DECOMMISSIONING COST STUDY for the YANKEE NUCLEAR POWER STATION

Prepared for

YANKEE ATOMIC ELECTRIC COMPANY

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1. SUMMARY

The Yankee Nuclear Power Station (YNPS) is located in a valley in the town of Rowe, Massachusetts on the east bank of the Deerfield River, three-quarters of a mile south of the Vermont-Massachusetts border. The site consists of approximately 2,000 acres straddling the Deerfield River in the towns of Rowe and Monroe Bridge, Massachusetts. The station is owned by the Yankee Atomic Electric Company (YAEC).

Construction of the station was completed in June 1960. The operating license was received in July 1960, with commercial operation beginning in July 1961. The operating license, amended in 1987, was due to expire in July of 2000 after an expected 40 year operating life. However, following a shutdown of plant operations in October of 1991, a decision was made to cease power operation. As such, this study was initiated to quantify, on a site specific basis, the potential costs of decommissioning the facility, so that YAEC can better evaluate its available options.

This study provides cost, schedule, waste generation/disposition and radiation exposure estimates associated with decommissioning YNPS following the cessation of station operation (October 1, 1991). The alternatives evaluated are DECON (Prompt Removal/ Dismantling) and SAFSTOR (Mothball with Delayed Dismantling). The estimates delineated within this document reflect expenditures following receipt of the dismantling order from the NRC. The costs to maintain the facility in its present configuration, until such time, are not addressed within this study.

<u>DECON</u> of a power reactor consists of removing from the site all fuel assemblies and source material, radioactive fission and corrosion products, and all other radioactive materials having activities above NRC release limits. The facility operator may then have unrestricted use of the site with no requirement for a license. This scenario is equivalent to the DECON mode as described in the rule on decommissioning issued by the Nuclear Regulatory Commission (NRC), "General Requirements for Decommissioning Nuclear Facilities." This study further assumes that the remainder of the reactor facility will be dismantled and all vestiges removed. The site is then restored and made available for alternative use.

<u>SAFSTOR</u> consists of placing and maintaining the facility in protective storage. During mothball operations, the plant staff conducts general plant decontamination activities, radiation surveys, and removal (including processing) of radioactive waste materials remaining from operations. In addition, a possession-only license is secured (if not done previously) and the security, surveillance and maintenance plans for the delay period are implemented. Delayed dismantling (decontamination) activities are initiated such that license termination is accomplished within the 60 year time period set by the NRC. As with the DECON alternative, this study further assumes that the remainder of the reactor facility is dismantled and the site is restored to its original landscape. An alternative to immediate decommissioning is one which provides for delayed decommissioning of a power reactor under certain conditions, i.e., if decommissioning is completed within 60 years of the conclusion of operations. The NRC can approve a decommissioning plan which provides for completion of decommissioning beyond 60 years if there is some demonstrated benefit to public health and safety (Ref. 1). The SAFSTOR alternative, evaluated in this study, assumes that the station will be decommissioned within 12 years of its cessation of power operations.

There are definite advantages to the DECON alternative. The alternative is less costly, in 1992 dollars, than a scenario involving extended delays in the station dismantling. (The ultimate cost of any alternative will depend upon future economic factors such as inflation and policy factors such as future NRC regulations and waste policy decisions and actions.) The NRC recognizes the advantages of DECON (Ref. 2) in that it (1) immediately eliminates a potential long term safety hazard and (2) those individuals familiar with the nuclear facility will still be available to support the dismantling effort.

The cost of the SAFSTOR alternative is significantly increased by the cost of maintaining the station in protective storage. However, SAFSTOR does have some advantage over the DECON alternative. Primarily, the dormancy period provides a decay period for the residual radioactivity, resulting in lower personnel radiation exposures during dismantling than are incurred in the DECON alternative, and a potential savings in the disposal cost for the waste volume generated during decommissioning operations.

Conversely, the utility continues to incur the cost of staffing and maintaining the site in the SAFSTOR alternative. In addition, at the end of the SAFSTOR dormancy period, the station must be partially reactivated (those systems necessary to support decommissioning operations) and/or replacement services must be procured. Refurbishment activities may involve requalifying the cranes and other lifting devices, reactivating electrical, lighting, air handling, and other service systems. The procurement of waste processing/treatment services would be necessary if plant systems could not be salvaged.

This study provides the costs for decommissioning the YNPS under current requirements based on present day costs and available technology. The cost and schedule estimates presented are based on the complete removal of all components and structures within the property lines, as the station is presently configured, except as noted within the body of this report. The costs for the DECON and SAFSTOR scenarios are shown in Table 1.1. Table 1.1 is a summary taken from the detailed cost tables, Tables 4.2 and 4.3, and scheduling analyses presented in Section 5.

While the disposal cost of spent fuel assemblies generated during plant operations is not considered a decommissioning expense, the presence of those assemblies on-site does have an impact on the cost of decommissioning. This study recognizes that the current spent fuel storage facilities at the YNPS will be active approximately fifty-six (56) months after the October 1991 shutdown. During this period, the fuel will be transferred to dry storage casks where the assemblies will remain for a period up to twenty-three (23) years. The overall

twenty-eight (28) year on-site storage period is dictated by YAEC projections of the availability of the United States Department of Energy (DOE) to accept spent nuclear fuel at its yet-to-be developed high level waste repository.

TABLE 1.1

COST AND SCHEDULE ESTIMATE SUMMARY

	Cost, 92\$ (Thousands)	Schedule (Months)
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DECON (Prompt Removal/Dismantling)		
Period 1 Operations	48,132	17.9
Period 2 Decommissioning Activities	120,385	19.6
Period 3 Site Restoration	13,057	11.3
Post Period 3 Dry Fuel Storage	50,561	239.1
TOTAL	232,135	287.9
SAFSTOR (Mothball with Delayed Dismantling)		
Period 1 Operations	31,253	17.9
Period 2 Dormancy	16,093	30.3
Period 3 Operations	34,150	17.9
Period 4 Decommissioning Activities	112,009	18.4
Period 5 Site Restoration	13,006	11.3
Post Period 5 Dry Fuel Storage	40.606	192.0
TOTAL	247,117	287.9

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2. INTRODUCTION

2.1 OBJECTIVE OF STUDY

The objective of this study is to prepare estimates of the cost, schedule, occupational exposure and waste volume generated in decommissioning the YNPS site including common and supporting facilities. The decommissioning alternatives evaluated are DECON (Prompt Removal/Dismantling) and SAFSTOR (Mothball with Delayed Dismantling).

The decommissioning scenarios are evaluated given the actual 31.2 year energy producing life operation for the station.

This study relies upon state-of-the-art estimating techniques, current regulations, and an enhanced experience base for projecting the current cost to decommission the YNPS.

2.2 SITE DESCRIPTION

The YNPS is located in a valley in the town of Rowe, Massachusetts on the east bank of the Deerfield River, three-quarters of a mile south of the Vermont-Massachusetts border. The site consists of approximately 2,000 acres straddling the Deerfield River in the towns of Rowe and Monroe Bridge, Massachusetts. The station is operated by the Yankee Atomic Electric Company (YAEC).

The Nuclear Steam Supply System (NSSS) consists of four closed loops connected in parallel to the reactor vessel. The principal components of each of the loops are two gate-type motor-operated stop valves, a steam generator, a canned- motor-type circulating pump, a check valve, and related piping. Pressure control is accomplished in the pressurizer vessel by maintaining a steam-water volume at an equilibrium temperature. The NSSS operates at a limit of 600 MWt (Megawatts thermal) or approximately 186 MWe (Megawatts electric). This system was supplied by the Westinghouse Electric Corporation.

The NSSS is located inside of the Vapor Container (VC). This is a spherical steel pressure vessel designed to contain the pressure build-up resulting from a major break in the Main Coolant System (MCS), or a main steam or feed line break releasing the contents of the secondary side of one steam generator into the VC. The VC is a nominal 125-foot diameter steel sphere with the equator approximately 86 feet above grade. It is supported by 16 steel columns. The VC houses, but does not support the Reactor Support Structure (RSS). The RSS consists of two concentric reinforced concrete cylinders which support the NSSS. The RSS is supported by eight reinforced concrete columns, six exterior, and two interior. These

columns are isolated from the VC by bellows where the steel encased concrete columns pass through the spherical shell. Total height of the RSS, including support columns is approximately 122 feet. The support columns are braced by 4-inch diameter steel rods with turn buckles that serve as cross bracing for lateral loads. The steel columns are supported by reinforced concrete pedestals. The pedestals are founded on massive concrete spread footings.

Heat produced in the reactor is converted to electrical energy by the Main Steam System. A turbine-generator system converts the thermal energy of steam produced in the steam generators into mechanical shaft power and then into electrical energy. The unit's turbine-generator consists of one high pressure and one low pressure turbine driving a direct-coupled generator. The turbine is operated in a closed feedwater cycle which condenses the steam; the heated feedwater is returned to the steam generators. Heat rejected in the main condenser is removed by the Circulating Water System.

Sherman Pond provides makeup for the Circulating Water System and serves as the normal ultimate heat sink for the YNPS. Water enters the intake through a vortex-eliminator and is drawn through a 120-inch corrugated steel pipe to the entrance of the concrete screenwell. The screenwell has a divided inlet channel with a traveling screen and one circulating water pump in each side. The circulating water is conducted to the Turbine Building and into the condenser inlet pipe, where it enters the inlet water boxes, providing cooling to the main condensers. The outlet water boxes are connected to a concrete seal pit located outside of the Turbine Building. The discharge water is then passed over a weir and returned to Sherman Pond, completing the loop.

2.3 REGULATORY GUIDANCE

The U.S. Nuclear Regulatory Commission (NRC) provides decommissioning guidance in the 10 CFR 50.75 rule "General Requirements for Decommissioning Nuclear Facilities" (Ref. 1) in addition to that previously set forth in Regulatory Guide 1.86 (Ref. 3). This rule defines three decommissioning alternatives acceptable to the NRC, i.e., DECON, (prompt removal/ dismantling), SAFSTOR (mothball), and ENTOMB (entombment).

<u>DECON (Prompt Removal/Dismantling)</u> is defined by the NRC as "the alternative in which the equipment, structures, and portions of a facility and site containing radioactive contaminants are removed or decontaminated to a level that permits the property to be released for unrestricted use shortly after cessation of operations."

<u>SAFSTOR (Mothball)</u> is defined as "the alternative in which the nuclear facility is placed and maintained in a condition that allows the nuclear facility to be safely stored and subsequently decontaminated (deferred decontamination) to levels that permit release for unrestricted use." ENTOMB (Entombment) is defined as "the alternative in which radioactive contaminants are encased in a structurally long-lived material, such as concrete; the entombed structure is appropriately maintained and continued surveillance... carried out until the radioactivity decays to a level permitting unrestricted release of the property." However, this process is restricted in overall duration to 60 years and therefore limited in application unless it can be shown that a longer duration is necessary to protect the health and safety of the public.

Prior to the new rule, no endpoint was identified for either the SAFSTOR or ENTOMB process, i.e., a facility could remain in either state indefinitely. This is no longer the case as the rule places upper limits on the completion of the decommissioning process. Consequently, with the new restrictions, the SAFSTOR and ENTOMB options are no longer decommissioning alternatives in themselves, as neither terminates the license for the site. At the end of the dormancy periods (up to 60 years), both alternatives would still require site decontamination/ decommissioning.

In most situations the DECON alternative is the preferred mode of decommissioning. This decommissioning alternative is favored because (1) it immediately eliminates a potential long term safety hazard and (2) individuals familiar with the nuclear facility will still be available to support the dismantling effort. In addition, both the mothball and entombment alternatives still require eventual decontamination/decommissioning even after the maximum allowed dormancy durations. This results in higher overall costs as on-going dormancy expenses and reactivation costs offset the potential savings gained from the delay.

SAFSTOR, however, may become the default alternative for many other facilities. Public Law 99-240, the Low-Level Radioactive Waste Policy Amendments Act of 1985, gave the State the responsibility for providing for the disposal of low-level radioactive waste generated within the State. The law encouraged the formation of regional sites and provided for the exclusion of access to existing sites by January 1, 1993. Unfortunately, the siting of new facilities has been arduous, and in some cases impossible. None of the currently sited facilities will be available by the end of this year, and many states remain without any real alternatives. As such, several utilities are facing the potential of having to store low-level waste at the site for some indefinite time. This situation would preclude the use of the DECON option.

3. DECOMMISSIONING ALTERNATIVE DESCRIPTIONS

Both the DECON and SAFSTOR alternatives are examined for the YNPS. The common goal is the removal of all radioactive materials from the site and ultimate release of the site for unrestricted and/or alternative use.

The following section describes the basic activities necessary for the DECON alternative. Although detailed procedures for each activity required are not provided, and actual sequences of work may vary, these activity descriptions provide a basis for detailed engineering planning and scheduling at the time of decommissioning. A synopsis of the SAFSTOR alternative is provided in Section 3.2.

3.1 DECON (Prompt Removal/Dismantling)

This alternative deals with the immediate removal of all radioactive materials from the site after the cessation of operations. This study does not address the cost of the removal of spent fuel from the site because such costs are assumed to be covered by the 1 mill/kwhr U.S. Department of Energy (DOE) surcharge. However, the study does consider the on site presence of spent fuel and its potential constraint on decommissioning activities. In addition to the removal of radioactivity, this study also assumes the removal of the remaining structures from the site; thereby permitting return of the YNPS site for other use.

Decommissioning activities at YNPS will be initiated upon approval of the Decommissioning Plan (DP), and receipt of the decommissioning order. YAEC is currently projecting receipt in January of 1995. In the interim, YAEC will develop the DP, secure a possession-only license, and continue to maintain the spent fuel storage facility at the site.

3.1.1 Period 1: Preparations

Upon receipt of the decommissioning order, detailed preparations are undertaken to provide a smooth transition from current plant operations to site decommissioning activities. Final planning for activities and writing of activity specifications and detailed procedures also begin at this time. Starting in 1996, spent fuel will be transferred to dry storage facility. This process is envisioned to last some five months and may be carried out by existing plant personnel operating under the current technical specification requirements associated with fuel transfer.

3.1.1.1 Engineering and Planning

The DP will describe how it will remove all radioactive components and essentially all radioactivity from the YNPS site. The majority of the cost to develop this document is staff related and will be incurred in the years prior to the commencement of decommissioning operations.

This request for dismantling of the reactor and termination of the facility's license will include a detailed plan describing the organization and program that will be used during the decommissioning of the facility. The plan will accomplish the required tasks within the As Low As Reasonably Achievable (ALARA as defined in 10 CFR 20) guidelines for protection of personnel from exposure to radiation and radioactive contaminants. It will also clearly describe how YAEC will continue to protect the health and safety of the public and the environment during the dismantling activity.

Prior to the start of decommissioning operations, work begins on the documentation and planning necessary for both licensing change applications and for accomplishing the work required. The development of a decommissioning organization within the utility is essential to this planning. This development includes identifying the staff requirements and commitment of key personnel.

In preparation for a change in license, regulatory criteria applicable to decommissioning are reviewed. The existing technical specifications are reviewed and modified to reflect decommissioning requirements and to delete non-applicable operating specifications. The DP is prepared during this time.

In addition, an environmental assessment may be required by the NRC and all applicable records, i.e., as-built or revised drawings and specifications, operating records, and site-specific background data, will be needed.

Much of the work in the development of the DP is also relevant to the development of the detailed engineering plans and procedures. This work includes:

- Site preparation plans for decommissioning activities,
- Detailed procedures and sequences for removal of systems and components,
- Procedures for sectioning and disposing of the reactor vessel and its internals,
 - Plans for decontamination of structures and systems,

- Design/procurement and testing of special equipment,
- Identification/selection of specialty contractors,
- Procedures for removal and disposal of radioactive materials, and
- Sequential planning of activities to minimize conflicts with simultaneous activities.

3.1.1.2 Site Preparations

Following the receipt of the decommissioning order, and in preparation for actual decommissioning activities, the following activities will be initiated:

- Prepare site support and storage facilities as required.
- Implementation of a task force to maintain spent fuel storage requirements and transfer spent fuel assemblies to dry storage casks. This activity will take place in the Spent Fuel Pit area and will span some seventeen months. This period includes a twelve month construction duration for the cask storage compound. Upon completion of the transfer of fuel assemblies from the wet fuel pit to dry storage casks, decommissioning operations can continue unimpeded by the presence of spent fuel in wet storage. This activity may be carried out by existing plant personnel in accordance with standard operating technical specifications. The spent fuel will remain in the dry storage casks for approximately twenty-three years. This duration is based upon projections concerning DOE's availability to receive spent fuel assemblies at the yet to be constructed high level waste repository.
- Clean all plant areas of loose contamination and process all liquid and solid wastes.
- Conduct radiation surveys of work area contamination and general dose levels; major component, piping, and structure dose levels (including the reactor vessel and its internals) and internal piping contamination levels.
- Calculate residual byproduct material inventory for plant components, structures and systems, and normalize neutron flux profiles from operations to survey data for development of packaging and shipping requirements and decommissioning safety requirements.

- Determine shipping container requirements for activated materials and fabricate such containers.
- * Develop procedures for occupational exposure control, control and release of liquid and gaseous effluents, control of solid radwaste, site security and emergency programs, and industrial safety. This study presumes that the decommissioning of the YNPS is performed in accordance with now current regulations as delineated in Section 4.4.

With the completion of Period 1 activities, decommissioning operations at YNPS can commence.

3.1.2 Period 2: Decommissioning Operations and License Termination

For the DECON alternative the decommissioning operations involve the following:

- Construct temporary enclosures in existing facilities and arrange existing storage facilities to support the dismantling activities. These may include: changing rooms and "hot" laundry for increased work force, protected and open laydown areas to facilitate equipment removal and shipping operations, additional roads or modifications to existing roads to facilitate hauling and transportation, and additional airlocked access portals to control movement to and from contaminated areas.
- Design, procure, and install water cleanup system for removal of cutting residues and crud deposits from the reactor vessel and piping systems.
- Design and fabricate special shielding and contamination control envelopes, special tooling and remotely operated equipment. Modify the reactor cavity/refueling chute area (RC/RCA) in the VC to support segmentation activities, and prepare rigging for segmentation and removal of piping sections and components, including the reactor vessel and its internals.
- Procure required shipping casks, liners, and Low Specific Activity (LSA) containers from suppliers.
- * Disassemble reactor vessel internal components and transfer them to the staging area in the RC/RCA. Segment upper and lower core support structures and in-core instrumentation for packaging and disposition by shielded container. Cutting operations are performed under water with remote equipment.

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- * Conduct decontamination of components and piping systems as required. Remove, package and dispose of piping and components as they are no longer required to support the decommissioning process.
- Remove control rod drive housings and instrumentation tubes from reactor vessel head and cut housings and tubes into sections for disposal in shielded containers.
- Isolate neutron shield tank (NST) area and lower the water level in the RC/RCA to below the reactor vessel flange. Sever reactor vessel flange from vessel shell. Bolt flange to reactor vessel closure head and complete the package with steel plate. Decontaminate exterior surfaces for transport and disposal.
- Remove reactor coolant piping and pumps once the water level has dropped below the elevation of the reactor vessel inlet and outlet nozzles. Piping is placed in standard LSA containers; the reactor coolant pumps are sealed and decontaminated for transport and burial.
- Segment the reactor vessel shell and nozzle zone above the vessel support lugs. Cutting is performed in air above the elevation of the NST ring girder, and under water below the elevation of the top of the NST ring girder. In-air cutting of generators are performed within a contamination control envelope. Segments are removed from the reactor/NST and placed in the RC/RCA for packaging. Shielded containers are used for transport to the disposal facility.
- The NST and RPV hang freely from a ring girder supported on a ledge in the concrete primary shield wall. Vertical support for the NST and RPV must be provided prior to segmentation below the ring girder elevation. However, the bottom of the NST and RPV are inaccessible for the placement of a support structure. Therefore, an estimated 50 cubic yards of grout will be injected in the areas listed below to provide a solid foundation for the support of the NST and RPV which will transfer vertical loads to the bottom of the concrete reactor support structure. Grouted areas to include:

The void between the bottom of the NST and the concrete reactor support structure injected. Grout is injected through existing drain lines and pipe vents;

Inside of the NST to the elevation of the RPV lower head; and

The void between the bottom of the reactor vessel and the inside wall of the NST to the elevation of the RPV lower head.

Continue segmentation of the reactor vessel shell below the vessel support lugs. Cutting is performed under water below the elevation of the top of the NST. Segments are removed from the reactor/NST and placed in the RC/RCA for packaging. Shielded containers are used for transport to the disposal facility. The lower head is left intact.

Segment the walls of the NST as the removal of the reactor vessel exposes the NST wall to the cutting torch. Segments of the NST walls are removed from the reactor/NST area and placed in the RC/RCA for packaging. Shielded containers are used for transport to the disposal facility.

 Remove lower head from cavity and seal the package. Decontaminate exterior surfaces for transport and disposal.

Remove grout in NST cavity and segment inner NST wall. Remove grout inside NST and segment outer NST wall. Segments of the NST walls and grout rubble are removed from the reactor/NST area and placed in the RC/RCA for packaging. Shielded containers are used for transport to the disposal facility.

Remove systems and associated components as they become nonessential to the support of vessel disposition, other decommissioning operations or worker health (e.g., decommissioning waste processing systems, electrical systems, HVAC systems, water systems).

Remove concrete biological shield and all accessible contaminated concrete (excluding steam generator and pressurizer cubicles). If dictated by the steam generator and pressurizer removal scenarios, remove those portions of the associated cubicles necessary for access and component extraction.

Remove steam generators and pressurizer for shipment and burial. Decontaminate exterior surfaces, as required, and seal-weld all openings in steam generators and pressurizer. These components can serve as their own burial containers provided that all penetrations are properly sealed. Decontaminate all remaining containment structure areas including steam generator and pressurizer cubicles.

- Perform radiation survey to assure that the remaining portions of the containment structure are free of surface contamination and that containment integrity is no longer required.
 - Remove contaminated equipment, the liner, and material from the fuel storage area and any other contaminated areas once the Spent Fuel Pit has been emptied. Decontaminate, utilizing radiation and contamination control techniques, until radiation surveys indicate that the structure can be released for unrestricted access and conventional demolition.
- Decontaminate remaining structures and facilities on site. Remediate any contamination exterior to the site structures.
- Ship and bury all remaining adjoactive materials.
- Conduct final radiation survey to assure that all radioactive materials have been removed. This survey may coincide with final NRC site inspection.
- Following notification by YAEC of completion of the decontamination and disposal of components and materials from the facility, the NRC regional staff conducts an on-site survey to verify that the acceptable activity and contamination levels are satisfied. When the requirements are satisfied, the NRC can terminate the Part 50 license for the main facility. The site will retain a Part 72 license for the storage of spent nuclear fuel until such time that DOE is able to take receipt. At that time, the storage facility will be decontaminated, if required, and the structure dismintled. A final survey would allow the NRC to terminate the Part 72 license, ending its jurisdiction over the YNPS facility.

3.1.3 Period 3: Site Restoration

Following completion of the decommissioning operations, site restoration activities may begin. These activities will permit unrestricted access by the public, therefore, precluding liability of the owners with regard to persons using the site, and assure compliance with applicable codes. All building foundations are backfilled using non-contaminated concrete rubble with a structural fill to the grade elevation. Site areas affected by the dismantling activities are cleaned up and the plant area graded and landscaped as required. These activities include:

Demolition of the remaining interior portions of the Vapor Container. Internal floors (and walls if above grade) are removed from the lower

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levels upward, using controlled demolition techniques. Concrete rubble and other suitable materials can be utilized on site for fill.

Remaining buildings are then removed using conventional demolition techniques for above ground structures, including the Turbine Building, Primary Auxiliary Building, Diesel Generator Building, Service Building and other site structures. In addition, outside storage tanks are drained and removed.

Preparation of the final dismantling program report.

3.2 SAFSTOR (Mothball with Delayed Dismantling)

The SAFSTOR decommissioning alternative provides a condition that ensures public health and safety from residual radioactivity remaining at the site without the need for extensive modifications to the facility. While "mothball" is used to describe this alternative (Ref. 3), it is a misnomer since under SAFSTOR reactivation of the plant for commercial operation is not intended. During the SAFSTOR period the facility is left intact and all structures are maintained in a sound condition. All systems not required to be operational for maintenance and surveillance purposes during the dormancy period are drained, de-energized, and secured. Minimal cleaning/removal of loose continuation and/or fixation and sealing of remaining contamination is performed. All access points to contaminated areas are sealed and/or secured to provide controlled access for inspection and maintenance.

The engineering and planning requirements are similar to those for the DECON alternative although a shorter time period is expected for these activities. Site preparations are also similar to those for the DECON alternative. However, with the exception of required radiation surveys, the mobilization and preparation of site facilities is less extensive.

Prior to commencement of decommissioning operations, YAEC will file a Decommissioning Plan (DP) with the NRC describing how it will remove all radioactive components and essentially all radioactivity from the YNPS site. This request for eventual dismantling of the reactor and termination of the facility's license includes a detailed plan describing the organization and program that will be used during the decommissioning of the facility. The plan will accomplish the required tasks within the ALARA guidelines for protection of personnel from exposure to radioactive and non-radioactive contaminants. It will also clearly describe how YAEC will continue to protect the health and safety of the public and the environment during the dismantling activities.

Decommissioning activities at YNPS will be initiated upon approval of the Decommissioning Plan (DP), and receipt of the decommissioning order. YAEC is currently projecting receipt in January of 1995. In the interim, YAEC will develop

the DP, secure a possession-only license, and continue to maintain the spent fuel storage facility at the site. Other activities ongoing in the period prior to the receipt of the decommissioning order are:

- Drain/de-energize/secure all non-contaminated systems not required to support decommissioning operations.
- Dispose of contaminated filter elements and resin beds not required for processing wastes from decontamination activities.
- Drain/dc-energize/secure all contaminated systems. Decontaminate as required.
- Prepare lighting and alarm systems whose continued use is required. De-energize and/or secure portions of fire protection, electric power, and HVAC systems whose continued use is not required.
- Clean loose surface contamination from building access pathways.

The "Possession Only" license permits ownership and possession of fuel, by-product material and reactor components, but does not permit operation of the reactor. This license status, though permitting significant relief from the technical specifications and other program requirements, still requires adequate surveillance, monitoring and reporting.

After plant shutdown, modified technical specifications are implemented. Spent fuel and in-core source materials are isolated in the spent fuel storage facilities awaiting ultimate disposal or until they can be transferred to another facility. These steps may be carried out by plant personnel in accordance with standard operating procedures. All liquid and solid wastes are processed and removed for disposition off-site.

3.2.1 Period 1: SAFSTOR Operations

Following approval of the DP by the NRC, the NRC issues an order authorizing implementation. The DP may then be implemented by YAEC. With much of the preparation performed in the years prior to 1995, the SAFSTOR activity is abbreviated. The following activities are expected to be performed prior to dormancy:

- Perform final radiation survey of plant; post warning signs as appropriate.
- Erect physical barriers and/or secure all access to radioactive or contaminated areas, except as required for controlled access for inspection and maintenance.

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- Drain and decontaminate spent fuel pool once all assemblies have been moved to the dry cask storage facility on-site. This decontamination is done by using high pressure spray as the water level is lowered. Cover pool with steel plate on steel framework and provide a High Efficiency Particulate Air (HEPA) filter unit.
- Install security and surveillance monitoring equipment and relocate security fence around secured structures as required.
- Nonradioactive structures, located outside the secured area, may be demolished. However, this study assumes that demolition would be delayed until after license termination.
- Prepare final decommissioning program report for submittal to NRC.

3.2.2 Period 2: SAFSTOR Dormancy

Activities required during the planned dormancy period, for the SAFSTOR alternative, include a 24 hour security force, preventive and corrective maintenance on security systems, area lighting, general building maintenance, heating and ventilation of buildings, routine radiological inspections of contaminated buildings, maintenance of structural integrity, and an environmental and radiation monitoring program.

Maintenance and equipment inspection activities are provided by a utility maintenance staff. Their duty is to maintain the structures in a safe condition, provide adequate lighting, ventilation, and heating, and perform periodic preventative maintenance on essential equipment.

An environmental surveillance program is carried out during the dormancy period to ensure that releases of radioactivity to the environment are controlled. Such releases are identified and quantified. Appropriate emergency procedures are established and initiated for releases that exceed prescribed limits. The environmental surveillance program will generally be a modified/abbreviated version of that carried on during normal plant operations.

Security during the dormancy period is conducted primarily to prevent unauthorized entry and to protect the public from the consequences of their own actions. Security detection and notification systems used during plant operations are augmented by the installation of audible alarms. Since contaminated areas and equipment can conceivably be reached by the breach of only a door or window, a full time security force is maintained on site throughout the SAFSTOR dormancy. Additionally, silent alarms may be installed to alert off-site security personnel to trespass and fire. Liaison with local law enforcement agencies is maintained and their assistance requested as necessary.

Primary physical security is provided by the security fence which must be maintained in good condition for the duration of this period. The facility will also be secured by high security locks on exterior doors and intrusion alarms. Fire and radiation alarms will be monitored continuously by site personnel.

In addition, until the fuel assemblies are removed from the site, additional security will be maintained on-site.

3.2.3 Periods 3-5: SAFSTOR Delayed Removal/Dismantling

At the end of the dormancy period for the SAFSTOR alternative, the remaining structures are completely dismantled. Basically, the same dismantling operations as those described for the DECON alternative will be performed. SAFSTOR Period 3 activities would correspond to the DECON Period 1 Planning Phase, Period 4 to the Period 2 Decommissioning Operations Phase, and Period 5 to the Period 3 Site Restoration Phase. Section 3.1 of this report delineates the activities associated with each of these phases of the decommissioning process. Because this alternative provides a period of decay of the residual radioactivity, lower personnel radiation exposures are incurred than with the DECON alternative. Many of the dismantling activities may employ manual techniques rather than remote procedures. Thus, dismantling operations can be simplified.

Although the initial radiation levels due to Cobalt-60 (Co60) will decrease during the dormancy period, the internal components of the reactor vessel will still have sufficiently high radiation dose rates to require remote sectioning under water due to the presence of long-lived radionuclides such as Niobium-94 (Nb94) and Nickel-59 (Ni59). Therefore, the dismantling procedures described for the DECON alternative would be employed. Portions of the concrete shield will still be radioactive because of the presence of activated trace elements with long half-lives and will require controlled removal, packaging, and burial procedures. It is unlikely that radioactive corrosion products on inner surfaces of piping and components will have decayed to levels that will permit unrestricted use or allow conventional removal. These systems and components are surveyed as they are removed with disposition dependent upon the existing release criteria. No systems in this study designated as contaminated in the DECON alternative are assumed to be releasable after the dormancy; these are removed and disposed of as contaminated material.

Following notification by YAEC of completion of the decontamination and disposal of components and materials from the facility, the NRC regional staff

conducts an on-site survey to verify that the acceptable activity and contamination levels are satisfied. When the requirements are satisfied, the NRC can terminate the Part 50 license for the main facility. The site will retain a Part 72 license for the storage of spent nuclear fuel until such time that DOE is able to take receipt. At that time, the storage facility will be decontaminated, if required, and the structure dismantled. A final survey would allow the NRC to terminate the Part 72 license, ending its jurisdiction over the YNPS facility.

Site restoration activities can be performed once the Part 50 license has been terminated. The site is graded and landscaped as required. A final decommissioning program completion report is then prepared.

4. COST ESTIMATE

Site-specific cost estimates were prepared for YNPS to account for the unique features of the nuclear steam supply system, electric power generation systems, site buildings and structures. The basis for the estimates, including the source of information, methodolog/, assumptions and total costs, is described in this section.

4.1 BASIS OF ESTIMATES

The site-specific cost estimates were developed using YNPS drawings and the inventory documents provided by YAEC. These drawings and documents were used to determine the general arrangement of the facility and to determine estimates of building concrete volumes, steel quantities, numbers and size of components, and land area of the site restored.

The decommissioning effort is a labor-intensive program. Representative labor rates for each geographical region and each craft or salaried worker are essential for the development of a meaningful site-specific decommissioning cost estimate. YAEC provided typical craft labor rates and salary data for utility personnel from recent labor contracts and records for typical craft personnel and salaried workers.

Rates for shipping radioactive wastes were provided by Tri-State Motor Transit published tariffs for this cargo. Transportation costs have escalated in the past few years and recent rates must be used for accurate site-specific cost estimates.

Disposition of radioactive wastes is a major contributor to the cost of decommissioning. The availability of burial sites is of national concern, with regional compacts being formed to provide adequate burial space for operating and planned reactors. In this study, an unspecified burial facility is assumed at a distance of 1,000 miles from the site. The basis for the estimate for low-level radioactive waste disposal relied upon a projection prepared by YAEC (Ref. 4) for volumetric unit costs. Package surcharges, e.g., on total curies, weight, special handling requirements, etc., were derived from information provided on the Chem-Nuclear facility at Barnwell, South Carolina (Ref. 5).

Assumptions and Key Inputs

- 1. YNPS drawings, equipment and structural specifications, including construction details, were provided by YAEC.
- Employee salary and craft labor rates for site administration, operations, construction and maintenance personnel were provided by YAEC for required positions and functions.

- Engineering services for such items as writing activity specifications, detailed procedures, detailed activation analyses, structural modifications, etc. are assumed to be provided by YAEC acting as its own Decommissioning Operations Contractor (DOC).
- 4. Material and equipment costs for conventional demolition and/or construction activities are taken from R.S. Means Construction Cost Data (Ref. 6).
- 5. Rates for shipping radioactive wastes were provided by Tri-State Motor Transit in published tariffs for this cargo (Ref. 7).
- 6. The basis for the estimate for low-level radioactive waste disposal relied upon a projection prepared by YAEC for volumetric unit costs. Package surcharges, e.g., on total curies, weight, special handling requirements, etc., were derived from information provided on the Chem-Nuclear facility at Barnwell, South Carolina (Ref. 5).
- 7. All costs in this estimate are in 1992 dollars. These estimates exclude interest and escalation.
- Site property taxes were provided by YAEC for inclusion in the total decommissioning cost.
- 9. These studies do address the removal but not the disposal of spent fuel from the site. The cost for disposal is assumed to be covered the 1 mill/kwhr surcharge the owners of YNPS are paying to DOE. These studies do consider the constraints that the presence of spent fuel on site may impose on other decommissioning activities due to the unavailability of DOE to accept the spent fuel inventory stored on site in a timely fashion. Consequently, the spent fuel will be stored in dry spent fuel storage casks for a period of up to twenty-three years before being transferred to DOE for ultimate disposition.
- 10. YAEC is projected to purchase 30 dry storage casks to support post operational spent fuel storage. All spent fuel remaining on site will be transferred to dry cask storage no later than June 30, 1996. This will allow decommissioning activities to proceed unhindered by spent fuel pool operational requirements and/or restrictions.
- 11. The YAEC staffing requirements during decommissioning vary with the level of activity on-site.
- 12. These studies follow the principles of ALARA through the use of work duration adjustment factors which incorporate such items as radiological protection instruction, mock-up training, the use of respiratory protection and personnel protective clothing. These items lengthen a task's duration, which

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increases the costs and lengthens the schedule. Costs are reported in the engineering and planning, for activity specifications and detailed procedures, to include ALARA considerations.

13. These studies are performed in accordance with the published study from the Atomic Industrial Forum/National Environmental Studies Project report AIF/NESP-036, "Guidelines for Producing Commercial Nuclear Power Plant Decommissioning Cost Estimates" (Ref. 8). The contents of these guidelines were prepared under the review of a task force consisting of representatives from utilities, state regulatory commissions, architect/engineering firms, the Federal Energy Regulatory Commission, the Nuclear Regulatory Commission, and the National Association of Regulatory Utility Commissioners.

4.2 METHODOLOGY

The methodology used to develop the cost estimates follow the basic approach originally presented in the AIF/NESP-009 study report, "An Engineering Evaluation of Nuclear Power Reactor Decommissioning Alternatives" (Ref. 9) and the U.S. DOE "Decommissioning Handbook" (Ref. 10). These references utilize a unit cost factor method for estimating decommissioning activity costs to simplify the estimating calculations. Unit cost factors for concrete removal (\$/cubic yard), steel removal (\$/ton), and cutting costs (\$/in) were developed from the labor and material cost information provided by YAEC. With the item quantity (cubic yards, tons, inches, etc.) developed from plant drawings and inventory documents, the activity-dependent costs are estimated.

The activity duration critical path was used to determine the total decommissioning program schedule. The program schedule is used to determine the period-dependent costs for program management, administration, field engineering, equipment rental, quality assurance and security. YAEC provided typical salary and hourly rates for personnel associated with period-dependent costs. The costs for conventional demolition of nonradioactive structures, materials, backfill, landscaping and equipment rental were obtained from the "Building Construction Cost Data" published by R. S. Means (Ref. 6). Examples of unit cost factor development are presented in the AIF "Guidelines" study (Ref. 8), one of which is reproduced in Appendix A. Appendix B lists the specific factors developed for the YNPS analysis.

The activity- and period-dependent costs are summed to develop the total decommissioning costs. A contingency is then applied as described below. "Contingencies" are defined in the American Association of Cost Engineers' Cost Engineers' Notebook (Ref. 11) as "specific provision for unforeseeable elements of cost within the defined project scope; particularly important where previous experience relating estimates and actual costs has shown that unforeseeable events which will increase costs are likely to occur." The cost elements in these estimates are based upon ideal conditions, therefore a contingency factor has been applied.

As with any major project, items which could occur that have not been accounted for in this estimate are changes in the regulatory requirements, the effects of craft labor strikes, bad weather halting or slowing down waste shipments to the burial ground, equipment/tool breakage, changes in the anticipated plant shutdown conditions, etc. In the AIF/NES-P-036 study, "Guidelines for Producing Commercial Nuclear Power Plant Decommissioning Cost Estimates" (Ref. 8), the types of unforeseeable events that are likely to occur in decommissioning are discussed and guidelines are provided for percentage contingency in each category. Application of these types of contingencies, on a line item basis, yielded a weighted average contingency of 19.05% for the DECON alternative and 18.39% for the SAFSTOR alternative.

The unit cost factor method provides a demonstrable basis for establishing reliable cost estimates. The detail of activities provided in the unit cost factors for activity time labor costs (by craft), and equipment and consumables costs provide assurance that cost elements have not been omitted. These detailed unit cost factors coupled with the plant-specific inventory of piping, components and structures provide a high degree of confidence in the reliability of the cost estimates.

The studies were prepared with consideration of any reasonable practices or procedures which would reduce the ultimate cost of decommissioning. For example, the projection of radioactive waste volume has decreased significantly from earlier forecasts. This savings was achieved by reassessing the decontamination of the YNPS inventory considering current technology and regulations.

4.3 SITE-SPECIFIC CONSIDERATIONS

There are a number of site-specific considerations that affect the method for dismantling and removal of equipment from the site and the degree of restoration required. The cost impact of these considerations identified herein are included in these cost studies.

4.3.1 Major Component Removal

The reactor pressure vessel (shell and nozzle zone) and reactor internal components are segmented for disposal and shipped in shielded casks. The process is described in Section. 3.1.2.

Segmentation and packaging of the internals packages is performed in the RC/RCA where a turntable and remote cutter will be installed. The vessel is segmented in-place using a mast-mounted cutter supported off the lower head and directed from a shielded work platform installed overhead in the reactor cavity. Shipping cask specifications and U.S. Department of Transportation (US DOT) regulations will dictate segmentation and packaging methodology; all packages designated meet current physical and radiological limitations and regulations. All cask shipments are made in US DOT

approved, currently available, truck casks. Both the closure head and the reactor vessel lower head are disposed of intact. These components are modified for shipment as their own containers and shipped to the burial site along with the steam generators, reactor coolant pumps and pressurizer.

Reactor coolant piping is cut from the reactor vessel once the water level in the vessel (used for personnel shielding during dismantling and cutting operations in and around the vessel) is dropped below the nozzle zone. The piping is boxed and shipped by shielded van. The reactor coolant pumps, motors and the pressurizer are lifted out intact, packaged and transported along with the steam generators.

The steam generators are extracted from the Vapor Container and moved to a temporary staging area on-site. The generators are then moved off-site by an overland transport to an operating rail siding. The generators are then moved by a dedicated train to the burial site.

The main turbine is dismantled using conventional maintenance procedures; the turbine rotors and shafts are removed to a clean laydown area for disposal. The lower turbine casings are removed from their anchors by controlled demolition. The main condensers are segmented and transported to the laydown area for disposal as scrap along with the lower turbine casings.

4.3.2 Transportation Methods

For the purposes of cost estimation, it was assumed that the NSSS components are transported by overland transporter to an area serviced by rail for shipment to the burial facility. These payloads include the reactor vessel head packages, reactor coolant pumps, the steam generators and the pressurizer unit. In this study, it is assumed that the steam generator units are removed sequentially and stored on-site in a temporary staging area. The generators are then trucked to a rail siding and loaded onto a heavy duty flatcar. This car is moved by a dedicated train containing the steam generators and other components such as the reactor coolant pumps, upper and lower vessel head packages, and the pressurizer. At the burial facility the generators are off-loaded to an overland transporter for the remaining distance to the burial site.

4.3.3 Site Conditions at Facility Closeout

It is assumed that the site is restored by regrading to conform to the adjacent landscape. Sufficient topsoil is to be placed to permit new growth of native vegetation.

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4.4 ASSUMPTIONS

The following are the major assumptions made in the development of the cost estimates for the YNPS.

- 1. YAEC will serve as its own Decommissioning Operations Contractor (DOC) in the decommissioning of the YNPS. The Decommissioning Operations Contractor (DOC) provides sufficient staff to perform the preparatory demolition planning and scheduling, and manage the demolition efforts. Site security during demolition is provided by YAEC's subcontractor. The demolition work is performed by YAEC acting as the DOC or a demolition subcontractor who will provide adequate staff, labor, equipment, materials and overhead to complete the demolition.
- An unspecified burial facility was assumed to exist within 1,000 miles of the site. This location was taken as the final destination for all radioactive waste shipments from the YNPS. The basis for the estimate for low-level radioactive waste disposal relied upon a projection prepared by YAEC (Ref. 4) for volumetric unit costs. Package surcharges, e.g., on total curies, weight, special handling requirements, etc., were derived from information provided on the Chem-Nuclear facility at Barnwell, South Carolina and projections of disposal costs provided by YAEC.

Disposal costs were calculated using actual component dimensions for those components not requiring additional packaging, e.g., the NSSS components.

3. The decommissioning activities are performed in accordance with the following regulatory documents:

10 CFR 20	Standards for Protection Against Radiation
10 CFR 30	Rules of General Applicability to Licensing of Byproduct
	Materials
10 CFR 40	Licensing of Source Material
10 CFR 50	Domestic Licensing of Production and Utilization
	Facilities
10 CFR 51	Licensing and Regulatory Policy and Procedures for
	Environmental Protection
10 CFR 61	Licensing Requirements for Land Disposal of
	Radioactive Wastes
10 CFR 170	Fees for Facilities and Material Licenses and Other
	Regulatory Services
29 CFR 1910	Occupational Safety and Health Standards
49 CFR 170-178	Department of Transportation Regulations Governing
	the Transport of Hazardous Materials

All environmental regulations in force in 1992 are in force during decommissioning effort.

- 4. Nuclear liability insurance provides coverage for damages or injuries due to radiation exposure from equipment, material, etc. used during decommissioning. Nuclear liability insurance is phased out upon final decontamination of the site. Nuclear liability as well as property insurance premiums were provided by YAEC.
- 5. The NSSS (reactor vessel and reactor coolant system) is chemically decontaminated using one chemical flush and two water rinses prior to segmentation. Typically, a decontamination factor (DF) of 10 is expected (Ref. 12).
- 6. Reactor vessel and internals packages conditions:

Any cladding failure that has or may occur during the lifetime of the plant is assumed:

1) to have released fission products at sufficiently low levels that the buildup of quantities of long-lived isotopes (e.g. cesium-137 or strontium-90) has been prevented in reaching levels exceeding those which permit the major NSSS components to be shipped as LSA waste and burial within the requirements of 10 CFR 61 or the regional burial ground, or

2) to have necessitated systematic decontamination during the operating life of the plant and therefore the levels again are at acceptable levels for transport as LSA waste and burial within the requirements of 10 CFR 61.

The curie contents of the vessel and internals at final shutdown (October 1, 1991) are derived from a preliminary activation analysis performed by TLG on the reactor core shroud; this study was based in part upon a detailed neutron flux calculation performed for YAEC by Westinghouse Electric Company. The balance of the reactor internals, vessel and neutron shield tank inner wall were taken from an analysis performed by YAEC, which in turn was based upon the NUREG/CR-0130 PWR study (Ref. 12).

7. The disposal costs for the reactor vessel (beltline and nozzle regions) and the internals packages are based on remote segmentation in-place, packaging in casks with shielding, and shipping by truck to the burial ground. A maximum normal road weight limit of 80,000 pounds is assumed for all truck shipments including cask shipments. This included vessel segment(s), supplementary shielding, cask tie-downs and tractor trailer. The maximum curies per shipment assumed permissible are based on the allowable license limits and practical curie limitations of available shielded shipping casks. The number

and curie content of vessel segments are selected to meet these limits. The upper and lower reactor vessel heads are shipped by rail along with the steam generators.

- 8. Overland transport costs for the steam generators are based on discussions with Reliance Trucking of Phoenix, AZ. Reliance has handled the overland transport and installation of NSSS components for several plants.
- 9. Steam generators are removed sequentially and stored on site until ready to be moved. This scenario will consolidate shipping and reduce mobilization costs for the heavy haul vehicles and specialty rail cars. The steam generators will be trucked to the nearest active rail siding.
- 10. Plant conditions & construction:

Insulation materials used throughout the station containing asbestos are remediated and disposed of in accordance with current disposal regulations.

Transformers and capacitors certified to have PCB oil are drained and/or incinerated according to the amount and type of PCB contained within them.

Lead is also removed and disposed of in accordance with current regulations regarding its treatment.

- 11. YNPS is isolated electrically from the rest of the transmission system and completely decommissioned (i.e., the station will be out of service prior to commencing the demolition effort).
- YNPS owners provide for the electrical power required to demolish the station.
- 13. Scrap generated during decommissioning is not included as a salvage credit line item in these studies for two reasons: (1) the scrap value merely offsets the associated site removal and scrap reprocessing costs, and (2) a relatively low value of scrap exists in the market. Scrap processing and site removal costs are not included in the estimate.
- 14. YAEC removes all items of furniture, tools, mobile equipment such as forklifts, trucks, bulldozers, other similar mobile equipment and other such items of personal property owned by YAEC that is easily removed without the use of special equipment. That is, the cost for removal of such non-affixed items is not included in this decommissioning cost estimate.
- 15. A future YAEC project team assigned to the decommissioning effort will investigate the economics of reusable construction materials.

- 16. Existing warehouses will remain for use by subcontractors, as well as YAEC. The warehouses will be dismantled as they are no longer needed to support the decommissioning program.
- 17. All contaminated piping, components and structures other than the reactor vessel and internals are assumed to meet DOT limits for LSA material.
- 18. Fuel oil tanks will be emptied. Tanks are cleaned by flushing or steam cleaning as required prior to disposal. Acid and caustic tanks are emptied through normal usage. Lubricating and transformer oils are drained and removed from site by a waste disposal vendor.
- 19. Water drain holes are drilled in the bottom of all sub grade structures to be abandoned. Piping and electrical manholes are backfilled with a suitable earthen material and abandoned. Vertical pump structures and sumps are backfilled with a suitable earthen material and abandoned. The site grade will be adjusted such that removal of foundations at grade will not be required.
- 20. Non-contaminated underground piping (except the intake, discharge, and circulating water piping) will be abandoned without special considerations. The plant intake and discharge circulating water piping will be capped and filled to eliminate the potential for collapse after the site is released for unrestricted access.
- 21. The station grounds are planted with vegetable matter for erosion control and will have a final contour consistent with adjacent surroundings. Culverts, head walls and rip-rap remain in place to allow natural drainage.
- 22. The switchyard is left intact for use by the balance of the utility's electrical distribution system. Transmission towers remain in place.
- 23. The perimeter fence is moved as appropriate to conform with the technical specifications in force at the various stages in the project. All road and parking area base material remains in place. Road and parking areas with asphalt surfacing or concrete are broken up and the area covered with fill. All gravel road and parking areas remain in place and the area covered with fill.
- 24. These studies estimate that there will be some radioactive waste generated which is greater than 10 CFR 61 Class C quantities, resulting from disposal of the highly activated sections of the reactor vessel internals. This waste will most likely be disposed of as High Level Waste in the DOE's deep geological repository unless an alternative solution is approved by the NRC. The cost

of disposal, unlike that for the spent fuel, is not covered by DOE's 1 mill/kWhr surcharge, and has been estimated as being highly radioactive Type C waste.

4.5 COST ESTIMATE SUMMARY

A summary of the decommissioning alternative costs with annual expenditures is provided in Tables 4.1a and 4.1b. Tables 4.2 and 4.3 show the detailed listing and costs of major activities for the decommissioning scenarios.

On Tables 4.2 and 4.3, "Decon", for example, refers to decontamination costs, and "Total" is the sum of Decon, Remove, Pack, Ship and Bury as well as other miscellaneous items not listed (such as insurance, property taxes, plant energy budget, and ISFSI transfer costs). All costs are reported out in 1992 (January) dollars. The scrap amount values are in standard tons.

TABLE 4.1a SUMMARY OF DECOMMISSIONING COSTS (Thousands of Dollars)									
Alternative	Period	Calendar Years	1992 Cost 1000s \$						
DECON (Prompt Removal/Dis	mantling)								
Preparations	1	1995	32,256.7						
Subtotal Period	1	1996	<u>15.875.8</u> 48,132.5						
Decommissioning	2	1996 1997	37,464.8 73,774.2						
Subtotal Period	2	1998	<u>9.146.0</u> 120,385.0						
Site Restoration	3	1998	12,144.3						
Subtotal Period	3	1999	<u>912.5</u> 13,056.7						
Post Period 3 - On-Site Fuel Storage		1999-2018	50,560.6						
Total Cost			\$232,134.4						

All costs reported in 1992 dollars; costs may not add due to rounding

SUMMARY	TABLE 4.1b MMARY OF DECOMMISSIONING COSTS (Thousands of Dollars)						
Alternative	Period	Calendar Years	1992 Cost 1000s 5				
SAFSTOR (Mothball/Delayed	Dismantling)						
Mothball Operations	1	1995	20,944.9 10.308.4				
Subtotal Period	1	1996	31,253.3				
Dormancy		1996-99	16,093.2				
Preparations	3	1999	22,404.1 11,746.2				
Subtotal Period	3	2000	34,150.2				
Decommissioning Activities	4	2000 2001 2002	35,427.6 72,923.7 3.657.8				
Subtotal Period	4	2002 C	112,009.0				
Site Restoration	5	2002	13,005.6				
Post Period 5 - On-site Fuel Storage		2002-2018	40,605.7				
Total Cost			\$247,117.1				

All costs reported in 1992 dollars; costs may not add due to rounding

TABLE # 2 COST ESTIMATE FOR PROMET DISMATLING YAPS W/475 YIS DECRY (1952 COST, 100CS UNLES)

ND.	activity	Decon		Package		BUTY	Total C	e ves ce	WIT I NORMEY	s s	Costs
******	*******	*******	*****	****	ana ana ana ana						******
	*** PERIOD 5 ***										
1.1	Resolve fuel & sources						n/a				
	Decon plant & process maste										
	tevies plant dwes & specs									8	
	Perfora detailed rad survey										
	fitimete by-product inventory									e	
	Subalt license amendment										
	End product description										
	Detailed by-product inventory									0	
	Define wajor work sequence									6	
	Perform salety analysis										
	Submit dismontling plan									0	
	Receive license amendment									ē	
13 1	Receive dismantling order						8			0	
1.2.4	activity specs										
	Plant & temporary facilities										
	Plant systems									0	
	Reactor Internals									6	
	Reactor versel									0	
	elological Shleie										
	molisture seperators/reheaters									0	
	Stean Generators										
	Reinforced concrete									0	
	furbine & condenser									ö	
	Plant structures i wulldings									ő	
14.17.1	Raste management									0	
14.12 1	Facility & site closeout									0	
15 1	Frep dismantiling seg										
78 7	Plant pres. & Temp swces						5418 I		13 00%	312	163
17 1	Design wir cleanup sys						8			0	
	R BB MB/CCE/tools/ttc						1197		15.00%	180	13
19 1	Procure cashs/limers						3			0	
	Detailed work procedures										
20.1	Plant systems										
	Wessel Acad									0	
	Reactor internels									3	
	teasining buildings									0	
	ORD cooling assembly ORD Housing a ICI Tubes									0	
	Incore Instrumentation						2			5	
	Reactor vessel						*				
	facility closeout									0	
	missife shields										
	Biological shield									8	
	molsture seperators & reheaters									9	
	Steam Benerators									0	
	Reinforced concrete										
	Turbine & condensers						8			0	
	Auxiliary buliding						e s			0	
	Reactor building						8			ö	
	Decon equipment	180	6				180		25.00%	45	2
	Decon supplies	54					59		25 00%	17	
	Process figuid weste	25		13	19	83	140		26 718	37	
	Auclear Insurance						670		13 00%	100	71
	Mon-muclear Insurance						349		15.00%	37	21
	Property Ences						685		15 00%	103	71
	Mealth physics sup		47.9				478		25 00%	120	3
	Heavy Equipment Bental		207				207		15.00%	31	2
	Dispose polid weste		2.47	5	2	227	234	22	24.59%	58	21
	15FSI Capital Expenditures					221	11490	**	15 00%	1724	152
	Plant energy budget						1086		15 00%	163	12
	tegulatory fees						430		8 00%		4
	WC fees										

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TABLE & 3 COST ESTIMATE FOR PROMPT DISMANTLINC. VMPS Wid 75 Vrs Decay (1992 Cost, 10005 units)

NC.	activity	Decon		Pactage		BUTY	Totai	Cu hes	Contingency	patingenc S	Costs
					Constant Sea Sea						
	OFT-BINE UTIFLEY Staff						9013		15 00%	1352	10365
	On-Site Utility Staff						114乐召		13 00%	87.14	13182
	ISPSI Transfer Costs						432		15 00%	45	497
	15751 Licensing Fees						18.8		0.00%		188
	ABC Overhead Expenses						1920		15 00%	298	2204
	*** PERICO 2 ***										
							2				
21	Decon primery imp Rev spent fuel racks	124		41	13	3190	124	314	50.005	6.7 704.2	188
	Nuclear Steam Supply System Removal										
1.1	***************************************										
	Reactor Coelant Pibling	28		27	14	1135	126.8	104		317	1577
23 3	Pressurizer Reilet Tank	8		3	3	135	19.5	1.3	25.66%	6.8	240
23 3	Reactor Coolant Pumps & motors	10		34	18	381	484	.95		117	6.010
	Pressul Izer	5		4	6	383	#.73	36		108	3.47
23.3	Steam Generators	37		136	2496	44.17	\$744	#23	27.38%	2180	11025
23.6	CROMS/ ICIS/Service Structure Removal	28				324	385	30		195	3#5
	Bracior Wessel Internals	51		194	335	4067	\$ 30 1	87		3322	96.23
23.8	Reactor Wessel	86	3026	.278	\$48	2889	6927	190		3989	10916
	Hazardous asterial Beapval and Disposal										
74 4	sibelioi removal program		1134	294	5	2185	36.18		25 025	804	4522
	PCE Removel		17	374		2.025	17		25.00%	404	4523
	Disposal of Plant Systems										
	administration Building ventilation - TB						4		15 00%		
	Als Disposal (Filtered Exhaust) - PAB		19	97	6	678	722	56	24.26%	\$75	897
25.3	alt Disposal (filtered Exhaust) - SFP		3	8	0	50	13		22 53%	3	24
25.4	Alf Disposal (filtered (shoust) - 908		0	0	0		2				3
25.8	Alt REMOVEL - TB		7				7		15.00%		
25.6	Auxiliary Sleam - 98		8						15.00%	1	
25.7	Battery Roes No. 3 Ventilation - PAS								15 00%	1	52
23.8	battery Booms 1 & 2 Ventilation - PAS		3				3		15 00%		
25.8	Boller Feed - 70		76				76		15.00%	8.5	
25.10	Boller Feed - WC	13	33	3	2	217	270	31		66	236
	Boiles Feed - YARD								15 00%	1	10
25.12	Charging & Wolume Control - PLS	37	50	27	7	970	1046	90		261	1306
29.13	Charging & Volume Control - VC	5-6	45	17	5	784	9.17	76		230	1134
25.14	Chemical Feed - TB						1		15.00%		
25.15	Chemical Shuldow: - PIB		13	13	2	190	218	18	23 405	51	26.5
	Circulating Water - 38						0.2		15 00%		
25.17	Circulatine mater - SH		10				10		15 0.0%		11
			3				3		15 00%		
	Circulating mater - vanb		22				22		15.00%		25
	Component Cooling Mater - PAS		31				31		15 00%		35
	Component Cooling Nater - WC	32		16	6	6.27	764	61		185	998
	Component Cooling Hater - HOE					411	3		15 00%	192	109
	Component Cooling Bater - "AED		2				2		15 00%		
	Compressed alz - Pat										
25.25	Chapressed Air - SE		3				3		15 00%		
	Compressed Alt - SPP		1						15 00%		
	Compressed Air - SPP						0.04		15 00%	8	0
	Compressed Air - 3ras						8.2		15.00%		
			15				15		15.00%	2	3
	Compressed alt - WC		3		9	18	23	2		5	.26
	Compressed alr - 908								15 00%	0	2
	Compressed Als - VARD		2				3		15 00%	0	2
25.33	Control Boos Ventilation - TB		2						15 00%	1	
	Corresion Control - Pas						1		15.00%		
	Debineralized weter - PAB		5				5		15 00%		
	Demineralized Weter - 78		7				7		15.00%		
	Desimeralized water - WOS		2				2		15.00%		3
	Demineratized mater - yakb								15 00%	¥	
	EDC Building Ventilation - DCB		*						15 308	0	
	Emergency doller Feed - Pag		7				÷		15.00%	1	
25.38	Exergency Boller Feed - 98								15.00%		
25.40						20	25				31
25.40	Emergency Boller Feed - WC										
25.40	Emergency Buller Feed - 16 Familier Feed - 1650	2	2			20			and the second sec		
25.40 25.41 25.42	Exergency Boller Feed - 90 Exergency Boller Feed - 958D Exergency Power - DCB	2	1	1.1		20	1		15 00% 15 00%	8	1

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TLG HF 205 (6/82)

ND.	activity	Decon	Resource	Pactage	Ship	Bury			Contingency		Costa
25.45	Emergency Power - SDCB		2				2		15 80%	9	
25 48	Emergency Power - SFSS						9		15.00%	0	
	Emergency Power - 3558		3				3		15 50%	e	
	faeigency Power - TB		12				12		15 00%	2	93
	Estraction Steam + TB		19				19		15 00%	3	33
25.50	Fire Detection & Suppression - DOB		2				2		15 00%	0	
25 51	Fire Detection & Suppression - FPH						0 5		13 00%	0	
	fire Detection & Suppression - Fap								15 00%	8	
	Fire Detection & Suppression - PAB								15 00%		
23.54	fire Detection & Suppression - SH		7				7		15 00%	1	
	Fire Detection & Suppression - \$7-8		5				3		15 DOB	1	
	Fire Detection & Suppression - TB		92				*2		15.00%	3.6	10
23 37	fire Detection & Suppression - WARD		49				49		15.00%	7	3
	Floor Drainage - PAB Fuel Dit - DGB		0				0.2		15 00%	0	
	Fuel Dit - FOTH								15 00%	0	
	fuel Oit - Th								15 00%		
									15 00%	0	
	Fuel OII - VARD Cas Storage Spom Wentilation - PAB		1						15.00%		
	Ceneration & Transmission - Sevo								15.00%	0	
	Generation & Transmission - 3875								15 00%	0	
	Generation & Transmission - VARD								15 80%		
	Cenerator Cas - 78		3				2		15.00%	8	
	Generator Seal Oli - 78		-				3		15 00%	8	
	Cland Seal - TB								13 80%		
	Heater Drains - TB		24				24		15 00%	4	
	Heating Steam/Condensate - DCB		3				3		15 00% 15 00%		
	Heating Steam/Condensate - f1CS								15 00%		
	Meating Steam/Condensate - Fap								15 00%	0	
	Heating Steam/Condensate - He w								15 00%	ő	
	Heating Steam/Condensale - Pab		26				29		15.00%	4	3
	Heating Steam/Condensate - PCAI		0				0.2		15.00%	ē	
	Heating Steam/Condensate - Pvs						W. 2		15.02%		
25.78	Heating Steam/Condensate - SB						0.5		15 00%	ē	
	Heating Steam/Condensate - 5558		5				2		15 80%		
	Heating Steam/Condensate - ST-8								15 60%		
	Heating Steam/Condensate - TB		48				48		15.00%		3
	Heating Steam/Condensate - WC		97	28	53	1154	1321	5 8 5		315	15.3
	Heating Steam/Condensate - wem		10				10		15 00%	2	
	Meating Steam/Condensate - YARD		7				7		15 00%	ĩ	
25.85	Lubricating Dil - 78		12				12		15.00%	2	1.1
25.24	asin Steam - SPSS								15 00%		
	Main Steam - TB		20				28		15 00%		3
25.88	main Steam - WC	41	21	12	4	578	663	54	26.218	17.4	83
25.89	Rein Steam - YAED		20				20		15.00%	3	
25.90	aisc Reacter Coolant - WC		4	3		105	112		24.19%	37	54
	way enclosure ventilation - YARD		0				0.3		15 00%	0	
	Hitropen - PAB		1				1		15 00%		
	Nitrogen - YARD		2				2		15 00%	0	
	PAS Non-filtered Wentilstion - PAS		2				2		15 00%	0	
	Pressure Control & Relief - WC	- 55	26	- A	2	281	232		25 60%	8.5	24
	Purification - 12P		1 13	3		117	142		25 20%	36	87
	Purification - PAS		30	10	4	401	644		23.418	106	95
	Radioective Heste Disposal - 189		5			49	34		5 23 62%	13	
	Redicactive maste Disposal - PAS		30		3	35 9	#01	3	6 23.85%	05	41
	B Redinactive Small - Th								15.00%	8	
	t Radioactive maste Disposal - WC		3	0		8	9		1 23.88%	2	
25 103	Radinactive Raste Disposal - NES		237	44	15	1795	2091	17	5 23.48%	491	254
25.103	adieactive meste Disposal - varb		2				2		13 00%	0	
	i safe shutemen - 5558		4				4		15 00%		
	5 Sale Shuldown - YARD								15.00%		
25.909	a Safe Shutdown Building Wentilation - 555						1		15 90%		
25.107	f Safety Injection - DOB		91	20	5	800	915	71	23.63%	216	113
	8 Safety Injection - Pag		23		2	285	318	21	23 87%	78	31
	safety injection - Sis		28		2		266	2		83	32
	Safety Injection - VC		17	4		170	201	1	24 86%	90	25
	safety injection - vitb		20				20		15.00%	3	1
	I Safety Injection Bidg Ventilation - PAB		2				2		13.00%	e	
	3 Somple - Pag		27	6	2	235	268	2		63	
	Sample - WC		22		2		216	1		51	21
	Sanitary Dispess? - YARD						1		15 00%		
25.116	Screenwalth - SH		3						15.00%		
25.117	Screennell Pumphouse Wantilation - 30		1						15.80%		
35,110	I Service Auliding Wentilation - 30		4						15 805		
	Service Hoter - PAB		19								

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lvi£y ND Activi£y	Decon	tesove	Pactage	ship	Βυτγ	To tal		Contingency	pntingenc S	Costs
23 120 Service Mater - SH		12				12		15 00%	2	1
25.121 Service mater - Th		35				35		15 00%		4
25 122 Service Water - WC		73	25		1070	1125	99		269	139
25.123 Service maler - YARD	and the second second	18			744	18	72	15 00%	223	150
25 124 Shutdown Cooling - Pag	49	89 14	18		104	127	10		31	15
25 135 Shutdown Cooling - VC			ő	0	2	3	0			
25.126 Speni Fuel - IIP 25.127 Speni Fuel - PAB		87	5	2	245	27.1	74		45	33
33 126 Spent Fuel - SFP		15	3		126	146	12		34	18
25 125 Spent Fucl - VC				0	5	5	0	23 995		
23.130 Spent Fuel - YARD		0				0.3		15 00%	0	
25 131 Station Service - DE #		0				8.1		13 00%	0	
15 137 Station Service - DOR		e				0.4		15 008		
15.133 Station Service - PmP		0				C 2		15 00%	e	
23.134 Station Service - On		39				39		15 00%		
25.135 Station Service - mT		*						15 00%	1	
25 136 Station Service - NEUPS		0				0 3		15 00%	0	
23.137 Station Service - PAB		127				167		15.00%	25	
25.136 Station Service - Pag (Contag)		135	5.6	5	636	790	62	22 96%	181	
25 139 Station Service - P6						8.1		15 00%	ē	
23 140 Station Service - PCA1						0.1		15 00%		
15.141 Station Service - PCA2 15.142 Station Service - PMCB		0				C 2		15 60%		
15. 143 Station Service - PVS		é				0.1		15.00%		
15. 144 Station Service - St		2				2		15 00%	0	
15. 143 Station Service - SDCB		6				0.4		15 00%	0	
3.144 Station Service - SPP						0.2		15 50%		
15.147 Station Service - SH		7				7		15 00%		
15 tes Station Service - SLED						0 7		15.00%	8	
15.149 Station Service - 37W		9				6.1		15.00%		
15 190 Station Service - SavD						P. 1		15 90%		
5.151 Station Service - Tab		8				3		15.00%		
5.152 Station Service - TB		188				1巻茶		15 00%		2
15.153 Station Service - VC		352	36	12	1583.	1963	154			24
25.154 Station Service - WOB		2.8			1.	11		15.00%		
75.155 Station Service - 1908 (Contam)		70	7	3	331	415	32	22.968		
25.154 Station Service - MH		1				0.1		15.00%		
25 157 Station Service - 8PH		97				52		15 00%		
15.158 Station Service - YaRD 15.155 Turbine Building Wentilation - TB		10				10		15 008		
25.160 VC MEALING/Choling/Air Becirc - VC		123	\$7	18	27.15	2973	26.5			36
25. 161 Wents & Drains - PAB		97	26	9	1052	1103	103	23.78%	291	14
25.162 Wents & Drains - TB		2				2		15 00%		
15. 163 Wents & Drains - MC	23	22		2	192	240	29			
15. 164 Wents & Drains - YARD		2				2		15.00%	0	
25 165 Mater Cleanus - PAB		4			7					
rs 164 Mater Treatment - 58		12				12		35.00%		
25.167 Meter Treatment - TB		6				8		15.00%		
15.164 Weler Treatment - 008						6.2		15.00%		
26 Frect Staffalding for Systems		244				244		15.00%	37	3
Decontablnation of Sile buildings										
27.1 Mapor Container	841	53	23		1258	5186	123	34,408	782	21
27.2 Primery Auxiliary	38				132	17.6	83		83	3
27.3 Pucl Handling Area	198			3	368	664	34			
17.4 Service	91			- 3	341	445	33	3 29 889	132	
37.5 Weste Disposal & Tark area	15	3			72	6.0	7			1000
27.8 Compactor	10			0	40	51				
IT.T OIS PCA Storage	- 13			0	49	0.3				
27 & PC/ Plant Berchouse			3		140	180	14	1 29.718	54	
28 License termination survey						292		15 00%		
29 Regulatory fees 30 MCC Fees						470 347		806.3 806.5		
Undistributed Cests										
U 2 : Decen equipment	190					180		25.005	45	
U 2 2 Decen supplies	76					76		25.001		
U.S.3 Process liquid mette	296		351	449	27 19	3814	28			4
U.2.4 Maximar Insurance						86.3		19.005	129	
U.3.8 meni-muclaer insurance						275		15 009		
U.2.6 Property taxes						590		15 009	1 29	

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APPENDING STREET

U 2 8 U 2 8 U 2 8 U 2 10	Health physics supplies									
U 2 10			524			524		25 00%	131	655
U 2 10	Heavy equipment rental		54.26			14.26		15.008	214	1648
	Seell tool allowance		87			\$7		25 00%	22	109
2 2 11	Fipe cutting equipment		534			559		75 008	140	645
	becon rig	7.27		 1.1	248	727	24	25 00% 24 64%	83	318
	Disposal of contseinated solld matte Flant energy budget					874		15 008	135	1006
	Off-Site unities Stati					#\$72		15 805	738	5440
	On-Site Utility Staff					16298		15 00%	2439	126.96
	15#51 Transfer Costs					20		15 00%	3	23
	13751 Licensing Fees					\$ 12		0 00%	8	6.12
	ABC Overhead Expenses					2100		15 00%	315	24.15
	*** F(R)CD 5 ***									
	Removal of Major Equipment									
	soin Turbine-penerator soin Condensers		21 67			21		15 00%	10	24
	Demolition of Site Buildings									

	Vapor Container		1279			1329		15 00%	158	1387
	Turbine		1000			1000		19 00%	150	3 1 50
	Primery Auxiliary		300			300		15 00%	45	345
	fuel Hendling area		211			215		15 00%		24
	Service Diesel Generator		22			27		15 90%		3
	Maste Disposal & Tamk Ares		60			60		15 00%		5
	Compacter		15			15		15 00%	2	7.
	Screen melt & Pump Mouse		121			121		15 00%	18	13
	Seal Pit					3.6		15 00%	3	51
	Safe Shutdown		16			16		15 80%	2	20
	Transformer Foundations		15			36		15 00%	2	
	kon-Essential U.P.S.					1		15.00%	8	
	Did PCA Storage		16			16		15 00%	2	1
	PCA Plant Warehouse Security Diesel Generator		42			#2		15 00%	5	
	Circulating Bales Pipes		72			72		15 00%	5.2	
	suditorium & information							15 005	. · · · · · · · · · · · · · · · · · · ·	
	Mapor Container Elevator		24			24		15 00%		2
	Fire Pune House & TE-55		5			1		15 00%		
	wee & Old Safety injection Tanks							15 00%		
\$3.22	Primery & Secondary Went Stacts							0 00%	8	_ 1 0
	Demineralized moter Tank Foundation		5			5		15 00%		
	office					6		15.00%		
	es rehouse		3.5			11		15 00%	2	
	maintance Polo					6		15 60%	1	
	Cate House Site Crade Alteration		378			8 328		15 50% 15 50%		37
	Cleseeut activities									
	Rackfill Sile		200			20-0		15 60%	30	23
	Grade à Landscape site		182			163		15.00%	26	22
	final Report to ARC Regulatory fees					133		15 005 0 005	20	37
	Undistributed Costs									

	Nuclear Insurance					378		15.009	56	67
	Mon-Rest Ensurance					157		15.80%	24	1.2
	Property taxes		1			265		15 90%	40	34
	Heavy equipment rantat		597			547		15.00%	90	54
U.3.5	Samil tool allowance Flant energy budget		21			21		25 505	8 24	2 58

TLG ENGINEERING, INC.

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22480 2480 50561 1333 \$37, 151 \$252, 135 10781 C0515 % Consingency 5 295 325 4396 4296 14.058 15.00% 13.00% 10.00% 10.00% 2151 2155 1212 Total 5194.984 TABLE 4 3 COST ESTIMATE FOR PORGET DISAAATLING: VARS #V4 75 VIE DECAY (1992 COSE, 10005 UNITE) Burk. Remove Package Ship Decon "a" indicates function performed by utility staff Off-Sile Utility Staff On-Sile Utility Staff 15F51 Tranif & Operating Costs AAC Overhead Froences . ACTIVITY TOTAL ACTIVITY ND

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155 Contingen Total 25 Const Ingency 100 20 100 1463 222 mm TOTAL 2051 [57:8472 120 D[1472 D[154447],1462 ve25 set 53 %1 DECBY (1982 2010, 10005 be118) Burry Ship Package 3 80 たいあっ日 28.5 -Che Con *** PERICO : ADINGATIING ACTIVITES U 1 1 Decen supplies U 2 2 Process light mark U 2 3 Process light mark U 2 4 bon-merical insurance U 2 4 bon-merical insurance U 3 6 Protect V 2555 U 3 6 Protect Definitions U 3 8 Pettap Provide allowance U 3 8 Pettap Contained polid mark U 3 1 8 1931 Cost U 1 1 1 Proceditures U 3 1 8 1931 Cost U 1 1 1 Proceditures U 3 1 1 8 1931 Cost V 1 1 1 2 Pettap U 1 1 1 2 Pettap Fortage Decom spent fuel tacks Linteria survey pilor to doramncy Secure building accesses Prepure a subsit interia report 2 : Pisni yystems 3 2 facility cieseout & dermancy Period a undistributed costs Detailed mork Procedures ACTIVITY SPECIFICATIONS Off-Site Unility Staff CO-Site Unility Staff 15751 Transfer Costs AGC Overhead Expenses ACTIVITY

TLG ENGINEERING, INC. -

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**	** PERIOD 2 methoalled annual maintance ***	 		 *********	N. M. M. M. M. M. M. M. W.	*****	Contingency	*******	

	uarterly inspection								
2 57	eal-annual environmenal survey								
3 91	esare reports								
5.00	caith physics supplies				3.6		15 00%		34
5.50	ucleat insurance on-muclear insurance				251		15 00%	38	281
7 01	operty taxes				167		15 00%	25	192
8 72	schage contaminated solid waste	0	0.5		(4.1)妻		15 00%	\$3	4.8
	sinlance supplies	0	8.5		1		10 31	0	2
	lant energy budget				7.4		15 00%	9.9	#5
11 80	equiatory fees				285		15 00%	22	16.7
12 NR	EC fees				225		0 00%	8	244
	ite and Boiton staff				7580		15 00%	357	225
14 15	SFS: Transfer Costs						15 00%	1	4.21
15 15	SFSI Licensing Fees				375		0 00%		375
16 45	K Overhead Expenses				1287		15 20%	193	1481
	** PERIOD 3 ACTIVITY COSTS ***								
5 20	rvies plant dwgs & specs								
	erform detailed rad survey								
	Nd product description								
- 10 C	tailed by-product inventory				120		15.00%	18	134
5 00	tline mejor mort sequence reform safety analysis							3	
	ibait dismanting plan								
	ceive dismantling order							0	
	ctivity specs								
A 7 A1	r-activate plant & temporary facilities								
	lant systems mactor internals								
	actor wessel								
	islegical shield								
9 6 80	fisture seperators/reheaters				8				
0 7 53	team Cenerators				*			0	6
	einforced concrete							0	
9.9 Tu	urbine & condenser				:			8	
· 18 PS	ant structures a buildings								
9.11 80	ite wana gement				8			8	
9.12 Fa	scillty & site closeout								
	lanning & Site Preperation								
16 #1	ep dismentling seg				1.0				1.
	lant prep & Temp. swces				3			0	0.3
12 De	tsign atr cleamap sys				418		15 00%	63	4511
13 21	geine/CCE/tools/etc				1187		12.000	0	1177
S4 Pr	ocure casks/liners & containers				8		15.00%	180	1377
De	stalled mort procedures								
18.1 P1	ant systems								
	tssel head				3				
15.5 80	eactor internals				-			8	
13.4 Re	easining buildings							8	
15 3 CR	to empline assembly								
	to Housing & ici Tubes							9	
15.7 15	wore instrumentation				8				
15.8 88	tactor wessel				8				
	stillty closeout								
	ssile mields				8				
	lological shield Histore seperators & reheaters								
15.13 51	teas generators				8			8	
	inforced concrete								
	ribine & condensers							8	
	millary building				â			8	

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TLG TV - 2005 16/82

TLO HE 205, 8482)

ACCIVITY									9 CP	nsingen	Tetal
ND.	AC 5 [w] 29	Decon		Partage		Buty	Fotal	CP AGE 1	Contingency	3	Costs
	Feriod 3 Undistributed Costs										
a Thursday	***************************************										1.1.1
	Nuclear Insurance						671 250		25 00%	168	212
	l Non-muclear insurance L'Property sages						530		15 80%	78	608
	Ship & bury maste from periods 1 & 2				10.5	784	8.87	65	22 29%	196	1083
	Health physics supplies		490				480		28 228	135	6.13
U. 1.4	Meavy foulpment mental		234				234		15 00%	35	269
	Plant energy budget						1008		15 00%	163	1251
	E Regulatory fees						338		0 00%		336
	· ···										
	off-site Letility Staff						9030		15 00%	1354	10384
	On-Site Utility Stall						11483		15.00%	1723	13206
	15753 Transfer Ct 195						83		15 00%	6	45
	ISPSI Licensing Fees						561 1924		0.00%	289	563 2213
	AAC Overhead Espenses						1928		12.908		
	*** PERIOD # ***										
1	i tav spent fuel racks		26	#1	13	3190	3270	314	24 778	810	+050
	Auclear Steam Supply System Removal										
	Reactor Coolant Figing	24			14	1135	126.1	108	25 12%	317	1577
	2 Pressullzer Bellef Tanà				3	134	18.1	13	23.66%	49	240
	Beschor Contant Pumps & Actors	18			12	381	291	35	24 50%	120	543
47.1	a Pressurizer 5 Steam Generalars	57			2495	8384	8712	423	22 378	2172	11884
	CROMS/ ICIS/Service Structure Removal	23				324	388	36	50 248	195	583
57.3	f Reactor vessel internals	37			354	7229	4397	6.5	55 32%	2433	6630
17 .1	Beactor wessel	83	3207	27.9	542	2612	\$7.26	190	58.67%	3959	10685
	Paraidous material Revenuel and Disposal										
10	1 Asbestos Removal Program		1154	294		2145	36.18		75 00%	905	#523
18.	2 PCE Removal		\$7				17		25 00%	4	21
	Disposal of Plant Systems										
	Administiztion Building Wentlistice - TB					e			15 05%		
	2 All Dispose! (Filtered Exhaust) - Pab					\$72	722	56	24 28%	175	847
19.	5 sir Disposed (Filtered Exhaust) - SPP					10	13		27.535	3	16
10.	é air Disposat (filtered Exheust) - 908				5	3	2	0	22 53%	0	3
	9 Alt Removal - TB				2	0	7		15.00%		
	8 Auxiliary Steam - TB 7 Battery Encode No. 3 Wentilation - Pap				0 0	0	:	0	15.00%	5	10
	8 Battery Rooms 1 & 2 Ventilation - Pab				, i	8			15 00%		
	a boiler feed - TB				0		76	0	15 00%	5.5	58
	o seiler feed - WC	23	3 31	5	3	217	267	21	24.65%	66	533
	t boilet Feed - VARD					0		8.90	15 00%	260	1301
	2 Charging & Volume Control - PAB 3 Charging & Volume Control - VC	3			7	920	904 9	76	24 935 25 976	237	1151
19.1	4 Chenical feed - TB					0	1	6	15.00%		
	5 Chemical Shutdown - Pag				2	190	217	18	23.47%	51	268
	6 Circulating Water - SB				0	0			15.00%		
	7 Circulating Water - SH		8 95			Ð	10	0	15.00%	1	
	a Circulating meter - Th						3	0	15.00%	8	23
	# Circulating Hater - YARD © Component Opoling Hater - PAB	1.1.1.1			0		31		15 00%		35
	1 Component Cooling Mater - WC					427	750	61	25.438	193	950
28 2	2 Component Cooling Water - 1008				0		3		15 00%		3
19.3	3 Component Cooling Meter - YLED		5 7		0		2	6	15 009		2
	4 COMpressed air - PaB		8 3	5 E	Ð	0	- 3	0	15 00%		4
	5 Compressed air - 58				8	0	1	8	15.00%		5
	N Compressed Air - SFP 7 Compressed Air - SFSS					5	0 6	0	15 009		
	8 Compressed atr - TB		5 1				15		15 005	2	\$7
18.7	9 Compressed als - WC				0	18	21	2	23 50%	5	26
19.3	Compressed air - wcm				9		3	0	15 008		2
18.3	1 Compressed als - YARD						2	0	15 20%		2
	2 Centrel Room Ventilation - TS						7		15 008		
	3 Oxfresion Control - Pub 4 Desincralized Weter - Pub		8		8	8			15 00%		
	STATISTICS STATES								2 20 20		
	5 Demineralized Water - 78		6 3						15.00%		

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TLD F8 (205 (6/82))

		TABL	2 4.	3		
COST	ESTIMATE	F (38)	DI.A	4ED	DISMANTE	ING:
					Decay	
	11002	ce 25		100	48217942	

ACTIVITY ND.	Activity	Decon	areacive.	Package	ship		BUTY	Total	Cu yes	Contingend	CW 3	ingen S	Total Costs	
	Activity Desineralized uster - WC8		(a,b,a,a,b,b,a,b,b,a,b,a,b,a,b,a,b,a,b,a			$(f_{i}) = (f_{i}) = (f_{$		(* * * * * * * * * *				0		
	Demineralized water - wom					0		8		15.00	80			
19 38	EDC Building ventilation - DCB			0		8				15 01				
10 30	fmergency Boller Feed - PAB		2. 7	0				7					2	
	Emergency Boller Feed - TB	1.	5 3			0				15 0	1.8			
	Emergency Boller Feed - VC		2 2			8	20	23		25 8	05			
	Emergency Boller Feed - YARD			0		0	0	4.5		15.0	28	2	12	
19 43	faergency Power - DG8		8 1			0	0	11		15.0			1	
18.44	Emergency Power - 58					ö				15 0	016		2	
19 45	Exergency Power - SDCB Exergency Power - SFSS		5 1			8	8			15 0		0	5	
	tweigency Power - \$358	1.111		0			Ø	3		5 15 0		8	. 4	
	Emergency Power - TB	10.00	5 12	0		0	0	12		5 15 0		2	13	
19 49	Estraction Steam - TB		8 19			0		2.9		0 15 0 5 15 0		3	22	
	Fire Detection & Suppression - DOB		9 2			0		-						
18 51	Fire Detection & Suppression - FPH			0		0	0	9		0 15 0 6 15 0		1	10	
19 52	fire Detection & Suppression - FWP		0 9			- 14	0	4	25.23	0 15 0			4	
19.53	Fire Detection & Subpression - PAB	4.2. H	6 4 6 7	0		0		3		8 13.0				
	Fire Detection & Suppression - 3H		6			ē	2	3 92 29		0 15.0			5	
14.95	Fire Detection & Suppression - 31-W			0		0	0	97		0 15.0	0%	16	106	
19.56	Fire Detection & Suppression - TB		B 41	0		0	0	29		0 15 0		7	57	
	Fire Detection & Suppression - YARD					8	0	0		0 15 0		8		
	fipor Drainage - PAB Fuel Dit - DOB		0							0 15.0		0	1	
10 00	Fuel Dit - FOTH		8	1 0		0	G	1		0 15.0		0		
	Fuel Dil - TB		0	1 0		0	0			0 15 0		0		
	Fuel Dit - VARD		S 1			0	0	7						
19 63	Cas Storage Roos ventilation - PAS		0	1 0		0	6			0 13.0		5		
19 64	Generation & Transmission - SEVD					0	0	1		0 15 3		8	2	
19 65	Generation & Transmission - TB		8	1 0		0	0	1.1.2		0 15.0	10.5			
	Generation & Transmission - YARD			3 0				3		0 15 1			3	
	Generator Gas - TB		0	3 0		0	0			0 15.1		0	1	
	Generator Seal Oil - TB		8	3 0		0	0			0 15.4		0	4	
	Cland Seal - TB		0 2	e		ö	0	28		6 15.1		4	26	
19.75	Hester Drains - TB		0			8		3		0 15 1		0	3	
19.71	1 Heating Steam/Condensate - DCB 2 Heating Steam/Condensate - FTCS			3 6			0			8 15.1		0		
19.71	Meating Steam/Condensate - FMP		-	1 0		0	0	- 1		0 15.1		0		
+9.7	Neating Steam/Condensate - NFV		0	5 6	i	0	0			0 15.1		8		
16 71	5 Heasing Steam/Condensate - PAB		¢ 2	6 8		8		26		0 75 1		4	30	
19 71	6 Heating Steam/Condensate - PCA1			0 0		0	0	0			00%			
19.7	7 Heating Steam/Condensate - PVS		-		P	0		1		0 15		8		
19.71	a Meating Steam/Condensate - 58		-	0 4		8				6 13			2	
	Mesting Steam/Condensate - 5550			2 6	5	0				0 15		0	2	
	6 Heating Steam/Condensate - ST-8			1 1			0	45		8 13		7	53	
19.8	1 Heating Steam/Condensate - TB			2 2		82	1184	23.16	1	15 23.	89%	314	7831	
19.8	2 Heating Steam/Condensate - VC			0 1		6	0	10		0 15.		2	82	
	3 Heating Steam/Condensale - NOR s Heating Steam/Condensale - VARD			2 1		0	0	7		D 15.				
	5 Lubricating Dil - 78			2 1		0	0	12		Q 15		2	83	
	a main steam - SISS			5	B	0	0				00%		2	
	7 main Stene - TB		8 2	18 4	8			28		6 15		4	32 835	
	a main Staam - WC			10 1.				662			238	174	23	
19.8	9 aain Steam - YARD				8	0	D	20				27	139	
19.8	e aisc seactor coolant - WC				3	2	103	0		8 15		8		
	is hav Enclosure ventilation - VaRD				e 5						800			
	17 Nitrogen - PAB				8	ě				0 15	80%	8	2	
19.9	13 witregen - YARD		1000		6		100	2		6 15	806		2	
10.0	e PAB Non-filtered Ventilation - PAB					2		379		10 25.	33%	99	268	
	A Pressure Control & Rellet - VC & Purification - LEP				5		117	14.9			13%	35	176	
	7 Purification - PAS				0		401	443			848	106	349	
	A Endimentive meste Disposal - IEP				1	0	49				608	13	6.9	
	M Radioactive maste Disposal - PAR		6 3	37	9	3	351	400			57%	95	495	
19.10	No Radioactive Maste Disposal - 78		e		8	8	0				005	0	2	
	11 Badloactive Waste Disposal - VC				0						71%	2 480	7569	
	22 Radioactive Waste Dispossi - 008			26 4		15	1795			75 23	538	480	1985	
	3 Radioactive Waste Disposal - YARD				5	0					00%			
19.10	sale shutdown - 3558				9		0	4			00%			
19.58	15 Safe Shutdown - YARD				5	8					00%		1	
19. 14	M Safe Shutdown Building Ventilation - SSS				e :	5	800				67%	218	1127	
	17 Safety injection - DCB				γ γ	2	285				90%		393	
	Me Balety injection - Pas				6	2	230				5.9%		327	
10.00	M Safety Injection - SIS				4	1	170			17 24	8.7%	96		
	it talety intertion - yath			20	0					8 15	805	3	24	
19.21	Natery injection - VC Isafety injection - VC It Safety injection - VARD		8	76		2								256

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TABLE # 3 COST ESTIMATE FOR DELAVED DISMANTLING VMPS M/8 75 VI DECAY (1992 COST, 10005 Whits)

ACTIVITY ND.	45=14184	Decon		Package		BUTY	70 28 5		Contingency		Costs
19, 11;	Safety injection bidg ventilation - PAB		2	c	0	0	2	0	15 008		2
	Sample - PEB		26	5	2	233	267	23			330
19.15	sample - WC				2	157	215	18			266
19.11	Sanitary Disposal - YARD	0		0		0	1	0	15 00%		1
	i Screenwaish - SH f Screenwell Pumphouse Ventilation - SH	0				0	3	8			
	Service Building wentliation - 58	6						0			
	Service mater - FAS	a a					15	0			17
	Service mater - SH			0			12	0			13
	Service mater - TB		35	0			35	0			40
19 123	2 Service mater - WC	0	67	25		1020	1123	99	23 968	26.9	1340
	Service Mater - VARD	0		6		0	18	6	15.008	3	20
	Shuidown Cooling - PAB	46	86	18	\$	744	章7 作	72			1101
	Shutdewn cooling - sc	5	14	3	1	10.4	126	10			158
	Spent fuel - stP	0	3	0		2	3	9			3
	Spent Fuel - PAB	0		8			270	24			335
	I Spent fuel - SFP R Spent fuel - MC	0				126	145	12			180
	Spent fuel - YARD	0			0		5		24.019		
	Station Service - DE R	e		0					15 009		0
	Station Service - DOB	0			é						
	Sixtion Service - THP	e			e				15 001		0
	E 1 million Service - OH	0		0	0	6			15.005		43
19.13	Station Service - at	0		0		0	6		15.008		7
19.13	Station Service - NEUPS	0	0			0	0	6	13 009		0
	f Station Service - #AB	0		0	. 6	0	167	0	15.005	25	192
	s Station Service - PAB (Contam)			8.7		6.36		6.2	33.061		96.1
	s Station Service - PB	0		0	2	0		6	15.001		0
	0 Station Service - PCA1	0					0	0			0
	t Station Service - PCA2	0					0	0			0
	2 Station Service - Pace	0					0				0
	3 Slation Service - PVS 6 Station Service - S8	0					0 2		15 009		0
	5 Station Service - 5008	0					0	0	15 001		2
	s Station Service - S**	0					0				
	7 Station Service - 5.										
	# Station Service - SLED										0
	Station Service - STW	9									0
18.15	o station service - savo	9					8				0
	1 Station Service - TAB	0		6		0		0	15.009	L 0	
	2 Station Service + TB	0				0	183				2 57
	3 Station Service - WC	e			12		1956				2406
	a Station Service - 1908	ø				0	2.2				13
	S Station Service - #D8 (Contam)	0			3			32			500
	6 Station Service - 994 7 Station Service - 9949	0			0		1	0			2
	& Station Service - VARD				0	0	32				80
	9 Turbine Building Ventilation - TB						10				11
	0 WC Heating/Cooling/als Recirc - WC						2916				3622
	1 Wents & Crains - PAB	0				1052	1140				1461
	2 venis & Drains - 18	e	2	0		0	3	0			3
	3 vents à Drains - VC	21	21		2	192	241	19	25 951		304
	a Wents & Droins - YARD	0				0	2	5			3
	3 Mater Cleanup - PAB				e	7		1			01
	é males Treatment - SB	0					12				5.6
	7 Water Treatment - TB	0									6
	s water Treatment - 000	•			c	•	8	•			
	6 Erect Scattolding for Systems		23 9				231		15.004	L 35	266
	Decontamination Of Site Buildings										
	i Wapor Container	308	50	25		1258	2148	123	34 153	5 734	2883
	2 Primery Auxillary	37	8				173	13		\$ \$2	725
	3 For 1 sandling area	199		7	3		\$30	36			858
	4 Service	56		7	3		438	33			558
	5 Weste Disposal & Tank Area	57			1	72	93	7			120
	\$ COMPACTOR	10			0	43	51	4			65
	7 Did PCA Storage	12					63	5			82
	8 PCA Plant marchouse	35		3		140	*79	14	29.501	5 53	232
3	3 ticense termination survey						257		15.001	8 39	295

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222 湖자 Tetal Costa 宗光兒 s contingency \$ 595 297 297 -100 200 200 200 200 200 200 ----22 55565 142 33 101 3 193 4625 5379 30 376 1977 50 Total DISMATLING: Decay uni 133 233 0861 BULY TABLE 4.5 ESTIMATE FOR DELAYED D WAS 8/8.75 VIS D U 10005 W ** 554 3519 Parkage いた an) 193 53 500 m a can COST 71 ž MB 116 23.1 Manor Container 23.3 Turbine 23.3 Turbine 23.3 Fuel Manding Area 23.5 Service Mail & Puer House 23.5 Service Mail & Pu Demilition of site buildings 8 1 Decon equipment 2 Decon equipment 3 Start relocation exberids a Process liquid waite 2 Sucreas liquid waite 5 war test insurance 5 war test insurance 6 war exupted insurance 7 Property Stars 7 Property Stars 9 Sault robal incomated 10 Sault robal Contaernated So 3 Start every budget 10 Start every budget 13 Start every budget E 001834 *** Site Claseout Activities thechiel Site Caso & Landscape site i final Report to ABC Seacuel of ablor toulparint DFF-Site Utility S1815 Cm-Site Utility S1815 15751 Tranifer Costs 15551 Licensing Ferst add Overhead Expenses MAIN Turbine/generator Main Condensers undistributed costs ACTIVITY *** P1 75 activity MD. - TLG ENGINEERING, INC. -

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TABLE 4.3 COST ESTIMATE FOR DELAYED DISMANTLINC: 9405 878 75 Yrt Decay (1992 cust. 10005 unity)

Activity NO.	activity	 emove Pack	and the second	Birry	Total	Cu Yds	Continger		contingen S	Total Costs
U 1 2 U 1 3 U 1 4 U 1 5 U 1 6	undistributed Costs Auclear insurance Anon-nuclear insurance Property Iaxes Heavy equipment rental Small tool allomance Plant energy budget Regulatory les	502 21			368 136 602 21 160 270		15 15 15 28 15 0	200 200 200 205		#24 180 288 693 26 184 270
	Off-Site Utility Staff On-Site Utility Staff ISFS: Transfer & Operating Costs A&C Overhead Expenses				19#3 2193 36914 1205		15 15 10 10	00%	373 3691	2236 2478 40606 1326
	TOTAL				\$208.727		18.	398	\$38.390	\$247.117

"a" indicates function performed by utility staff

AN 0.14

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4.6 DECOMMISSIONING vs. SITE RESTORATION

The total projected cost of dismantling the YNPS, for the DECON alternative, is \$232,134,790. Of the total cost, approximately \$210,324,690 is directly attributable to the engineering and planning for and the actual disposition of the residual radioactivity at the YNPS. Of the total cost for SAFSTOR, \$247,117,090, approximately \$224,896,870 is directly attributable to the engineering and planning for and the actual disposition of the residual radioactivity at the XNPS. It should be noted, however, that a direct accounting of only these costs is not entirely accurate in portraying the actual cost of "decommissioning" as defined by the NRC and consideration must also be given to the methods of executing the decontamination processes.

Nuclear power plants are designed to contain the radioactivity inherent in the normal operation of the facility. Accordingly, radioactive and potentially radioactive systems are located in shielded labyrinths, tunnels and pipe chases. This inaccessibility, while essential during operation serves to impede decommissioning activities. Consequently, disposition of these components requires that in many situations that additional access (and working space) be developed. This access is achieved by dismantling structures and components along the intended path of egress and in the immediate working area; material which in most cases is non-radioactive and therefore not normally perceived as a necessary constituent in facility decontamination. Failure to establish adequate working room will increase the residence times for decontamination and dismantling activities resulting in increases in the incurred occupational exposure.

The cost associated with the removal of non-contaminated and other releasable materials in support of the decommissioning process are commonly referred to as cascading costs. Upon evaluating the dismantling processes involved in decommissioning the YNPS, it is estimated that an additional \$7,924,950 of "cascading costs" will be incurred for the DECON alternative and \$7,821,930 for the SAFSTOR alternative in the decommissioning process. Consequently, for the utility to meet the intent of the NRC's definition of decommissioning, ("...release of the property for unrestricted use and termination of license") a cost of \$218,249,630 would be required to terminate the facility's license for the DECON alternative, or approximately 94.02% of the total cost. The total cost to terminate the license for the SAFSTOR alternative is 232,718,800, or 94.18% of the total decommissioning cost. These percentages of the projected costs for license termination at YNPS are NRC's minimum requirements for decommissioning as delineated in title 10 of the code of Federal Regulations, Part 50.75. The remaining 5.98% for DECON and 5.82% for SAFSTOR would be required for site restoration as described in Section 3.

4.7 SPENT FUEL STORAGE COSTS

Costs associated with the design, licensing, construction, and operation of an independent spent fuel (SF) storage installation (ISFSI) have been included in the cost estimate. ISFSI costs are based on the following assumptions: (1) the U.S. Department of Energy (DOE) removes all YNPS spent fuel between 1998 and 2018, and, (2) YNPS transfers any SF remaining in the spent fuel pit to the ISFSI by June 1, 1996. Thirty vertical, dry concrete storage casks, each holding 21 spent fuel assemblies, are required. The rate of spent fuel removal by DOE is based on the lower rate of fuel acceptance published by DOE in the Annual Capacity Report until the opening of a repository in 2010. Thereafter, accelerated shipment of YNPS SF is assumed until 2018 when the last SF is removed by DOE.

5. SCHEDULE ESTIMATE

The schedules for the decommissioning alternatives considered for the YNPS in this study follow the sequence presented in the AIF/NESP-036 study with minor changes to reflect recent experience and revised estimates. TLG has devised schedules for YNPS. The assumptions are listed in Section 5.1. Figure 5.1 presents the schedule of key activities for the DECON scenario. Figure 5.2 presents the schedule for the SAFSTOR scenario. Note that the activities listed in the schedules do not reflect a one to one correspondence with the activities in Tables 4.2 and 4.3, but reflect splitting some activities for clarity and combining others for convenience. Figure 5.3 contains a legend defining the schedule nomenclature and depictions. The schedule was prepared using the computer code "HPM-3.02" (Ref. 13).

5.1 SCHEDULE ESTIMATE ASSUMPTIONS

The schedule in Figures 5.1 and 5.2 reflect the results of a precedence network developed for the YNPS decommissioning activities. The durations used in the precedence network reflect the actual manhour estimates from Tables 4.2 and 4.3. The schedule output is then adjusted by stretching certain activities over their slack range; other activities were pushed to the end of their slack period. The following assumptions were made in the development of the schedules for the YNPS.

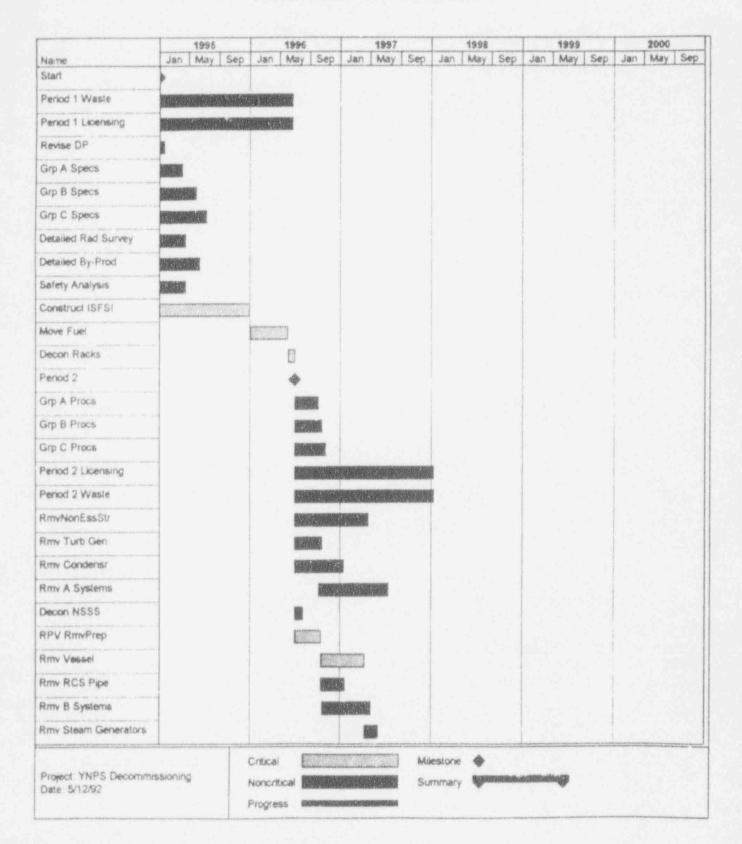
- 1. All work except vessel and internals removal activities is performed during an 8-hour workday, 5 days per week with no overtime. There are eleven paid holidays per year.
- 2. Vessel and internals removal activities are performed by using separate crews for different activities working on different shifts, with a corresponding backshift charge for the second shift.
- 3. System and facilities supporting operations in the Spent Fuel Pit (SFP) will continue until such time that all spent fuel has been transferred from the SFP to dry cask storage modules, i.e., decontamination of the Spent Fuel Pit can begin approximately fifty-six months (56) after shutdown.
- 4. Multiple crews work parallel activities to the maximum extent possible consistent with optimum efficiency, adequate access for cutting, removal and laydown space, and with the stringent safety measures necessary during demolition of heavy components and structures.

5.2 PROJECT SCHEDULE

The period dependent costs presented in Tables 4.2 and 4.3 are based upon the durations developed in the schedules for the DECON and SAFSTOR alternatives. Durations are established between several milestones in each project period; these durations are used to establish a critical path for the entire project. In turn, the critical path duration for each period was used as the basis for determining the total costs for these items.

A project time line is delineated in Figure 5.4 for both decommissioning alternatives. Milestone dates are based on a 31 year plant operating life.

FIGURE 5.1 DECON ACTIVITY SCHEDULE



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	FIGURE	5.1
DECON	ACTIVITY	SCHEDULE
	(continue	d)

and an experiment of the second se	9	995	191		1997		1998	I second a second second second second		1999			2000	Terr
Name	Jan I	May Sep	Jan Ma	ay Sep	and the second se	iep .	Jan Mey	Sep	Jan	May	Sep	Jan	May	Sec
Rmy Pressurizer														
VC Group C Sys]								
VC Decon														
PAB Group C Sys														
PAB Decon						1								
PAB Group D Sys					KINI	1								
FHA Group C Sys					I.									
FHA Group D Sys														
Remove Racks														
FHA Decon														
TB Group C Sys									÷					
TB Group D Sys					M									
Other Group C Sys					8									
Other Decon					1.1									
Other Group D Sys					1									
Fni Survey						£	3							
Period 3	1						\$							
VC Interior Demolition							[3						
DG Interior Demolition]					1	1		6.					
DG Exterior Demolition	1.													
Office Interior Demolition	1					-	1. 3		12			63		
PAB Interior Demolition									['					
PAB Exterior Demolition	1								6.2					
FHA Interior Demolition					1.1.1.1.1			1	£.,					
FHA Exterior Demoiltion	1994				1.11			1	Ε.					
SB Interior Demotition	1		1.1				8		10					
SB Exterior Demolition					1.30 30		1		1.					
TB Interior Demolition	1													
TB Exterior Demolition								1				1		
Project YNPS Decommi Date 5/12/92	ssioning		Critical Noncritica Progress				nmary 👹	,		andh				

FIGURE 5.1 DECON ACTIVITY SCHEDULE (continued)

	10	1995		1	1896			1997			1998			1999			2000	
Name	Jan	May	Sep	Jan	May	Sep	Jan	May	Sep	Jan	Mary	Sep	Jan	May	Sep	Jan	May	Sep
VC Exterior Demoiltion																		
Backfill Site												. 1	3					
Rmy Essential Struc	1.											1						
Landscaping	1 .												1					
End																		

Project: YNPS Decommissioning Date: 5/12/92			Milestone Summary		
--	--	--	----------------------	--	--

1995 1996 1997 1998 1999 2000 2001 2002 2003 Jan Jul Name Start Construct ISFSI Move Fuel Decon Racks Interim Survey Period 1 Waste Period 1 Licensing STREET, CA Period 2 Domancy Period 3 Grp A Spece Grp 8 Specs Grp C Specs Detailed Rad Survey Detailed By-Prod Safety Analysis Period 3 Licensing Revise DP Period 3 Waste Period 4 Grp A Procs Grp B Procs Grp C Proce Period 4 Licensing Period 4 Waste (;) RmvNonEssStr Rmv Turb Gen Rmv Condens/ Rmv A Systeme Milestone Critical Project YNPS Decommissioning Summary SCHOOL STATE OF STATE OF STATE Noncritical Date 5/12/92

FIGURE 5.2 SAFSTOR ACTIVITY SCHEDULE

FIGURE 5.2 SAFSTOR ACTIVITY SCHEDULE (continued)

	1995 Jan Jul	1996 Jan Jul	1997 Jan Jul	1998 Jan Jui	eeer lui nat	Jan Jul	Jan Jul	Jan Jul	2003 Jan Jul
Name Decon NSSS	Jan Jul	Jan Jul	Jan J Juli	Jan Jun	Jan John		San Son		
RPV RmvPrep									1444.4
Rmv Vessel						F	100		
Rmv RCS Pipe						12.			
						ESS.			
Rinv B Systems									
Rmv Steam Generators									
Rmy Pressurizer							1		
VC Group C Sys									
VC Decon									6. DE 1
PAB Group C Sys							an Car		
PA8 Decon]						1		
PAB Group D Sys									
FHA Group C Sys							1		
FNA Group D Sys									
Remove Racks							1		
FHA Decon							200000		
TB Group C Sys								1997.2	
TB Group D Sys								1.1.1	
Other Group C Sys						1.11	1	1.1	
Other Decon	1								1.50.5
Other Group D Sys							1		
Find Survey									
Penod 5			1				1.125	*	
VC Interior Dmin									1.1
DG Interior Dmin	8. A.							1	
DG Exterior Dmin									
Office Interior Dmin						6.79		-	
PAB Interior Dmin	Sec.		1.1					8	
PAB Exterior Dmin								1	
Project: YNPS Decommit Date: 5/12/92	ssioning	Critica Noncri Progre	itical		Milestoni Summar		ann an the second s		

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	19	95	19	96	19	97	19	98	19	99	20	00	20	01	20	02	20	03
Name	Jan	Jul	Jan	Jul	Jan	Jui	Jan	Jul	Jan	Jul	Jan	Jul	Jan	Jul	Jan	Jul	Jan	Ju
FHA interior Dmin															No.		£	
FHA Exterior Dmin																1		
SB Interior Dmin	1.				1		1								8			
SB Edenior Dmin	1						1								1			
TB Interior DmIn	1														10000			
TB Exterior Dmin	1															1		
VC Exterior Dmin																		
Backfill Site																8		
Rmv Essential Struc	1															1		
Landscaping	1.															- 1		
End	1																	

FIGURE 5.2 SAFSTOR ACTIVITY SCHEDULE (continued)

	Critical	Milestone	*
Project YNPS Decommissioning Date: 5/12/92	Noncritical	Summary	(Warnerson and Carlos
	Progress		

FIGURE 5.3 DEFINITION OF TERMS

Backfill Site	Backfill below grade voids
Construct ISFSI	Construction of dry storage compound
Decon Racks	Hydrolasing spent fuel racks
Decon NSSS	Reactor system decontamination
Detailed Rad Survey	Site characterization prepared in support of decommissioning planning
Detailed By-Prod	Calculation of site radionuclide inventory
DG Exterior Dmln	Demolition of the exterior of the Diesel Generator Building (DG)
DG Interior Dmln	Demolition of the interior of the DG
Dormancy	SAFSTOR dormancy period
End	End of primary decommissioning operations
FHA Decon	Decontamination of Fuel Handling Area (FHA)
FHA Exterior Dmln	Demolition of the exterior of the FHA
FHA Group C Sys	Disposition of essential contaminated systems located with the FHA
FHA Group D Sys	Disposition of essential non-contamination systems located within the FHA
FHA Interior Dmln	Demolition of the interior of the FHA
Fnl Survey	Final site radiological survey in support of license termination
Grp A Procs	Preparation of detailed work procedures for non-essential systems
Grp B Procs	Preparation of detailed work procedures in support of NSSS disposition
Grp C Procs	Preparation of detailed work procedures for essential contamination systems
Grp A Specs	Preparation of activity specifications for non-essential systems
Grp B Specs	Preparation of activity specifications in support of NSSS disposition
Grp C Specs	Preparation of activity specifications for essential contamination systems
Interim Survey	Radiological survey, prior to plant dormancy
Landscaping	Landscape site
Move Fuel	Transfer of spent fuel to dry storage compound
Office Interior Lymln	Demolition of the interior of the Office Building
Other Group D Sys	Disposition of essential non-contamination systems located
Caller Group D 035	within the remaining facility
PAB Interior Dmln	Demolition of the interior of the Primary Auxiliary Building (PAB)
PAB Exterior Dmln	Demolition of the exterior of the PAB
PAB Decon	Decontamination of PAB

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FIGURE 5.3 DEFINITION OF TERMS (continued)

Disposition of essential contaminated systems located with the PAB Group C Sys PAB isposition of essential non-contamination systems located PAB Group D Sys thin PAB ensing activities in support of decommissioning operations Period 1 Licensing JECON), and mothball operations (SAFSTOR) Processing waste generating in Period 1 Period 1 Waste Decommissioning Operations (DECON), Dormancy Period 2 (SAFSTOR) Period 3 Site Restoration (DECON), Preparations for delayed decommissioning (SAFSTOR) Licensing activities in support of Period 3 decommissioning Period 3 Licensing planning (SAFSTOR) Period 3 Waste Processing waste generating in Period 3 (SAFSTOR) Period 4 Delayed Decommissioning Operations (SAFSTOR) Period 4 Licensing Licensing activities in support of decommissioning operations (SAFSTOR) Period 4 Waste Processing waste generating in Period 4 Site restoration (SAFSTOR) Period 5 Remove Racks Remove spent fuel racks Disposition of systems non-essential to decommissioning **Rmv** A Systems operations Disposition of systems essential to removal of reactor system **Rmv B Systems** Removal of the condenser Rmy Condensr Dismantle remaining essential structures and facilities **Rmv** Essential Struc Demolition of non-essential structures RmvNonEssStr Rmv Pressurizer Disposition of pressurizer Disposition of reactor coolant piping **Rmv RCS Pipe** Disposition of steam generators **Rmv Steam Generators** Removal of the turbine generator set Rmy Turb Gen Segmentation of reactor vessel and internals Rmv Vessel Revision of Decommissioning Plan for delayed decommissioning Revise DP (SAFSTOR) Preparation for reactor vessel and internals segmentation RPV RmvPrep Preparation of Safety Analysis Report Safety Analysis Demolition of the exterior of the Service Building (SB) SB Exterior Dmln Demolition of the interior of the SB SB Interior Dmln Commencement of decommissioning operations on 1/1/95Start Demolition of the exterior of the Turbine Building (TB) TB Exterior Dmln

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FIGURE 5.3 DEFINITION OF TERMS (continued)

영상 방법을 감독하는 것이 많이 많이 많다.	
TB Group C Sys	Disposition of essential contaminated systems located within the TB
TB Group D Sys	Disposition of essential non-contamination systems located within the TB
TB Interior Dmln	Demolition of the interior of the TB
VC Group C Sys	Disposition of essential contaminated systems located within the Vapor Container (VC)
VC Decon	Decontamination of VC
VC Interior Dmln	Demolition of the interior of the VC
VC Exterior Dmln	Demolition of the exterior of the VC

FIGURE 5.2 Yankee Nuclear Power Station Decommissioning Timeline (not to scale) DECOW (Prompt Removal Dismantling): Spent Fuel Storage Wet ISFS1 ----- Dry ISFS1 -| Preparations | Decommissioning | Site Operations Restore Stertup Period Period Period Shut-TLG ENGINEERING, INC. 2 3 down 1 36 38 39 58 ---- Plant Operation ------>31 35 <-Construction-> <-----1996 1998 1999 2018 1991 1995 1960 1957 Dec July Jen Jan. Nov July Oct Jan

SAFSTOR: (Mothball with Delayed Dismantiing)

ALL IN MARCH

- Dry ISFSI -<----- Wet ISFSI -----> | <-----Mothball | | Preparations | Decommissioning | Site Operations Restore Operations | Dormancy Stertup Period Period Period Period Period Shut-5 3 4 1 2 down 42 42 58 («-Construction->«---- Plant Operation ------>31 35 36 38 40 2018 2000 2002 2002 1996 1999 1957 1960 1991 1995 Dec Dec Jul Jan Nov July Oct Jan July Jan

Spent Fuel Storage

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6. RADIOACTIVE WASTE VOLUME

The radioactive waste volume generated during decommissioning is shown by line activity in the cost tables. Approximately 3,556 cubic yards of radioactive material are generated in the DECON alternative and 3,747 cubic yards are generated in the SAFSTOR alternative. SAFSTOR produces additional volumes of waste from dormancy operations.

Waste volumes are quantified consistent with 10 CFR 61 classifications. The waste volumes shown in Table 6.1 are calculated based on the gross container volume to be shipped and buried in controlled burial grounds.

Most of the materials for controlled burial are categorized as Low Specific Activity (LSA) material containing less than Type A quantities as defined in 49 CFR 173-178 (Ref. 14). The containers must be strong tight packages. For this study, commercially available steel containers are used for packaging piping, small components and concrete.

The reactor vessel and internals are categorized as large quantity shipments and, accordingly, must be shipped in reusable shielded casks with disposable liners. In this case, the liner volume is taken as the waste volume.

The waste volume attributed to decommissioning is primarily generated during Period 2 of DECON, and Period 4 of SAFSTOR. The radioactive waste generated as a result of the decommissioning of the YNPS is destined for disposal at an unspecified burial facility within 1,000 miles of the site.

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TABLE 6.1

RADIOACTIVE WASTE BURIAL VOLUMES FOR DECOMMISSIONING YNPS

Alternative		Volume ² cubic yards)
DECON		
Total	A B C >C	3,233.0 235.8 41.9 <u>45.3</u> 3,556.0
SAFSTOR		
Total	A B C >C	3,507.4 175.0 39.4 <u>25.2</u> 3,747.0

Waste is classified according to the requirements as delineated in Title 10 of the Code of Federal Regulations, Part 61.55

No estimate has been made of the LSA waste that will be generated during the operation of the fuel storage facility.

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7. OCCUPATIONAL EXPOSURE

Estimates of occupational radiation exposure were developed by TLG Engineering. Radiation doses to decommissioning workers are calculated as the product of the estimated radiation zone work force requirements and the radiation exposure rates postulated for each decommissioning task. The decommissioning occupational exposure estimates are based on the following assumptions:

- 1. Occupational exposure estimates include only the craft labor necessary for decontamination, removal and packaging activities as well as all required health physics personnel exposures in support of these activities. Casual exposures to the plant staff are not included in this estimate.
- 2. Personnel exposure to radiation is minimized by utilizing shielding and remote handling techniques and avoiding higher radiation fields when personnel presence is not necessary.
- 3. Local exposure rates near items such as tanks and pipes are reduced by a successful chemical decontamination program prior to work in that area.
- 4. Careful prompt accounting of accumulated radiation exposure is maintained to rapidly identify tasks causing excessive dose accumulation by workers so that corrective action can be taken.
- 5. Cobalt-60 is the primary contributor to radiation exposure. The reduction in personnel exposure from area and components after the delay is assumed to be that of the reduction in Cobalt-60.
- 6. Exposures as the result of spent fuel storage activities are expected to be minimal and therefore are not included.

Table 7.1 summarizes the occupational personnel exposures that are projected to be incurred in the performance of the various decommissioning activities. Exposures were calculated from an estimate of residence times within the various radiation areas for each plant system and structure targeted for decontamination and decommissioning. The totals reflect only craft exposure, i.e., no estimate has been made of staff exposures due to their incidental nature.

TABLE 7.1

OCCUPATIONAL EXPOSURE FOR DECOMMISSIONING YNPS

Decommissioning Alternative	ManRem ¹
DECON (Prompt Dismantling)	1,094.5
SAFSTOR (Mothball w/Delayed Dismantling)	1,037.7

No estimate has been made of the occupational radiation exposure that will be incurred during the operation of the fuel storage facility due to the low residency times required in any radiation field.

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8. CONCLUSIONS

Decommissioning technology is well established and the tools and equipment necessary to completely dismantle the YNPS are available and have been demonstrated. The cost to decommission the nuclear station using the DECON (Prompt Removal/Dismantling) alternative is \$232,134,790, including shipment of all wastes and dismantled materials to a regional burial site and demolition of the remaining site structures. The cost for decommissioning the YNPS using the SAFSTOR option is \$247,117,090. The estimates reflect the site-specific features of the YNPS and the estimated cost of radioactive waste shipping and burial costs. An analysis of the major activities contributing to the total cost is shown in Table 8.1.

The staff costs and burial costs represent the largest percentages of the total cost. These cost centers reflect the labor intensive nature of the decommissioning process and the large increase in recent low-level waste burial cost projections, respectively. Shipping will be most sensitive to increases in fuel costs and distances to existing or new burial facilities. Removal costs are dependent on the degree of remotely operated equipment available in the future and the associated higher cost of that equipment versus the savings in labor costs. These results point to the need for periodic reviews of these estimates.

This study for the YNPS provides an estimate for decommissioning the site under current requirements based on present day costs and available technology. As additional dismantling experience on large reactors becomes available, cost estimates must be modified to reflect this experience. In addition, historical the costs for low-level waste disposal have increased at rates significantly higher than inflationary trends and, therefore, should be reviewed periodically.

SUMMARY OF I	TABLE 8.1 DECOMMISSIONING C	COSTS
Work Category	1992 Costs (Thousands)	Percent of Total Costs
DECON (Prompt Removal/Dismantli	ng)	
Decontamination	3,991	2.05
Removal	20,130	10.32
Packaging	1,906	0.98
Shipping	4,405	2.26
Burial (off-site)	43,682	22.40
Decommissioning Staffs	45,776	23.48
Other *	75,094	38.51
		100.00
TOTAL **	232,135	
SAFSTOR (Mothball w/Delay Remov	val/Dismantling)	
	. 이상 위험하는	
Decontamination	2,948	1.41
Removal	20,165	9.66
Packaging	1,818	0.87
Shipping	4,083	1.96
Burial (off-site)	41,472	19.87
Decommissioning Staffs	59,386	28.45
Other *	78,855	37.78
		100.00
TOTAL ***	247,117	

Other includes: insurance, property taxes, plant energy budget, and ISFSI transfer costs as examples

** Includes an average contingency of 19.05%

*** Includes an average contingency of 13.39%

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9. REFERENCES

- 1. Federal Register Volume 53, Number 123 (p 24018+), June 27, 1988, Nuclear Regulatory Commission, Code of Federal Regulations, Title 10, Parts 30, 40, 50, 51, 70 and 72 "General Requirements for Decommissioning Nuclear Facilities."
- U.S. Nuclear Regulatory Commission, "Final Generic Environmental Impact Statement on Decommissioning of Nuclear Facilities," NUREG-0586, August 1988.
- U.S. Nuclear Regulatory Commission Regulatory Guide 1.86, "Termination of Operating Licenses for Nuclear Reactors", June 1974.
- Yankee Atomic Electric Company correspondence YRP 250/92 W.O. #3791 dated May 1, 1992.
- 5. Chem-Nuclear Services, Inc., Low-Level Radioactive Waste Management Facility, Barnwell, S.C., Rate Schedule.
- 6. "Building Construction Cost Data 1992", Robert Snow Means Company, Inc., Kingston, Massachusetts.
- 7. Tri-State Motor Transit Company, published tariffs, Interstate Commerce Commission (ICC) Docket No. MC-109397 and Supplements.
- T.S. LaGuardia et al., "Guidelines for Producing Commercial Nuclear Power Plant Decommissioning Cost Estimates", AIF/NESP-036, May 1986.
- 9. W.J. Manion and T.S. LaGuardia, "An Engineering Evaluation of Nuclear Power Reactor Decommissioning Alternatives", AIF/NESP-009, November 1976.
- W.J. Manion and T.S. LaGuardia, "Decommissioning Handbook", U.S. Department of Energy, DOE/EV/10128-1, November, 1980.
- Cost Engineers Notebook: American Association of Cost Engineers, AA-4.000, pg 3 of 22, Rev. 2 (January 1978) (Updated periodically).
- R.I. Smith, G.J. Konzek, W.E. Kennedy, Jr., "Technology, Safety and Costs of Decommissioning a Reference Pressurized Water Reactor Power Stalon," NUREG/CR-0130 and addenda, Pacific Northwest Laboratory for the Nuclear Regulatory Commission. June 1978.
- "Harvard Project Manager", Computer Software and User's Manual, Version 3.02, Software Publishing Corporation, Mountain View, California. January, 1991

9. REFERENCES (continued)

 U.S. Department of Transportation, Section 49 of the Code of Federal Regulations, "Transportation", Parts 173 through 178.

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APPENDIX A

UNIT COST FACTOR DEVELOPMENT

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1.0

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APPENDIX A

UNIT COST FACTOR DEVELOPMENT

Example: Unit Cost Factor for Removal of Contaminated Heat Exchanger < 3000 lbs.

1. SCOPE

Heat exchangers weighing < 3,000 lb will be removed in one piece using a cran small hoist. They will be disconnected from the inlet and outlet piping. The heat exchanger will be sent to the packing area.

2. CALCULATIONS

Act Activity ID Description	Act. Dur	Crt. Dur
a Mount pipe cutters	45	45
b Install contamination controls	20	(a)
c Disconnect inlet and outlet lines	60	60
d Cap openings	20	(c)
e Unbolt from mounts	30	30
f Remove contamination controls	15	(e)
g Rig for removal	30	(e)
h Remove, wrap in plastic, send to packing area	60	60
Totals (Activity/Critical)	280	/195
Duration adjustment(s): + Respiratory protection adjustment (50 % of critical duration)	98 68	
+ Radiation/ALARA adjustment (35 % of critical duration)	00	
Adjusted work duration	361	
+ Protective clothing adjustment (30 % of adjusted duration)	108	
Productive work duration	469	
+ Work break adjustment (8.33 % of productive duration)	39	
Total work duration	508 n	nin
*** Total duration = 8.467 hr ***		

****** Total duration = 8.467 hr *******

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UNIT CO	ST FACTOR DEV	ELOPMENT			
3. LABOR REQUIRED					
Crew	Number	Duration (hr)	Rate (\$/hr)	Cost	
Laborers Craftsmen Foreman	3.0 2.0 1.0	8.467 8.467 8.467	\$21.25 \$27.30 \$28.16	S	539.77 462.30 238.43
Subtotal labor cost Overhead & Profit on labor @ 27	7.89%				,240.50 345.98
Total labor cost				\$1,	,586.48
4. EQUIPMENT & CONSUMAB	LES COSTS				
Equipment Costs					none
Consumables/Materials Costs -Blotting paper 50 @ \$0.76/sq f -Plastic sheets/bags 50 @ \$0.06 -Gas torch consumables 1 @ \$6	/sq ft {3}			\$ \$ \$	38.00 3.00 6.87
Subtotal cost of equipment and a Overhead & profit on equipmen		15.000%		s s	47.87 7.18
Total costs, equipment & materi	al			s	55.05
TOTAL COST Removal of conta	minated heat exch	anger < 3000 j	pound:	\$1	,641.53
Total labor cost: Total equipment/material costs:		·			,586.48 55.05
Total adjusted exposure manhou Total craft labor manhours requ	rs incurred: ired per unit:	32.3 50.8			

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5. NOTES AND REFERENCES

- 1. Durations are shown in minutes. The integrated duration accounts for those activities that can be performed in conjunction with other activities, indicated by the alpha designator of the concurrent activity. This results in an overall decrease in the sequenced duration.
- Work difficulty factors were developed in conjunction with the AIF program to standardize decommissioning cost studies and are delineated in the "Guidelines" study (Ref. 7, p. 64).
- 3. Adjusted for regional material costs; for Springfield, MA
- 4. References:
 - 1. R.S. Means (1992) Division 016 Section 420-6360 pg 19
 - 2. McMaster-Carr Ed. 94 pg 735
 - 3. R.S. Means (1992) Division 015 Section 602-0200 pg 12

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APPENDIX B DECON UNIT COST FACTOR LISTING

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APPENDIX B DECON UNIT COST FACTOR LISTING

Unit Cost Factor	Site Value	VC Value
Removal of clean instrument and sampling tubing, \$/linear for	\$0.25	\$0.28
Removal of clean pipe 0.25 to 2 inches diameter \$/linear foot	\$1.49	\$1.49
Removal of clean pipe >2 to 4 inches diameter \$/linear foot	\$1.96	\$2.15
Removal of clean pipe >4 to 8 inches diameter \$/linear foot	\$8.65	\$9.37
Removal of clean pipe >8 to 14 inches diameter \$/linear foot	\$13.58	\$14.69
Removal of clean pipe >14 to 20 inches diameter \$/linear foo	t \$15.59	\$17.17
Removal of clean pipe >20 to 36 inches diameter \$/linear foc	st \$21.71	\$23.73
Removal of clean pipe >36 inches diameter \$/linear foot	\$29.61	\$32.30
Removal of clean valves >2 to 4 inches	\$22.27	\$24.09
Removal of clean valves >4 to 8 inches	\$86.47	\$93.67
Removal of clean valves >8 to 14 inches	\$135.77	\$146.94
Removal of clean valves > 14 to 20 inches	\$155.93	\$171.67
Removal of clean valves >20 to 36 inches	\$217.10	\$237.26
Removal of clean valves > 36 inches	\$296.15	\$323.03
Removal of clean pipe fittings >2 to 4 inches	\$23.54	\$25.25
Removal of clean pipe hangers for small bore piping	\$16.46	\$17.65
Removal of clean pipe hangers for large bore piping	\$60.97	\$66.22
Removal of clean pumps, <300 pound	\$118.17	\$118.17
Removal of clean pumps, 300-1000 pound	\$347.92	\$347.92
Removal of clean pumps, 1000-10,000 pound	\$1,344.14	\$1,464.62
Removal of clean pumps, > 10,000 pound	\$2,341.39	\$2,553.44
Removal of clean pump motors, 300-1000 pound	\$17.9.95	\$129.95
Removal of clean pump motors, 1000-10,000 pound	\$598.41	\$652.55
Removal of clean pump motors, >10,000 pound	\$1,283.79	\$1,400.02
Removal of clean turbine-driven pumps < 10,000 pounds	\$1,531.93	\$1,531.93

Site Value Unit cost factor used in the decontamination, decommissioning and dismantling of all structures and facilities exclusive of the Vapor Container.

VC Unit cost factors used in the decontamination, decommissioning and dismantling of the Vapor Container, incorporating higher work difficulty factors for access.

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APPENDIX B

Unit Cost Factor	Site Value	VC Value
Removal of clean turbine-driven pumps > 10,000 pounds	\$2,910.16	\$2,910.16
Removal of clean PWR turbine-generator	\$95,719.73	\$95,719.73
Removal of clean hea: exchanger <3000 pound	\$666.89	\$666.89
Removal of clean heat exchanger >3000 pound	\$1,961.52	\$1,961.52
Removal of clean feedwater heater/deaerator	\$4,193.38	\$4,571.81
Removal of clean moisture separator/reheater	\$9,905.65	\$10,783.31
Removal of clean PWR main condenser	\$247,043.80	\$268,247.91
Removal of PWR main steam generator	\$247,043.80	\$268,247.91
Removal of clean tanks, <300 gallons	\$207.76	\$207.76
Removal of clean tanks, 300-3000 gallons	\$454.91	\$496.00
Removal of clean tanks, >3000 gallons,		
\$/square foot surface area	\$4.43	\$4.80
Removal of clean electrical equipment, <300 pound	\$76.91	\$76.91
Removal of clean electrical equipment, 300-1000 pound	\$270.15	\$270.15
Removal of clean electrical equipment, 1000-10,000 pound	\$540.31	\$540 31
Removal of clean electrical equipment, > 10,000 pound	\$1,145.38	\$1,249.52
Removal of clean electrical transformers < 30 tons	\$867.77	\$867.77
Removal of clean electrical transformers > 30 tons	\$2,290.79	\$2,499.05
Removal of clean standby diesel-generator, <100 kW	\$811.72	\$886.35
Removal of clean standby diesel-generator,	¢1 012 02	£1 079 AS
100 kW to 1 MW	\$1,812.93	\$1,978.41
Removal of clean standby diesel-generator, >1 MW	\$3,753.84	\$4,095.72
Removal of clean electrical cable tray, \$/linear foot	\$6.53	\$7.10
Removal of clean electrical conduit, \$/linear foot	\$2.72	\$3.08
Removal of clean mechanical equipment, <300 pound	\$76.91	\$76.91
Removal of clean mechanical equipment, 300-1000 pound	\$270.15	\$270.15
Removal of clean mechanical equipment, 1000-10,000 pound	d \$540.31	\$540.31

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APPENDIX B

Unit Cost Factor	Site Value	VC Value
Removal of clean mechanical equipment, >10,000 pound	\$1,145.38	\$1,249.52
Removal of clean HVAC equipment, <300 pound	\$76.91	\$76.91
Removal of clean HVAC equipment, 300-1000 pound	\$270.15	\$270.15
Removal of clean HVAC equipment, 1000-10,000 pound	\$540.31	\$540.31
Removal of clean HVAC equipment, > 10,000 pound	\$1,145.38	\$1,249.52
Removal of clean HVAC ductwork, \$/pound Removal/manual flame cut of clean thin metal	\$0.57	\$0.57
components, \$/linear inch	\$2.83	\$3.06
Surface decontamination of equipment, \$/square foot	\$4.67	\$4.83
Decontamination of large components, \$/square foot	\$16.23	\$17.02
Decontamination rig look-up and flush	\$2,040.02	\$2,040.02
Chemical flush of components/systems, \$/gallon	\$4.36	\$4.36
Asbestos clean removal (pipe/components), \$/cubic foot Removal of contaminated instrument	\$4.67	\$4.83
and sampling tubing, \$/linear foot	\$0.43	\$0.46
Removal of contaminated pipe 0.25 to 2 inches diameter \$/linear foot	\$** : /	\$13.50
Removal of contaminated pipe >2 to 4 inches diameter \$/linear foot	\$23.79	\$24.86
Removal of contaminated pipe >4 to 8 inches diameter \$/linear foot	\$54.00	\$56.51
Removal of contaminated pipe >8 to 14 inches diameter \$/linear foot	\$102.47	\$106.95
Removal of contaminated pipe >14 to 20 inches diameter \$/linear foot	\$112.33	\$117.71
Ren.oval of contaminated pipe >20 to 36 inches diameter \$/linear foot	\$140.24	\$146.97
Removal of contaminated pipe >36 inches diameter \$/linear foot	\$182.51	\$191.01

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Unit Cost Factor	Site Value	VC Value
	#100 FO	6107.10
Removal of contaminated valves >2 to 4 inches	\$122.59	\$126.13
Removal of contaminated valves >4 to 8 inches	\$308.52	\$324.59
Removal of contaminated valves >8 to 14 inches	\$512.33	\$534.77
Removal of contaminated valves >14 to 20 inches	\$595.28	\$624.32
Removal of contaminated valves >20 to 36 inches	\$766.16	\$799.78
Removal of contaminated valves >36 inches	\$988.66	\$1,040.29
Removal of contaminated pipe hangers for small bore piping	\$38.55	\$40.93
Removal of contaminated pipe hangers for large bore piping	\$150.35	\$158.84
Removal of contaminated pumps, <300 pound	\$398.28	\$398.28
Removal of contaminated pumps, 300-1000 pound	\$989.64	\$989.64
Removal of contaminated pumps, 1000-10,000 pound	\$3,355.71	\$3,517.85
Removal of contaminated pumps, >10,000 pound	\$7,645.23	\$8,027.93
Removal of contaminated pumps, projector pound	\$414.29	\$414.29
Removal of contaminated pump motors, 1000-10,000 pound	\$1,477.90	\$1,552.74
Removal of contaminated pump motors, >10,000 pound	\$3,142.44	\$3,292.14
Removal of contaminated turbine-driven		
	\$3,942.88	\$3,942.88
pumps < 10,000 pound Removal of contaminated turbine-driven	40,778,000	
	\$7,420.65	\$7,420.65
pumps > 10,000 pound	\$1,641.53	\$1,641.53
Removal of contaminated heat exchanger <3000 pound	\$5,195.54	\$5,195.54
Removal of contaminated heat exchanger >3000 pound	623,115.13	\$654,691.88
Removal of PWR main steam generator	POder of a had a had	4004,002.00
Removal of contaminated tanks, <300 gallons	\$725.01	\$725.01
Removal of contaminated tanks, >300 gallons, \$/square foot	\$15.31	\$16.03
Removal of contaminated electrical equipment, <300 pound	\$273.45	\$273.45
Removal of contaminated electrical		
equipment, 300-1000 pound	\$706.04	\$706.04
Removal of contaminated electrical		
equipment, 1000-10,000 pound	\$1,317.24	\$1,317.24
- Buck wards and a second second		

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(continued)		
Unit Cost Factor	Site Value	VC Value
Removal of contaminated electrical		
	\$2,573.59	\$2,744.51
Removal of electrical transformers < 30 tons	\$955.89	\$955.89
	\$2,576.42	\$2,811.42
Removal of standby diesel-generator, <100 kW	\$907.83	\$990.48
	\$1,957.10	\$2,135.88
Removal of standby diesel-generator, >1 MW	\$4,231.87	\$4,616.34
Removal of contaminated electrical cable tray, \$/linear foot	\$24.70	\$25.98
Removal of contaminated electrical conduit, \$/linear foot	\$20.80	\$22.58
Removal of contaminated mechanical equipment, <300 pound Removal of contaminated mechanical	\$273.45	\$273.45
equipment, 300-1000 pound	\$706.04	\$706.04
Removal of contaminated mechanical		
equipment, 1000-10,000 pound Removal of contaminated mechanical	\$1,317.24	\$1,317.24
equipment, >10,000 pound	\$2,573.59	\$2,744.51
Removal of contaminated HVAC equipment, <300 pound	\$273.45	\$273.45
Removal of contaminated HVAC equipment, 300-1000 pound Removal of contaminated HVAC	\$706.04	\$706.04
equipment, 1000-10,000 pound	\$1,317.24	\$1,317.24
Removal of contaminated HVAC		
equipment, >10,000 pound	\$2,573.59	\$2,744.51
Removal of contaminated HVAC ductwork, \$/pound Removal/plasma arc cut of contaminated thin metal	\$1.89	\$1.89
components, \$/linear inch	\$1.76	\$1.86
Additional decontamination of surface by washing,\$/square fo Additional decontamination of surfaces by	ot \$4.67	\$4.83
hydrolasing, \$/square foot	\$16.23	17.02

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APPENDIX B

DECON UNIT COST FACTOR LISTING (continued)

(continued)		
Unit Cost Factor	Site Value	VC Value
Description sig book up and fluch	\$2,040.02	\$2,040.02
Decontamination rig hook-up and flush Chemical flush of components/systems, \$/gallon	\$4.36	\$4.36
Asbestos contaminated removal (pipe/components), \$/cubic f	and the second sec	\$4.83
Removal of standard reinforced concrete, \$/cubic yard	\$270.95	\$270.95
Removal of grade slab concrete, \$/cubic yard	\$145.94	\$145.94
Removal of clean concrete floors, \$/cubic yard	\$163.33	\$177.16
Removal of sections of clean concrete floors, \$/cubic yard	\$571.42	\$571.42
Removal of clean heavily reinforced concrete	\$124.23	\$135.15
w/#9 rebar, \$/cubic yard	\$ 1 W 1 1 W	
Removal of contaminated heavily reinforced concrete w/#9 rebar, \$/cubic yard	\$1,005.98	\$1,056.30
Removal of clean heavily rein concrete		
w/#18 rebar, \$/cubic yard	\$158.37	\$172.29
Removal of contaminated heavily rein concrete		A
w/#18 rebar, \$/cubic yard Removal heavily rein concrete w/#18 rebar & steel	\$1,334.21	\$1,402.15
embedments, \$/cubic yard	\$234.21	\$255.01
Removal of below grade suspended floors, \$/square foot	\$163.33	\$177.16
Removal of clean monolithic concrete structures, \$/cubic yar	d \$443.39	\$482.84
Removal of contaminated monolithic concrete structures, \$/cubic yard	\$1,002.85	\$1,053.16
Removal of clean foundation concrete, \$/cubic yard	\$377.86	\$377.86
Removal of contaminated foundation concrete, \$/cubic yard	\$934.81	\$981.76
Explosive demolition of bulk concrete, \$/cubic yard	\$18.35	\$18.35
Removal of wooden structures, \$/cubic foot	\$0.43	\$0.47
Removal of clean hollow masonry block wall, \$/cubic yard	\$51.51	\$51.51
Removal of contaminated hollow masonry		
block wall, \$/cubic yard	\$123.84	\$123.84
Removal of clean solid masonry block wall, \$/cubic yard	\$51.51	\$51.51
Removal of contaminated solid masonry block wall,	\$123.84	\$123.84
\$/cubic yard	\$5.22	\$5.22
Backfill of below grade voids, \$/cubic yard Placing entombment concrete, \$/cubic yard	\$256.40	\$256.40

- TLG ENGINEERING, INC. -

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(continued)	Class	NO
Unit Cost Factor	Site Value	VC Value
Removal of subterranean tunnels/voids, \$/linear foot	\$85.82	\$85.82
Placement of concrete for below grade voids, \$/cubic yard	\$74.77	\$74.77
Excavation of clean material, \$/cubic yard	\$2.48	\$2.48
Excavation of contaminated material, \$/cubic yard	\$5.95	\$5.95
Excavation of submerged concrete rubble, \$/cubic yard	\$8.30	\$8.30
Removal of clean concrete rubble, \$/cubic yard	\$8.01	\$8.01
Removal of contaminated concrete rubble, \$/cubic yard	\$19.13	\$19.13
Removal of building by volume, \$/cubic foot	\$0.18	\$0.18
Removal of clean building metal siding, \$/square foot	\$0.88	\$0.88
Removal of contaminated building metal siding, \$/square foot	\$2.12	\$2.12
Asbestos removal clean fireproofing/structural, \$/cubic foot	\$3.55	\$3.55
Asbestos removal (roofing), \$/cubic foot	\$3.61	\$3.61
Removal of standard asphalt roofing, \$/square foot	\$0.00	\$0.00
Removal of galbestos wall panels, \$/square foot	\$0.00	\$0.00
Removal of transite panels, \$/square foot	\$1.16	\$1.26
Placement of cofferdam, \$/linear foot	\$0.00	\$0.00
Scarifying contaminated concrete surfaces (drill & spall)	\$3.95	\$4.18
Scabbling contaminated concrete floors \$/square foot	\$2.49	\$2.70
Scabbling contaminated concrete walls \$/square foot	\$14.78	\$15.48
Scabbling contaminated ceilings \$/square foot	\$49.25	\$51.60
Scabbling structural steel \$/square foot	\$3.32	\$3.48
Removal of clean overhead cranes/monorails < 10 ton capacity	\$343.08	\$373.78
Removal of contaminated overhead	1000	4010.10
cranes/monorails < 10 ton capacity	\$789.78	\$828.09
Removal of clean overhead cranes/monorails > 10 - 50 ton capacity	\$822.33	\$897.08
Removal of contaminated overhead cranes/monorails > 10 - 50 ton capacity	\$1,895.73	\$1,991.66

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Unit Cost Factor	Site Value	VC Value
Removal of polar cranes > 50 ton capacity, each	\$3,329.99	\$3,631.40
Removal of gantry cranes > 50 ton capacity, each	\$12,789.03	\$13,952.99
Removal of structural steel, \$/pound	\$0.22	\$0.22
Removal of clean steel floor grating, \$/square foot	\$2.00	\$2.00
Removal of contaminated steel floor grating, \$/square foot	\$4.82	\$4.82
Removal of clean free-standing steel liner, \$/square foot Removal of contaminated free-standing steel	\$6.65	\$7.19
liner, \$/square foot	\$16.28	\$17.01
Removal of clean concrete anchored steel liner, \$/square for Removal of contaminated concrete anchored steel	ot \$3.33	\$3.60
liner, \$/square foot	\$18.86	\$19.77
Placement of scaffolding in clean areas, \$/square foot	\$3.09	\$3.09
Placement of scaffolding in contaminated areas, \$/square for	ot \$5.08	\$5.08
Landscaping with topsoil, \$/acre	\$14,810.03	\$14,810.03
Landscaping w/o topsoil, \$/acre	\$4,526.51	\$4,526.51
Cost of LSA box & preparation for use	\$967.66	\$967.66
Cost of LSA drum & preparation for use	\$125.49	\$125.49
Cost of cask liner for CNSI 14-195 cask	\$6,549.89	\$6,549.89
Cost of cask liner for CNSI 8-120A cask (resins)	\$9,278.44	\$9,278.44
Cost of cask liner for CNSI 8-120A cask (filters)	\$9,271.63	\$9,271.63
Decontamination of surfaces with vacuuming, \$/square foot	\$0.77	\$0.80

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Rev.Date	Page Description	Approval	
05/92	Original Issue	FWS	

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