

October 3, 1990 LD-90-075

Project No. 675

Mr. Thomas V. Wambach, Project Manager Division of Reactor Projects III/IV/V and Special Projects Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D. C. 20555

Subject:

Combustion Engineering Standard Safety Analysis

Report - Design Certification, Amendment H

Reference:

Letter LD-89-035, A. E. Scherer (C-E) to T. E.

Murley (NRC), dated March 30, 1989

Dear Mr. Wambach:

This letter transmits an advance copy of Amendment H to the Combustion Engineering Standard Safety Analysis Report - Design Certification (CESSAR-DC). Our application for design certification was submitted via the reference letter.

For your convenience, we have summarized the contents of Amendment H in Enclosure I. This copy of Amendment H (Enclosure II) is provided for your information and planning purposes. The formal amendment is currently being printed and, together with the affidavit required by 10 CFR 50.4(b) and 10 CFR 50.30(b), will be submitted in approximately one month.

ABB Combustion Engineering Nuclear Power

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Telephone (203) 686-1911 Fax (203) 285-9512 Telex 99297 COMBEN WSOR Mr. Thomas V. Wambach LD-90-075 Page 2 October 3, 1990 If you have any questions, please feel free to call me or Mr. S. E. Ritterbusch of my staff at (203) 285-5206. Sincerely yours, COMBUSTION ENGINEERING, INC. E. H. Kennedy Manager Nuclear Systems Licensing EHK: 1w Enclosures: As Stated CC: C. L. Miller (NRC) w/o encl.
T. E. Murley (NRC) w/o encl.
F. W. Ross (DOE - Germantown)
G. L. Vine (EPRI)

SUMMARY OF CESSAR-DC, AMENDMENT H

Chapter 1 (General Arrangements

The reactor building arrangement drawings of Section 1.2 have been replaced with general arrangement drawings for the site layout, reactor building, and the reactor building nuclear annex.

Chapter 2 (Site Characteristics)

- Restrictions on site selection have been added to preclude certain external events (e.g., aircraft hazards).
- Section 2.3 has been modified to include the site envelope for radiological release dispersion (i.e., X/Qs).

Chapter 6 (Engineered Safety Features

- The small break LOCA and long-term cooling analyses have been added to Section 6.3.
- Inservice inspection guidelines have been provided in Section 6.6.

Chapter 14 (Startup Testing)

The startup test program has been expanded to cover the "balance-of-plant". Some changes have also been made to the test program for the nuclear steam supply system.

Chapter 15 (Safety Analysis)

The design-basis safety analyses have been provided.

Appendix B (Probabilistic Risk Assessment

Level II and III PRA results have been added. A concise summary of the PRA will be provided in the next amendment to CESSAR-DC (December 1990).