

MAY 12 1981

MEMORANDUM FOR: Darrell G. Eisenhut, Director
Division of Licensing

FROM: G.C. Lainas, Assistant Director
for Safety Assessment
Division of Licensing

SUBJECT: SUMMARY OF THE OPERATING REACTOR EVENTS MEETING
ON MAY 6, 1981

On May 6, 1981 an Operating Reactor Event meeting was held to brief the Office Director, the Division Directors, and their representatives on events which occurred since the last meeting. The list of meeting attendees is included as Attachment 1.

The events discussed and the significant elements of these events are presented in Attachment 2. In addition to these events, the assignment of follow-up review responsibility was discussed. The assignments made during this meeting and the status of previous assignments are presented in Attachment 3.

The next Operating Reactor Events meeting is scheduled for May 13, 1981.

G.C. Lainas

G.C. Lainas, Assistant Director
for Safety Assessment
Division of Licensing

Attachments:
As stated

cc w/attachments:
Meeting Attendees

| | |
|--------------|--------------|
| ORAB Members | G. Zech |
| H. Denton | T. Novak |
| E. Case | G. Lainas |
| R. Mattson | R. Tedesco |
| R. Vollmer | J. Olshinski |
| S. Hanauer | G. Holahan |
| T. Murley | H. Thompson |

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|---------|-----------|--------------|------------|----------|--|--|
| OFFICE | DL:ORAB | DL:ORAB/SL | DL:ORAB/BC | DL:AD/SA | | |
| SURNAME | Swooke:sh | GHolahan:sah | Olshinski | GLainas | | |
| DATE | 5/12/81 | 5/12/81 | 5/12/81 | 5/12/81 | | |

MEETING ATTENDEES

S. Wookey
A. Oxforth
B. Siegel
J. VanVliet
K. Eccelston
T. Novak
T. Murley
R. Mattson
R. DeYoung
N. Moseley
J. Sniezek
B. Mills
D. Eisenhut
G. Lainas
J. Olshinski
E. Case
J. Heltemes
R. Purple
G. Holahan
J.P. Knight
J. Kramer
S. Varga

OPERATING REACTOR EVENTS MEETING

MAY 6, 1981

0 BEAVER VALLEY UNIT 1, MAY 2, 1981 PLANT TRIP - EMERGENCY PREPAREDNESS PLAN IMPLEMENTED - UNUSUAL EVENT PNO-I-81-59

- WHILE AT 100% POWER, CONDUCTING STEAM GENERATOR MOISTURE CARRYOVER TESTING. 1 CURIE OF NA-24 TRACER IN FEEDWATER.
- "A" SG FEEDWATER REGULATING VALVE FAILED OPEN, REACTOR TRIP ON HIGH SG "A" LEVEL.
- NORMAL TRIP RESPONSE, EXCEPT STEAM DUMP TO CONDENSER SEEMED SLUGGISH.
- SUSPECTED MAIN STEAM CODE SAFETY VALVE OPENING.
- LICENSEE DECLARED UNUSUAL EVENT, EMERGENCY PREPAREDNESS PLAN PARTIALLY IMPLEMENTED FOR ONSITE AND OFFSITE RADIOLOGICAL MONITORING. NONE DETECTED.
- INSPECTION FOLLOWING TRANSIENT IDENTIFIED FEEDWATER VALVE AND PIPING SUPPORT DAMAGE.
- IE FOLLOW UP.

0 FORT CALHOUN, APRIL 28, 1981 - PARTIAL LOSS OF D.C.

- PARTIAL LOSS OF DC CAUSED LOSS OF LOAD TRIP, PARTIAL LOSS OF COMPONENT COOLING WATER AND PARTIAL CONTAINMENT ISOLATION.
- LOSS OF DC CAUSED BY LOOSE WIRE IN FEEDER PANEL.

- OPERATOR SECURED RCP'S AFTER 4 MINUTES RUN TIME WITHOUT COMPONENT COOLING WATER. PUMP VENDOR, BYRON JACKSON, RECOMMENDS MAX. 12 MINUTES AT 250°F.
- UNIT ON NATURAL CIRCULATION FOR 2 HOURS USING ATMOSPHERIC DUMPS.
- ALL SYSTEMS APPEARED TO FUNCTION NORMALLY.

o MONTICELLO, APRIL 27, 1981

- LOSS OF OFFSITE POWER, LOSS OF NORMAL DECAY HEAT REMOVAL DURING REFUELING OUTAGE.
- THE "B" RHR LOOP OUT OF SERVICE FOR MAINTENANCE.
- THE "A" RHR PUMP BREAKER RACKED OUT WHILE ENERGIZED, CURRENT SURGE TRIPPED OTHER BREAKERS SUPPLIED BY SAME BUS AND THE OUTPUT BREAKER FROM THE AUXILIARY TRANSFORMER. A FIRE STARTED IN THE BREAKER CABINET.
- OFFSITE POWER LOST FOR 10 MINUTES, HEAT UP OF REACTOR COOLANT 0.5°F PER HOUR.
- FUEL POOL COOLING SYSTEM WAS USED TO REMOVE DECAY HEAT FROM REACTOR CAVITY.
- THE "B" RHR LOOP RETURNED TO SERVICE IN 8 HOURS.

o BRUNSWICK UNIT 1, APRIL 25, 1981

- LOSS OF NORMAL DECAY HEAT REMOVAL, COLD SHUTDOWN FOR MAINTENANCE OUTAGE.
- ONE RHR LOOP OUT OF SERVICE FOR MAINTENANCE.
- THE BAFFLE IN THE OPERATING RHR HEAT EXCHANGER FAILED, ALLOWING COOLING WATER TO BYPASS THE TUBE BUNDLE.
- BAFFLE FAILURE CAUSED BY HIGH ΔP 15 PSI ACROSS THE HEAT EXCHANGER NORMAL OPERATING ΔP 1 PSI. HIGH ΔP CAUSED BY CLAMS IN THE HEAT EXCHANGER.
- DECAY HEAT REMOVED BY SPENT FUEL POOL COOLING SYSTEM.
- HEAT EXCHANGER REPAIRED AND RETURNED TO SERVICE IN 16 HOURS.

o BRUNSWICK UNIT 2, MAY 5, 1981

- AT MIDNIGHT SHUTTING DOWN, 2 INOPERABLE RHR HEAT EXCHANGERS, DISPLACED BAFFLE IN ONE OF TWO HEAT EXCHANGERS, ONE IN SERVICE OPERATING AT REDUCED ΔP ACROSS U-TUBES.
- AT 7:30 AM GROUP I ISOLATION (REACTOR TRIP AND MSIV CLOSURE, USING RCIC FOR DECAY HEAT REMOVAL).
- 3 PREVIOUS BAFFLE FAILURES.
- IE FOLLOW UP.

o 3 INADVERTENT SAFETY INJECTIONS INITIATED BY SECONDARY SIDE

- SALEM UNIT 2, SHUTDOWN AND COOLING DOWN, DUMP VALVE SETPOINT DRIFT - STEAM LINE ΔP SI.
- SEQUOYAH UNIT 1, SHUTDOWN TESTING MSIV'S HIGH STEAM FLOW AND LO TAVE SI.
- SEQUOYAH UNIT 1 100% GENERATOR TRIP TEST PROBLEM TRANSFERRING RCP'S - STEAM LINE - ΔP SI.

o COOPER, APRIL 28, 1981

- FAILED TURBINE BLADES, VIBRATIONS EXPERIENCED IN LOW PRESSURE TURBINE.
- 3 BLADES MISSING, SHROUD MISSING FROM A GROUP OF 6 BLADES, NINE BLADES HAVE BROKEN ROOTS.
- SAME TURBINE ROTOR WHICH WAS GIVEN TEMPORARY FIX IN MARCH OF 1980.

o MAINE YANKEE, MAY 2, 1981

- REACTOR COOLANT PUMP SEALS FAILED.
- FAILURE OF #2 AND #3 SEALS.
- OPERATION CONTINUING WITH 2 SEALS STILL FUNCTIONING UNTIL REFUELING.
- EMERGENCY TECH SPEC 5/5/81 TO ALLOW CLOSURE OF LOOP ISOLATION VALVES TO CLOSE FROM CONTROL ROOM IF NECESSARY.

MULTIPLE FAILURE INFORMATION ON THE 73 EVENTS
DISCUSSED AT OPERATING REACTOR BRIEFINGS DEC 1980-MAY 1981

INDEPENDENT FAILURES

| <u># FAILURES</u> | <u>FAILURE RATE</u> |
|-------------------|---------------------|
| 2 | 5 IN 5 MONTHS |
| 3 | 1 IN 5 MONTHS |
| 4 | 2 IN 5 MONTHS |
| \geq 5 | 0 IN 5 MONTHS |

CONSEQUENTIAL FAILURES

| <u># CONSEQUENTIAL FAILURES</u> | <u>FAILURE RATE</u> |
|---------------------------------|---------------------|
| 1 | 3 IN 5 MONTHS |
| 2 | 1 IN 5 MONTHS |
| 3 | 0 IN 5 MONTHS |
| \geq 4 | 0 IN 5 MONTHS |

THE FOLLOWING PRELIMINARY NOTIFICATIONS RELATING TO THE FOLLOWING ACTIONS WERE RECEIVED DURING THE PAST 2 WEEKS.

- PNO-I-81-53, PEACH BOTTOM UNIT 2 - UNSCHEDULED SHUTDOWN.
- PNO-I-81-54, BEAVER VALLEY UNIT 2 - ONSITE FATALITY.
- PNO-I-81-55, HADDAM NECK - INADVERTENT TRANSPORT OF CONTAMINATED MATERIAL OFFSITE.
- PNO-TMI-81-09, THREE MILE ISLAND UNIT 2 - EPICOR-II RESIN LINER SHIPMENT.
- PNO-I-81-57, PEACH BOTTOM UNITS 2 AND 3 - RADIOACTIVE WASTE DISCHARGE LINE BREAK.
- PNO-I-81-58, PEACH BOTTOM UNITS 2 AND 3 - POSSIBLE STRIKE BY CONTRACT GUARD FORCE.
- PNO-I-81-59, BEAVER VALLEY UNIT 1 - PLANT TRIP AND UNPLAINED RELEASE - UNUSUAL EVENT.
- PNO-I-81-60, STEPHAN CHEMICAL Co. - THORIUM CONTAMINATION IN MAYWOOD, NEW JERSEY.
- PNO-II-81-31, BRUNSWICK UNIT 1 - LOSS OF NORMAL DECAY HEAT REMOVAL.
- PNS-II-81-07, NUCLEAR FUEL SERVICES, INC. - NUCLEAR FUEL SERVICES APRIL 30, 1981 REPORTED INVENTORY DIFFERENCE.
- PNO-III-81-42, MONTICELLO - HPCIS INOPERABLE DUE TO CHECK VALVE FAILURE.
- PNO-III-81-40c, DUANE ARNOLD - POSSIBLE DEFECTS IN RECIRCULATION PIPING AND THERMAL SLEEVE WELDS.
- PNO-III-81-43, DAVIS-BESSE - PLANT STARTUP AFTER STEAM GENERATOR REPAIR.
- PNO-III-81-44, POINT BEACH UNIT 2 - STEAM GENERATOR TUBE DEGRADATION.

- PNO-III-81-45, MONTICELLO - LOSS OF OFFSITE POWER DURING REWELING OUTAGE.
- PNO-IV-81-11, COOPER - FAILED TURBINE BLADES.
- PNO-V-81-25, SAN ONOFRE UNIT 1 - EARTHQUAKE AT SAN ONOFRE UNIT 1.
- PNO-V-81-26, SAN ONOFRE UNIT 1 - INVESTIGATION OF ALLEGED THEFT OF POTENTIALLY CONTAMINATED HAND TOOLS.