



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 10, 1994

Docket No. 52-004

Mr. Patrick W. Marriott, Manager
Licensing & Consulting Services
GE Nuclear Energy
175 Curtner Avenue
San Jose, California 95125

Dear Mr. Marriott:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE SIMPLIFIED
BOILING WATER REACTOR (SBWR) DESIGN (Q920.1-Q920.2)

The staff has determined that it needs additional information to support its review activities related to the SBWR design certification. Some additional information on the safeguards material provided in Chapter 13 of the standard safety analysis report is needed (Q920.1-Q920.2).^{*} Please provide a written response to the enclosed questions within 90 days of the date of this letter.

You have previously requested that portions of the information submitted in the August 1992, application for design certification of the SBWR plant, as supplemented in February 1993, be exempt from mandatory public disclosure. The staff has not completed its review of your request in accordance with the requirements of 10 CFR 2.790; therefore, that portion of the submitted information is being withheld from public disclosure pending the staff's final determination. The staff concludes that this RAI does not contain those portions of the information for which you are seeking exemption. However, the staff will withhold this letter from public disclosure for 30 calendar days from the date of this letter to allow GE the opportunity to verify the staff's conclusions. If, after that time, you do not request that all or portions of the information in the enclosure be withheld from public disclosure in accordance with 10 CFR 2.790, this letter will be placed in the NRC's Public Document Room.

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^{*}The numbers in parentheses designate the tracking numbers assigned to the questions.

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Mr. Patrick W. Marriott

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This RAI affects nine or fewer respondents, and therefore is not subject to review by the Office of Management and Budget under P.L. 96-511.

If you have any questions regarding this matter, please contact me at (301) 504-1178 or Mr. Son Ninh at (301) 504-1125.

Sincerely,

(Original signed by)

Melinda Malloy, Project Manager
Standardization Project Directorate
Associate Directorate for Advanced Reactors
and License Renewal
Office of Nuclear Reactor Regulation

Enclosure:
RAI on the SBWR Design

cc w/enclosure:
See next page

Distribution (w/enclosure):

*Central File	PDST R/F	MMalloy
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*To be held for 30 days

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DATE	03/9/94	03/9/94	03/9/94	03/10/94

OFFICIAL DOCUMENT NAME: SBWR9417.MM

Mr. Patrick W. Marriott
GE Nuclear Energy

Docket No. 52-004

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REQUEST FOR ADDITIONAL INFORMATION (RAI) ON THE
SIMPLIFIED BOILING WATER REACTOR (SBWR) DESIGN

Safeguards

- 920.1 Chapter 9, Section 5.2.2.1 of Volume III of the Electric Power Research Institute (EPRI) Advanced Light Water Reactor (ALWR) Utility Requirements Document (URD) states that a vulnerability analysis should be performed prior to finalizing the design to analyze issues associated with insider and outsider sabotage. In Section 5.2.2 of the staff's Final Safety Analysis Report on Volume III of the URD, the staff stated that it would review the vulnerability analysis as part of its review of individual applications. Provide a SBWR vulnerability analysis addressing insider and outsider sabotage for staff review.
- 920.2 Chapter 9, Section 5.2.3 of Volume III of the EPRI ALWR URD states that a comprehensive listing and evaluation of all vital equipment shall be developed. Section 13.6.3.3 of the SBWR standard safety analysis report (SSAR) contains a list of vital areas and vital equipment within those areas, however, the list is not comprehensive and specific enough for the staff to determine if all vital equipment has been identified. In accordance with Section 13.6 of the Standard Review Plan and its Review Guideline 17 (see NUREG-1178, "Vital Equipment/Area Guidelines Study; Vital Area Committee Report," Appendix B), the staff considers that all seismic Category I systems and applicable portions of their supporting and auxiliary systems should be designated as vital equipment in the sense of 10 CFR Part 73. If the equipment is classified as seismic Category I and not considered vital, sound technical basis needs to be provided. Provide a specific, comprehensive listing of vital equipment for the SBWR and the necessary evaluation.

Enclosure