



April 8, 1994

Docket No. 50-423
B14800

Re: 10CFR50.46(a)

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 3
Plan for Reanalysis to Address Small Break
Loss-of-Coolant Accident - Safety Injection in the Broken Loop

By letter dated November 4, 1993,⁽¹⁾ Northeast Nuclear Energy Company (NNECO) reported a significant change to the Westinghouse NOTRUMP small break loss-of-coolant accident (LOCA) evaluation model currently used in the licensing basis for Millstone Unit No. 3. Since the change has been determined to be a significant change to the model as defined in 10CFR50.46(a)(3)(i), the following will address the requirement to propose a plan for reanalysis as required in 10CFR50.46(a)(3)(ii).

At the January 12, 1994, meeting between Westinghouse and the NRC, Westinghouse agreed to provide to the NRC an addendum to WCAP-10054-P-A describing the safety injection (SI) model used in NOTRUMP including SI to the broken loop. This addendum will reference the improved condensation model described in WCAP-11767 and provide justification for application of this model to small break LOCA calculations. The NRC will be requested to formally review the addendum to WCAP-10054. Since the NRC review may affect the changes to the NOTRUMP evaluation model, NNECO proposes not to reanalyze for the purpose of addressing the SI in the broken loop issue prior to completion of the NRC's review. In the interim, NNECO will track the Peak Cladding Temperature (PCT) change reported in our November 4, 1993, letter (+150°F/-150°F) as a permanent change to the calculated PCT for Millstone Unit No. 3. Once the NRC has completed review of the NOTRUMP addendum on SI in the broken loop, and, if resolution of this issue results in a change to the NOTRUMP coding, NNECO would propose to capture the new model at the next licensing action requiring

(1) J. F. Opeka letter to U.S. Nuclear Regulatory Commission, "Millstone Nuclear Power Station, Unit No. 3, Reporting of Changes to, and Errors in, Emergency Core Cooling System Models or Applications," dated November 4, 1993.

ADD 1/0

U.S. Nuclear Regulatory Commission
B14800/Page 2
April 8, 1994

reanalysis of the small break LOCA and for which NOTRUMP would be used as the evaluation model. If the NOTRUMP evaluation model coding is not changed as a result of resolution of this issue, then NNECO proposes to not reanalyze but to report the net effect on PCT identified in the NRC's Safety Evaluation Report as a permanent change at the next required 10CFR50.46 report.


We trust that this information satisfies the reporting requirements of 10CFR50.46 (a) (3) (ii).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: J. F. Opeka
Executive Vice President

BY:


E. A. DeBarba
Vice President

cc: T. T. Martin, Region I Administrator
V. L. Rooney, NRC Project Manager, Millstone Unit No. 3
P. D. Swetland, Senior Resident Inspector, Millstone Unit
Nos. 1, 2, and 3