

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

<u>CITY PUBLIC SERVICE BOARD OF SAN ANTONIO</u>

CENTRAL POWER AND LIGHT COMPANY

C'TY OF AUSTIN, TEXAS

DOCKET NO. 50-499

SOUTH TEXAS PROJECT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48 License No. NPF-80

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Houston Lighting & Power Company* (HL&P) acting on behalf of itself and for the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and City of Austin, Texas (COA) (the licensees) dated January 25, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*Houston Lighting & Power Company is authorized to act for the City Public Service Board of San Antonio, Central Power and Light Company and City of Austin, Texas and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

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- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-80 is hereby amended to read as follows:
 - 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 48, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 10 days from its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Signa (Bed

Suzanne C. Black, Director Project Directorate IV-2 Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 1, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 48

FACILITY OPERATING LICENSE NO. NPF-80

DOCKET NO. 50-499

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

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SPECIAL TEST EXCEPTIONS

3/4.10.5 POSITION INDICATION SYSTEM ~ SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.10.5 The limitations of Specification 3.1.3.3 may be suspended during the performance of individual full-length shutdown and control rod drop time measurements provided;

- a. Only one shutdown or control bank is withdrawn from the fully inserted position at a time, and
- b. The rod position indicator is OPERABLE during the withdrawal of the rods.*

APPLICABILITY: MODES 3, 4, and 5 during performance of rod drop time measurements.

ACTION:

With the Position Indication Systems inoperable or with more than one bank of rods withdrawn, immediately open the Reactor trip breakers.

SURVEILLANCE REQUIREMENTS

4.10.5 The above required Position Indication Systems shall be determined to be OPERABLE within 24 hours prior to the start of and at least once per 24 hours thereafter during rod drop time measurements by verifying the Demand Position Indication System and the Digital Rod Position Indication System agree:

- a. Within 12 steps when the rods are stationary, and
- b. Within 24 steps during rod motion.

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^{*}This requirement is not applicable during the initial calibration of the Digital Rod Position Indication System provided: (1) K_{eff} is maintained less than or equal to 0.95, and (2) only one shutdown or control rod bank is withdrawn from the fully inserted position at one time.

3/4.10 SPECIAL TEST EXCEPTIONS

3/4.10.7 DNB PARAMETERS SURVEILLANCE EXEMPTION FOR 2RE03

LIMITING CONDITION FOR OPERATION

3.10.7 The Surveillance Requirement of 4.2.5.3 to perform a precision heat balance to determine RCS flow at least once per 18 months is suspended for Unit 2 until the precision RCS heat balance flow measurement can be completed after entering MODE I after 2RE03 provided Reactor Power is maintained less than or equal to 75% RTP.

<u>APPLICABILITY</u>: This Specification is effective ONLY for Unit 2 cycle 4 while in MODE 1 until the requirements of 4.2.5.3 are met.

ACTION:

With Reactor Power greater than 75% RTP, restore Reactor Power to less than or equal to 75% RTP immediately.

SURVEILLANCE REQUIREMENTS

4.10.7.1 Verify Reactor Power is less than or equal to 75% RTP every 4 hours until the precision heat balance RCS flow verification is complete.

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SPECIAL TEST EXCEPTIONS

3/4.10.5 POSITION INDICATION SYSTEM - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.10.5 The limitations of Specification 3.1.3.3 may be suspended during the performance of individual full-length shutdown and control rod drop time measurements provided;

- Only one shutdown or control bank is withdrawn from the fully inserted a. position at a time, and
- b. The rod position indicator is OPERABLE during the withdrawal of the rods.*

APPLICABILITY: MODES 3, 4, and 5 during performance of rod drop time measurements.

ACTION:

With the Position Indication Systems inoperable or with more than one bank of rods withdrawn, immediately open the Reactor trip breakers.

SURVEILLANCE REQUIREMENTS

4.10.5 The above required Position Indication Systems shall be determined to be OPERABLE within 24 hours prior to the start of and at least once per 24 hours thereafter during rod drop time measurements by verifying the Demand Position Indication System and the Digital Rod Position Indication System agree:

- a. Within 12 steps when the rods are stationary, and
- Within 24 steps during rod motion. b.

*This requirement is not applicable during the initial calibration of the Digital Rod Position Indication System provided: (1) Keff is maintained less than or equal to 0.95, and (2) only one shutdown or control rod bank is withdrawn from the fully inserted position at one time.

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3/4.10.6 CET AND RCS RTD CALIBRATION EXEMPTIONS FOR 2RE03

LIMITING CONDITION FOR OPERATION

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<u>APPLICABILITY</u>: This Specification is effective ONLY for Unit 2 refueling outage 2RE03 while in MODES 3, 4, and 5.

ACTION:

With RCS boron concentration less than the refueling $K_{eff} = 0.95$ value immediately initiate and continue boration at a greater than or equal to 30 gpm of a solution containing greater than or equal to 7000 ppm boron or equivalent until RCS boron concentration is greater than or equal to the refueling $K_{eff} = 0.95$ value.

SURVEILLANCE REQUIREMENTS

- 4.10.6.1 a. Verify the Core Exit Thermocouples and the Reactor Coolant System Resistance Temperature Detectors calibration procedure is completed and the minimum required instruments are declared OPERABLE prior to entering MODE 2.
 - b. The boron concentration of the RCS shall be determined by chemical analysis at least once per 24 hours.

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Unit 2 - Amendment No. 48

3/4.10 SPECIAL TEST EXCEPTIONS

3/4.10.7 DNB PARAMETERS SURVEILLANCE EXEMPTION FOR 2RE03

LIMITING CONDITION FOR OPERATION

3.10.7 The Surveillance Requirement of 4.2.5.3 to perform a precision heat balance to determine RCS flow at least once per 18 months is suspended for Unit 2 until the precision RCS heat balance flow measurement can be completed after entering MODE 1 after 2RE03 provided Reactor Power is maintained less than or equal to 75% RTP.

APPLICABILITY: This Specification is effective ONLY for Unit 2 cycle 4 while in MODE 1 until the requirements of 4.2.5.3 are met.

ACTION:

With Reactor Power greater than 75% RTP, restore Reactor Power to less than or equal to 75% RTP immediately.

SURVEILLANCE REQUIREMENTS

4.10.7.1 Verify Reactor Power is less than or equal to 75% RTP every 4 hours until the precision heat balance RCS flow verification is complete.