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Ref. # 10CFR50.46

William J. Cahill, Jr.
Group Vice President

April 5, 1994

U. S. Nuclear Regulatory Commission (NRC)
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)
DOCKET NOS. 50-445 AND 50-446
ANNUAL REPORT OF CHANGES IN PEAK CLADDING TEMPERATURE

- REF: 1) TU Electric letter logged TXX-93169 from Mr. William J. Cahill, Jr., to the NRC dated April 5, 1993
- 2) TU Electric letter logged TXX-92175 from Mr. William J. Cahill, Jr., to the NRC dated April 7, 1992

Gentlemen:

In accordance with the requirements of 10CFR50.46(a)(3)(ii), TU Electric submits the attached changes or errors discovered in the Emergency Core Cooling System (ECCS) calculations of peak cladding temperature (PCT).

The last report of PCT changes, submitted in Reference 1, stated that TU Electric was awaiting NRC approval of the NOTRUMP evaluation model for the analysis of the small break Loss of Coolant Accident (LOCA) for Unit 1. The use of the NOTRUMP model would resolve the Small Break Loss of Coolant Accident Burst and Blockage issue and reduce the Unit 1 PCT value for small break LOCA below the 10CFR50.46 criterion of 2200 °F. In addition, the use of NOTRUMP would decrease the small break LOCA PCT value below the PCT calculated for large break LOCA, thereby re-establishing the large break LOCA as the limiting transient for Unit 1. The NRC approved the use of the NOTRUMP model for Unit 1 by letter dated April 26, 1993. The Unit 1 analysis of record for small break LOCA is now based on the NOTRUMP model. As a result the large break LOCA is once again the limiting transient.

The PCT for the Unit 1 large break LOCA based on the Westinghouse 1978 ECCS evaluation model was 2066 °F. However, beginning with the fourth fuel cycle for Unit 1 (December 1993), TU Electric analyzed the Unit 1 large break LOCA using NRC approved TU Electric methodology. The use of the TU Electric methodology establishes a new analysis of record for the Unit 1 large break LOCA.

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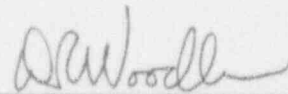
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Reference 2 provided a schedule of May 31, 1995, for the reanalysis of the Unit 2 large break LOCA. This schedule was consistent with the schedule for use of Siemens fuel (Cycle 3), based on an anticipated 12 month fuel cycle. Unit 2 is currently scheduled for 18 month fuel cycles. Consequently, the Unit 2 large break LOCA reanalysis, is now scheduled for May 31, 1996.

Attached are the changes to the CPSES Units 1 and 2 PCT values for the evaluation models used in the analyses of the limiting transient (large break LOCA).

Sincerely,

William J. Cahill, Jr.

By: 
D. R. Woodlan
Docket Licensing Manager

BSD
Attachment

c - Mr. L. J. Callan, Region IV
Mr. T. A. Bergman, NRR
Mr. L. A. Yandell, Region IV
Resident Inspectors, CPSES (2)

CPSES UNIT 1
PEAK CLADDING TEMPERATURE CHANGES (°F)

PREVIOUSLY REPORTED PCT VALUE	2066 ⁽¹⁾
ANALYSIS OF RECORD PCT (TU ELECTRIC METHODOLOGY)	2003 ⁽²⁾
ECCS MODEL ASSESSMENTS	N/A
LICENSING BASIS PCT + MARGIN ALLOCATIONS	2003

(1) Based on the Westinghouse 1978 ECCS evaluation model

(2) Based on TU Electric large break LOCA analysis methodology

CPSES UNIT 2
PEAK CLADDING TEMPERATURE CHANGES (°F)

PREVIOUSLY REPORT PCT	1915
ANALYSIS OF RECORD (WESTINGHOUSE METHODOLOGY)	1804
PREVIOUSLY REPORTED ECCS MODEL ASSESSMENTS	111 ⁽¹⁾
CURRENT MODEL ECCS ASSESSMENTS	-6
LUCIFER CODE ERROR CORRECTION	-6
LICENSING BASIS PCT + MARGIN ALLOCATIONS	1909

(1) Reanalysis of the Unit 2 large break LOCA will be performed by May 31, 1996 (Cycle 3).