



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 189
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 21, 1993, as supplemented on March 11, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 189, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 7, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 189

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove

77a

Insert

77a

TABLE 3.2.F (Cont'd) - SURVEILLANCE INSTRUMENTATION

Item	Minimum No. of Operable Instrument Channels	Parameter	Instrument	Type Indication and Range	Action*
11	2	Suppression Chamber Water Level (wide range)	LR-8(9)123A, B	Recorder 1-21 ft.	(10)(11)
12	1	Control Rod Position	N/A	28 Volt Indicating Lights)	(1)(2)(3)(4)
13	1	Neutron Monitoring	N/A	SRM, IRM, LPRM) 0-100%)	
14	1	Safety-Relief Valve Position Indication	POAM-2(3)-2-71A-L TE-2(3)-2-113A-L	Acoustic or Thermocouple	(5)
15	2	Drywell High Range Radiation Monitors	RR-8(9)103A, B	Recorder 1-1E(+8) R/hr	(7)
16	1	Main Stack High Range Radiation Monitor	RR-0-17-051	Recorder 10 ⁵ to 10 ¹¹ CPS (Log Scale)	(7)
17	1	Reactor Building Roof Vent High Range Radiation Monitor	RR-2979 (Unit 2) RR-3979 (Unit 3)	Recorder 10 ⁷ to 10 ¹³ CPM (Log Scale)	(7)
18	2	Drywell Hydrogen Concentration Analyzer and Monitor	2(3)BS215, 2(3)CS215 H2R-4(5)965B, C	Analyzer and Recorder 0-20% Volume	(1)(2)(3)

* Notes for Table 3.2.F appear on pages 78 and 78a.



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PHILADELPHIA ELECTRIC COMPANY

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DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated December 21, 1993, as supplemented on March 11, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph ? C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 194, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 7, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 194

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

Insert

77a
225

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225

TABLE 3.2.F (Cont'd) - SURVEILLANCE INSTRUMENTATION

Item	Minimum No. of Operable Instrument Channels	Parameter	Instrument	Type Indication and Range	Action*
11	2	Suppression Chamber Water Level (wide range)	LR-8(9)123A, B	Recorder 1-21 ft.	(10)(11)
12	1	Control Rod Position	N/A	28 Volt Indicating Lights	(1)(2)(3)(4)
13	1	Neutron Monitoring	N/A	SRM, IRM, LPRM 0-100%	
14	1	Safety-Relief Valve Position Indication	POAM-2(3)-2-71A-L TE-2(3)-2-113A-L	Acoustic or Thermocouple	(5)
15	2	Drywell High Range Radiation Monitors	RR-8(9)103A, B	Recorder 1-1E(+8) R/hr	(7)
16	1	Main Stack High Range Radiation Monitor	RR-0-17-051	Recorder 10 ⁵ to 10 ¹¹ CPS (Log Scale)	(7)
17	1	Reactor Building Roof Vent High Range Radiation Monitor	RR-2979 (Unit 2) RR-3979 (Unit 3)	Recorder 10 ⁷ to 10 ¹³ CPM (Log Scale)	(7)
18	2	Drywell Hydrogen Concentration Analyzer and Monitor	3AC872, 3BC872 XR-90411A, XR-90411B	Analyzer and Recorder 0-30% volume	(1)(2)(3)

* Notes for Table 3.2.F appear on pages 78 and 78a.

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LIMITING CONDITIONS FOR OPERATION SURVEILLANCE REQUIREMENTS3.10 CORE ALTERATIONSApplicability

Applies to the fuel handling and core reactivity limitations.

Objective

To ensure that core reactivity is within the capability of the control rods and to prevent criticality during refueling.

SpecificationA. Refueling Interlocks

1. The reactor mode switch shall be locked in the "Refuel" position during core alterations and the refueling interlocks shall be operable except as specified in 3.10.A.5 and 3.10.A.6 below.
2. Fuel shall not be loaded into the reactor core unless all control rods are fully inserted.

4.10 CORE ALTERATIONSApplicability

Applies to the periodic testing of those interlocks and instrumentation used during refueling and core alterations.

Objective

To verify the operability of instrumentation and interlocks used in refueling and core alterations.

SpecificationA. Refueling Interlocks

1. Prior to any core alterations within or over the reactor core, the reactor switch "Refuel" position interlocks shall be functionally tested. They shall be tested at weekly intervals thereafter until no longer required. They shall also be tested following any repair work associated with the interlocks.
2. Prior to performing control rod or control rod drive maintenance on control cells without removing fuel assemblies, it shall be demonstrated that the core can be made subcritical by a margin of 0.25 percent Δk at any time during the maintenance with the strongest operable control rod