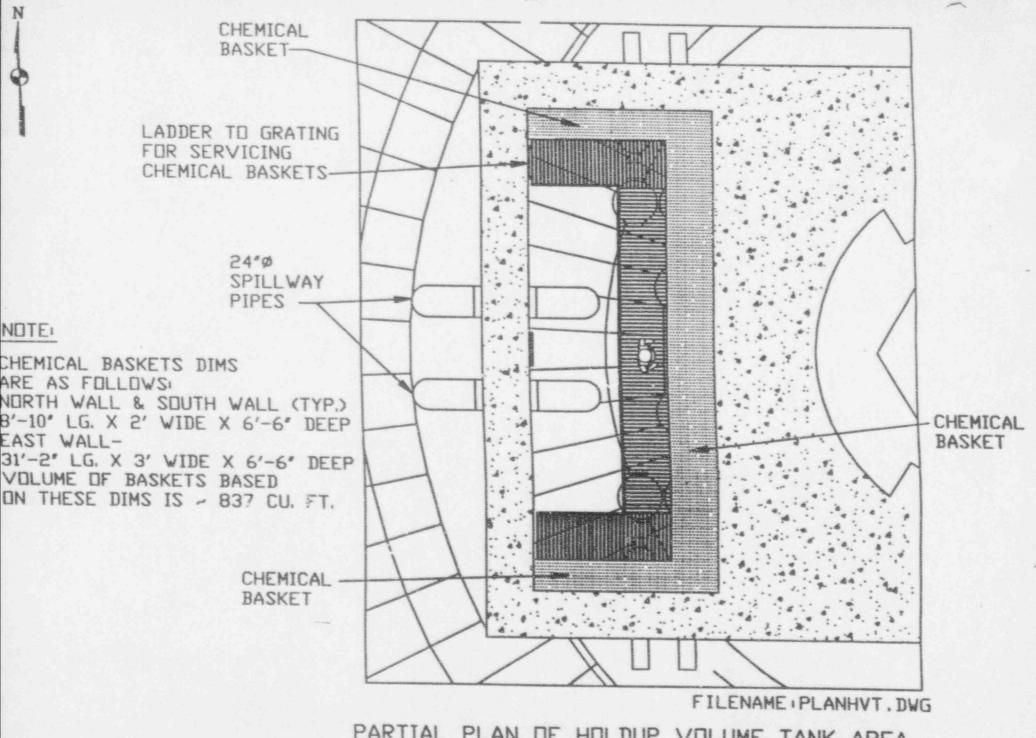
MAXIMUM BORON CONCENTRATION OF EXTERNAL REFUELING WATER STORAGE TANK - 4400 PPM

PHOSPHATE CONCENTRATION EQUAL TO 2000 PPM THIS ACHIEVES A pH OF 7.7 AT A REFERENCE TEMPERATURE OF 25 °C

A MINIMUM ph OF 7.0 IS ASSUMED WITHIN 4 HOURS THROUGH DISSOLUTION WITHIN THE CONTAINMENT SUMP BEFORE RECIRCULATION IS INITIATED

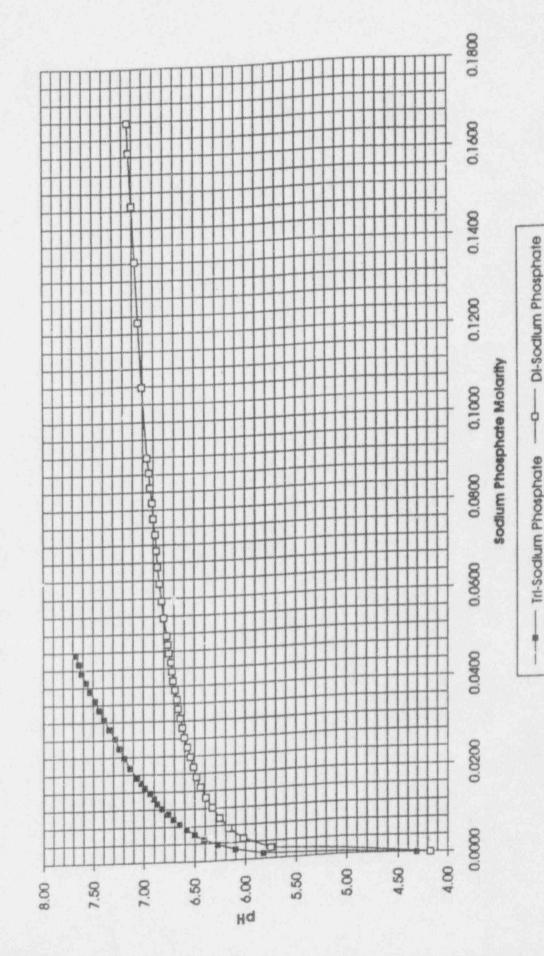






PARTIAL PLAN OF HOLDUP VOLUME TANK AREA

pH of 4400 ppm 8 Boric Acid with Sodium Phosphate Added



## PARTICLE SIZE DEFINITION INPUT TO SWNAUA

### DISTRIBUTION USED FOR EPRI REPORT:

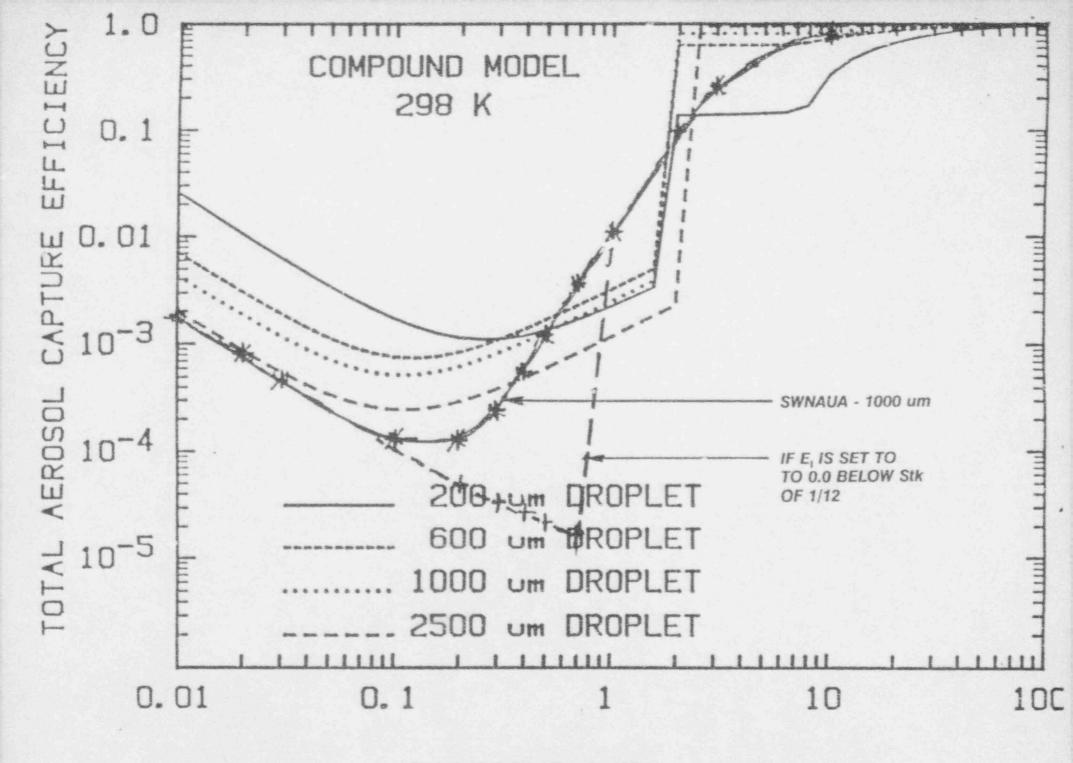
 $r_g = 0.21~\mu m,~\sigma = 1.7$  (USED BY SWEC FOR ANS LARGE BREAK LOCA STUDIES - 1983)

PERCENT LEAKED @  $\sim$  100 MIN AFTER START OF FUEL RELEASE (NO HYGRO) = 0.00065%

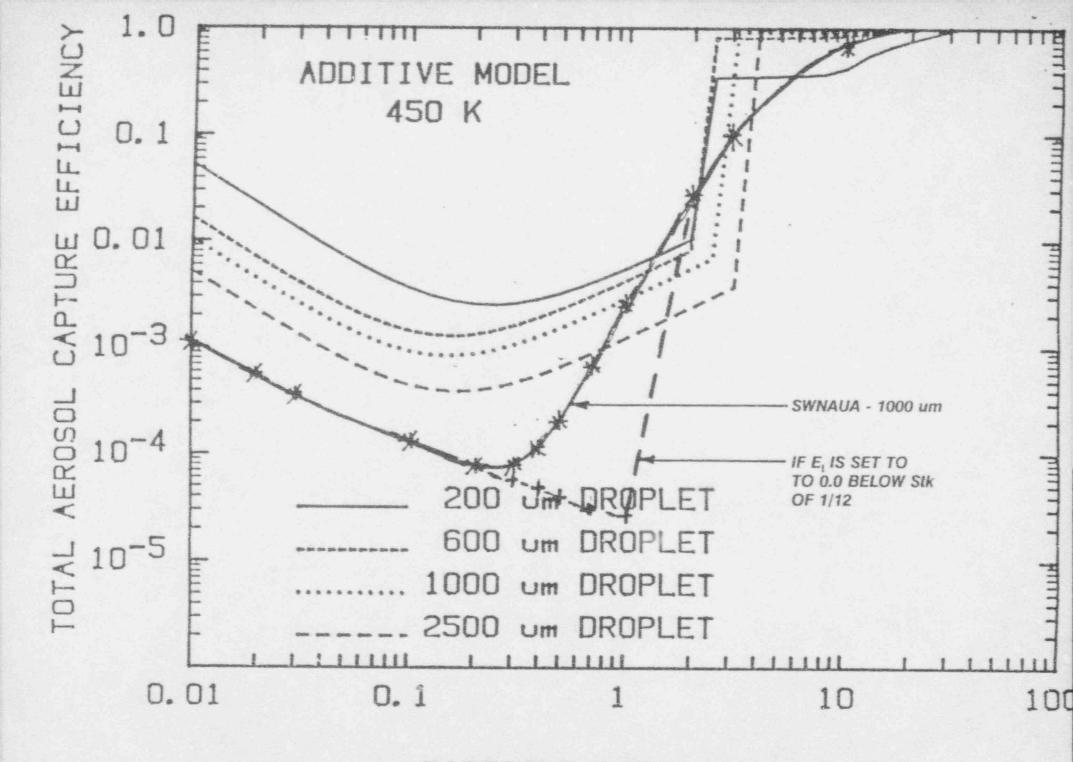
DISTRIBUTION(S) USED FOR SYSTEM 80+ DBA LOCA CALC (BASED ON RAFT ANALYSIS OF STEP-1 EXPERIMENT)

 $r_g = 0.08 \ \mu m, \ \sigma = 1.6 \ (GAP \ RELEASE)$  $r_g = 0.40 \ \mu m, \ \sigma = 1.46 \ (FUEL \ RELEASE)$ 

PERCENT LEAKED @ ~ 100 MIN AFTER START OF FUEL RELEASE = 0.0018%

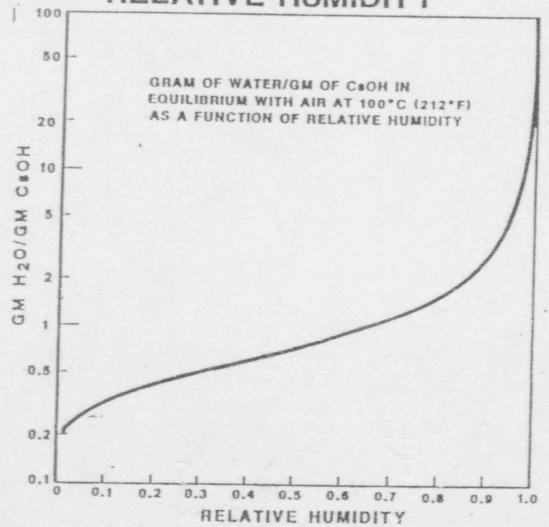


PARTICLE SIZE (um)

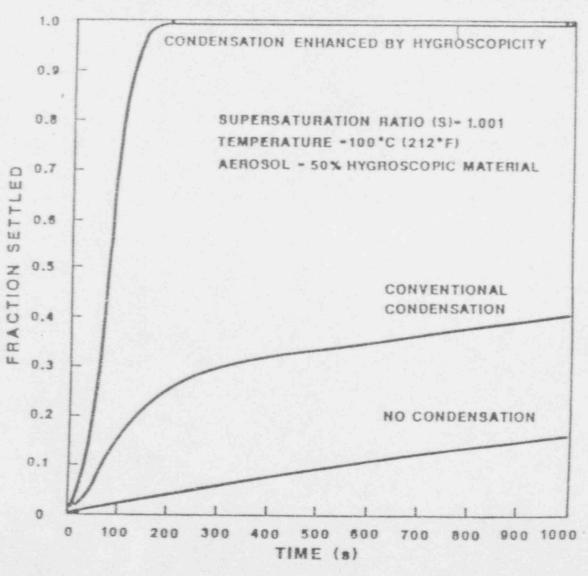


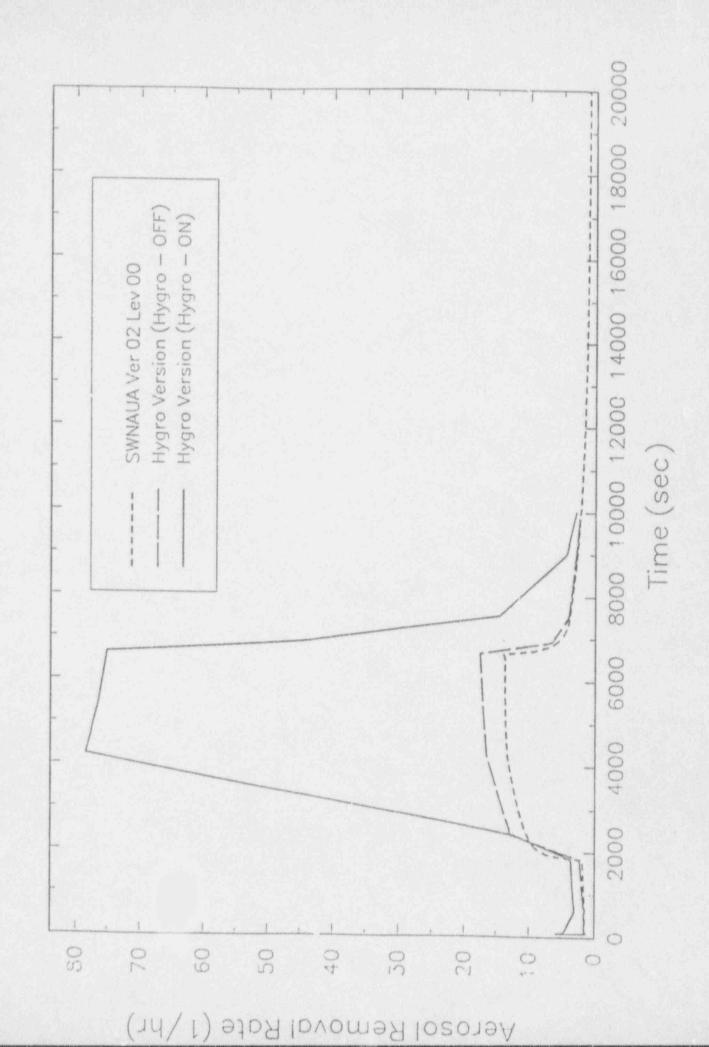
PARTICLE SIZE (um)

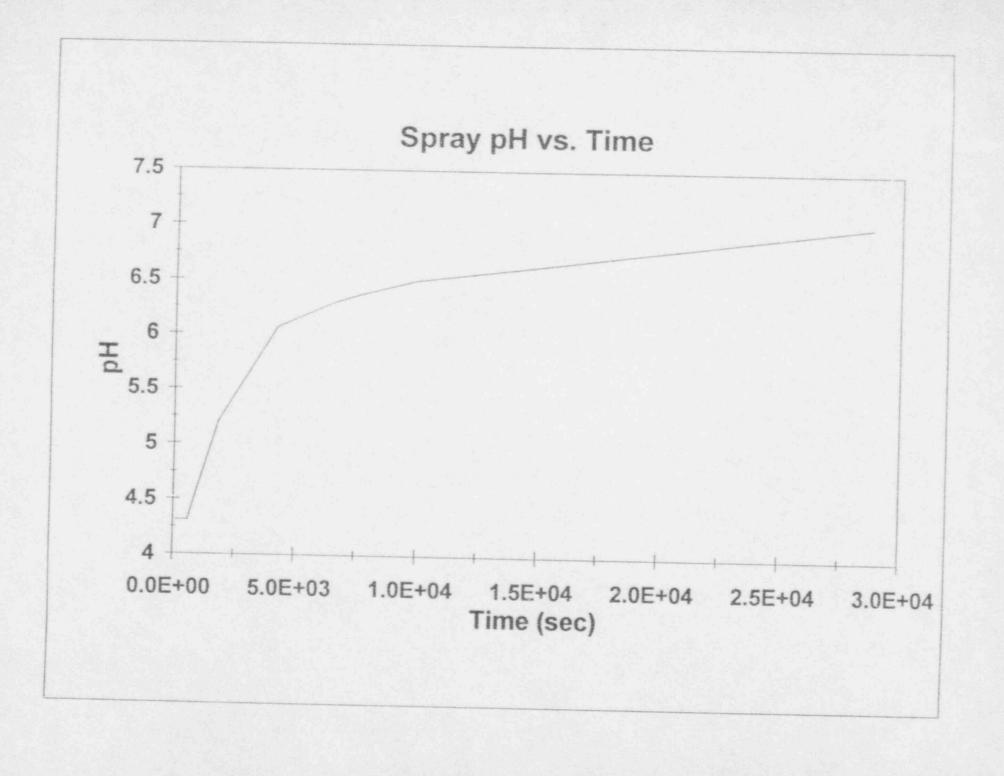
GRAM OF H2O/GRAM OF CSOH VS RELATIVE HUMIDITY



## FRACTION SETTLED VS TIME







## Basis of Analysis

- TSP added at a constant rate over 7.5 hours
- pH vs. TSP data based on Duke Engineering experiments
- Partitions Coefficients Calculated using NUREG/CR-5950
  - Section 3.2 (Appendix C)
  - Section 3.3.1
- Instantaneous Equilibrium w. radiolysis
- No credit for iodate reaction



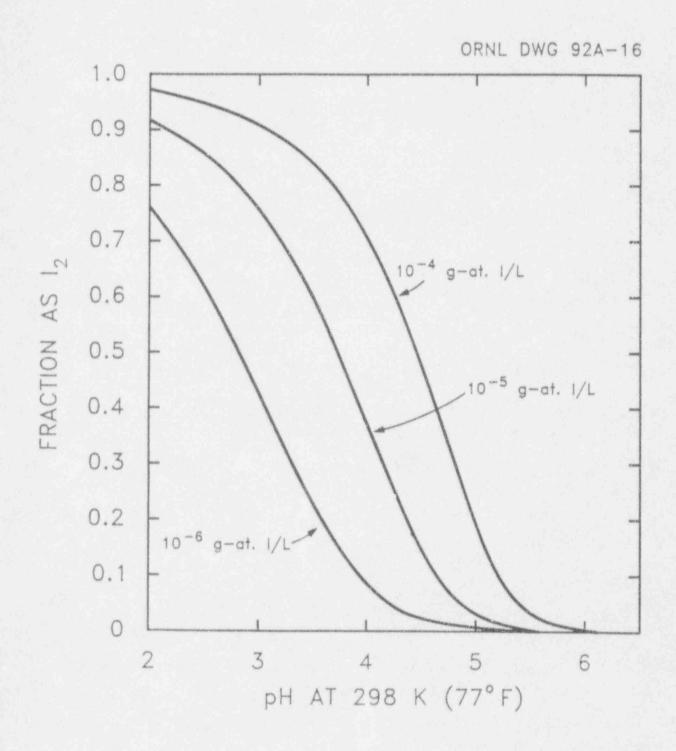
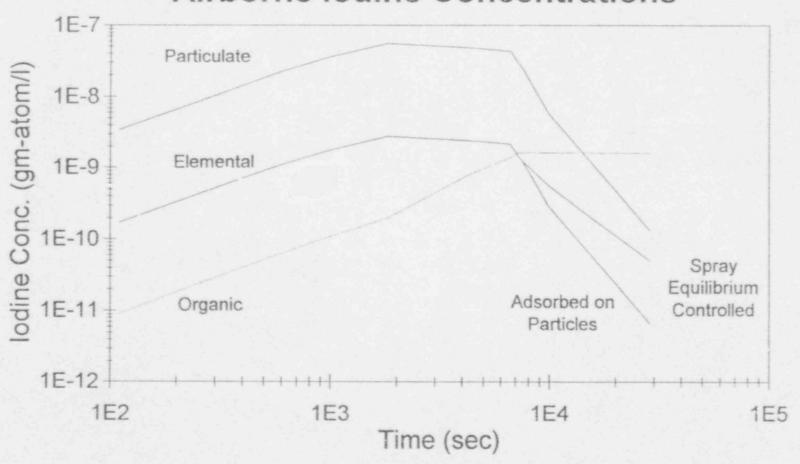
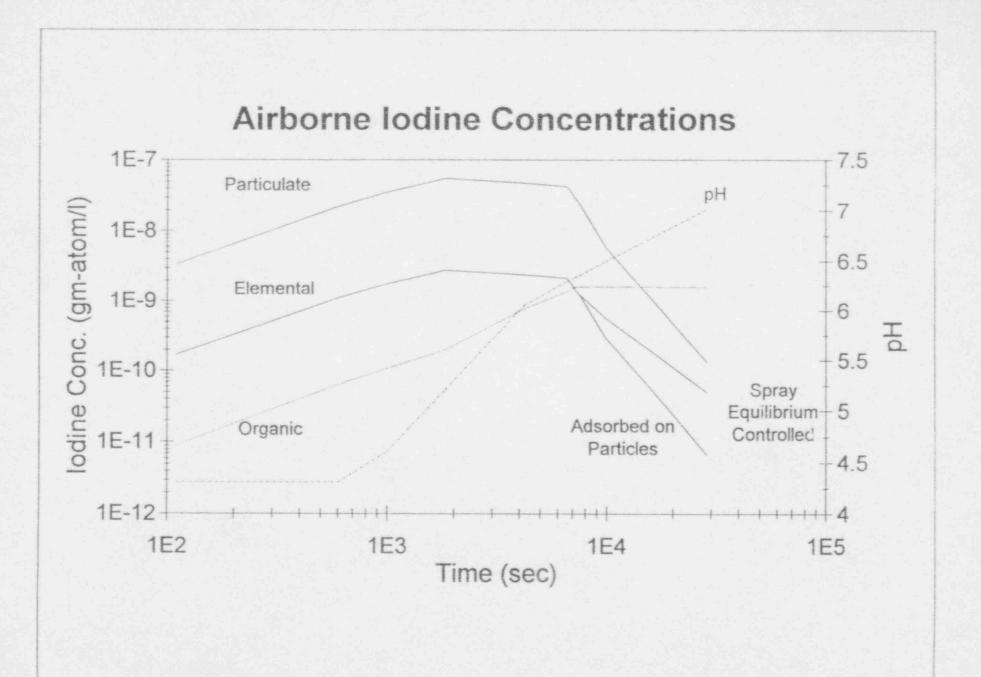


Figure 3.1. Model calculations of fraction as I2 vs pH









DBA LOCA SPRAY LAMBDA

Figure 6.5-5

Amendment R July 30, 1993

