

NUREG/CR-6194  
TMI V(92)EG01  
EGG-2731

---

---

# Metallographic and Hardness Examinations of TMI-2 Lower Pressure Vessel Head Samples

---

---

Prepared by G. E. Korth

Idaho National Engineering Laboratory  
EG&G Idaho, Inc.

Prepared for  
U.S. Nuclear Regulatory Commission

9404110355 940331  
PDR ADOCK 05000320  
P PDR

## AVAILABILITY NOTICE

### Availability of Reference Materials Cited in NRC Publications

Most documents cited in NRC publications will be available from one of the following sources:

1. The NRC Public Document Room, 2120 L Street, NW., Lower Level, Washington, DC 20555-0001
2. The Superintendent of Documents, U.S. Government Printing Office, Mail Stop SSOP, Washington, DC 20402-9328
3. The National Technical Information Service, Springfield, VA 22161

Although the listing that follows represents the majority of documents cited in NRC publications, it is not intended to be exhaustive.

Referenced documents available for inspection and copying for a fee from the NRC Public Document Room include NRC correspondence and internal NRC memoranda, NRC bulletins, circulars, information notices, inspection and investigation notices, licensee event reports, vendor reports and correspondence, Commission papers, and applicant and licensee documents and correspondence.

The following documents in the NUREG series are available for purchase from the GPO Sales Program: formal NRC staff and contractor reports, NRC-sponsored conference proceedings, international agreement reports, grant publications, and NRC booklets and brochures. Also available are regulatory guides, NRC regulations in the *Code of Federal Regulations*, and *Nuclear Regulatory Commission Issuances*.

Documents available from the National Technical Information Service include NUREG-series reports and technical reports prepared by other Federal agencies and reports prepared by the Atomic Energy Commission, forerunner agency to the Nuclear Regulatory Commission.

Documents available from public and special technical libraries include all open literature items, such as books, journal articles, and transactions. *Federal Register* notices, Federal and State legislation, and congressional reports can usually be obtained from these libraries.

Documents such as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings are available for purchase from the organization sponsoring the publication cited.

Single copies of NRC draft reports are available free, to the extent of supply, upon written request to the Office of Administration, Distribution and Mail Services Section, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at the NRC Library, 7920 Norfolk Avenue, Bethesda, Maryland, for use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from the American National Standards Institute, 1430 Broadway, New York, NY 10018.

## DISCLAIMER NOTICE

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, or any of their employees, makes any warranty, expressed or implied, or assumes any legal liability of responsibility for any third party's use, or the results of such use, of any information, apparatus, product, or process disclosed in this report, or represents that its use by such third party would not infringe privately owned rights.

NUREG/CR-6194  
TMI V(92)EG01  
EGG-2731

---

---

# Metallographic and Hardness Examinations of TMI-2 Lower Pressure Vessel Head Samples

---

---

Manuscript Completed: February 1994  
Date Published: March 1994

Prepared by  
G. E. Korth

Idaho National Engineering Laboratory  
Managed by the U.S. Department of Energy

EG&G Idaho, Inc.  
Idaho Falls, ID 83415

Prepared for  
Division of Systems Research  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001  
NRC FIN L1004  
Under DOE Contract No. DE-AC07-76ID01570

## ABSTRACT

Fifteen steel samples were removed from the lower pressure vessel head of the damaged TMI-2 nuclear reactor to assess the thermal threat to the head posed by 15 to 20 metric tons of molten core debris relocating there during the accident. Full sections of thirteen of the samples and partial sections of the other two samples underwent hardness and metallographic examinations at the Idaho National Engineering Laboratory. These examinations have shown that eleven of the fifteen samples did not exceed the ferrite-austenite transformation temperature of 727°C during the accident. The remaining four samples did show evidence of having a much more severe thermal history. The samples from core grid positions F-10 and G-8 are believed to have experienced temperatures of 1,040 to 1,060°C for about 30 minutes. Samples from positions E-8 and E-6 appear to have been subjected to 1,075 to 1,100°C for approximately 30 minutes.

## CONTENTS

ABSTRACT .....	iii
LIST OF FIGURES .....	vi
LIST OF TABLES .....	vi
EXECUTIVE SUMMARY .....	vii
FOREWORD .....	ix
ACKNOWLEDGMENTS .....	xi
INTRODUCTION .....	1
MATERIAL .....	2
SAMPLE EXAMINATIONS .....	4
RESULTS AND DISCUSSION .....	5
Hardness Measurements .....	5
Microstructure .....	8
Comparison with Standards .....	11
CONCLUSIONS .....	17
REFERENCES .....	18
Appendix A—Microstructure and Hardness Profiles of All TMI-2 Metallurgical Samples and Midland Archive Material .....	A-1
Appendix B—Microstructure of First Series of Midland Archive Samples Given Accident- Simulated Heat Treatments .....	B-1
Appendix C—Microstructure of TMI-2 Samples F-10(M-3), E-8(M-3), G-8 (408P-3), E-6 (402A-1), and Accident-Simulated Heat Treated Slices of Samples H-4(M-3) and M-11(M-3) .....	C-1

## LIST OF FIGURES

1.	Schematic showing the source of the TMI-2 metallographic samples. . . . .	2
2.	Location of lower head boat samples with respect to core positions . . . . .	3
3.	Composite hardness profiles of lower pressure vessel head metallographic samples. Data for eight samples are shown in upper figure; data for remaining seven samples plus Midland archive material are shown in lower figure . . . . .	6
4.	Longitudinal hardness profiles taken from the H-8 (x-series) strips . . . . .	7
5.	Final hardness of Samples E-8(m-3) and F-10(m-3) compared to cooling rate effects/ $T_{max}$ curves for the final hardness of Midland archive material . . . . .	8
6.	Stainless steel/low alloy steel interface of Samples F-10(m-3) (a) and E-8(m-3) (b) illustrating the band of carbon diffusion into the stainless steel . . . . .	9
7.	SEM micrographs of the carbon diffusion band [Sample F-10(m-3)] . . . . .	10
8.	Calculated time/temperature/distance relationship of carbon diffusion into austenite . . . . .	11
9.	Thermal histories of the 12 Midland archive standards (first series) . . . . .	12
10.	TEM micrographs showing the presence of carbides at the austenite-ferrite interface in the stainless steel weld cladding . . . . .	13
11.	Diagram of time/temperature observations of A533B pressure vessel steel clad with Type 308L stainless steel . . . . .	16

## LIST OF TABLES

1.	Sample identification and heat treatments given the slices from H-4(m-3) and M-11(m-3) samples . . . . .	14
----	---	----

## EXECUTIVE SUMMARY

Fifteen steel samples were removed by metal disintegration machining from the lower head of the TMI-2 reactor pressure vessel for determination of mechanical properties and metallurgical condition following the TMI-2 accident, in which 15 to 20 metric tons of molten core debris relocated onto the lower head. The samples were triangular in shape with the apex penetrating approximately 50 mm into the 141 mm thick lower head. The objective of the investigation was to learn, to the extent possible, the thermal history of the lower head and to determine the post-accident properties of the A533B pressure vessel low alloy steel so that a margin to failure assessment can be performed.

To accomplish this task, an OECD TMI-2 VIP program (Organization for Economic Co-operation and Development Three Mile Island-2 Vessel Investigation Project) was formed by the Nuclear Regulatory Commission (NRC) with the United States and nine European countries and Japan as the participants. Argonne National Laboratory (ANL) was given the responsibility, by NRC, of receiving and decontaminating the triangular-shaped "boat samples," sectioning them into mechanical property and metallurgical specimens, and shipping the finished test specimens to the OECD partners for testing. ANL also provided considerable background information by performing various tests and examinations on the Midland archive material (A533B steel from the lower head of an abandoned reactor that had an almost identical fabrication history).

Full cross sections, including the stainless steel weld cladding, from thirteen of the boat samples were sent to the Idaho National Engineering Laboratory (INEL) for hardness and metallographic examinations for the purpose of mapping the thermal history of the lower head. Only partial sections of the other two samples (low alloy steel only, without the stainless steel weld cladding) were received at INEL and therefore only the A533B steel was examined for these two. (Full cross sections of these two samples, containing the stainless steel cladding, were not able to be decontaminated but hot cell micrographs of the interface area were provided by ANL.) This report gives the results of the examinations performed by INEL on the thirteen full section and two partial section metallographic samples.

Only four of the fifteen samples examined at INEL showed evidence of thermal exposures during the accident exceeding the ferrite-austenite transformation temperature of 727°C. Sample F-10(m-3) is believed to have experienced temperatures of 1,040 to 1,060°C for about 30 minutes, and Sample E-8(m-3) appears to have experienced 1,075 to 1,100°C for 30 minutes. Limited examination of Samples G-8(m-1) and E-6(m-1) at the INEL and micrographs provided by ANL of G-8(408P-3) and E-6(402A-1) showed that G-8 had received a thermal exposure similar to F 10(m-3) and E-6 a thermal exposure similar to E-8(m-3). The evidence for these thermal histories was obtained at locations within the sample very near to the weld cladding/low alloy steel interface. At a depth of 50 mm into the vessel wall, as measured from the inside surface (45 mm from the weld cladding/low alloy steel interface), the temperature was determined to be approximately 100°C lower. The other eleven samples appear to be metallurgically in the as-fabricated condition, which means their peak temperature during the accident was less than the transformation temperature of 727°C, the lowest temperature for which microstructure modifications could be observed.

## FOREWORD

The contents of this report were developed as part of the Three Mile Island Unit 2 Vessel Investigation Project. This project is jointly sponsored by eleven countries under the auspices of the Nuclear Energy Agency of the Organization for Economic Cooperation and Development. The twelve sponsoring organizations are:

- \* The Centre d'Etudes d'Energie Nucléaires of Belgium,
- \* The Säteilyturvakeskus of Finland,
- \* The Institute de Protection et de Sûreté Nucléaire of the Commissariat à l'Energie Atomique of France,
- \* The Gesellschaft für Reaktorsicherheit mbH of Germany,
- \* The Comitato Nazionale per La Ricerca e per Lo Sviluppo Dell' Energia Nucleare e Delle Energie Alternative of Italy,
- \* The Japan Atomic Energy Research Institute,
- \* The Consejo de Seguridad Nuclear of Spain,
- \* The Statens Kärnkraftinspektion of Sweden,
- \* The Office Fédéral de l'Energie of Switzerland,
- \* AEA Technology of the United Kingdom,
- \* The United States Nuclear Regulatory Commission, and
- \* The Electric Power Research Institute.

The primary objectives of the Nuclear Energy Agency (NEA) are to promote cooperation between its Member governments on the safety and regulatory aspects of nuclear development, and on assessing the future role of nuclear energy as a contributor to economic progress.

This is achieved by:

- encouraging harmonisation of governments' regulatory policies and practices in the nuclear field, with particular reference to the safety of nuclear installations, protection of man against ionising radiation and preservation of the environment, radioactive waste management, and nuclear third party liability and insurance;
- keeping under review the technical and economic characteristics of nuclear power growth and of the nuclear fuel cycle, and assessing demand and supply for the different phases of the nuclear fuel cycle and the potential future contribution of nuclear power to overall energy demand;
- developing exchanges of scientific and technical information on nuclear energy, particularly through participation in common services;
- setting up international research and development programmes and undertakings jointly organized and operated by OECD countries.

In these and related tasks, NEA works in close collaboration with the International Atomic Energy Agency in Vienna, with which it has concluded a Cooperation Agreement, as well as with other international organizations in the nuclear field.



## ACKNOWLEDGMENTS

The author gratefully acknowledges the contribution of G. L. Fletcher, who performed the metallography and hardness measurements. Appreciation is extended to R. N. Wright, P. Kuan, and M. L. Carboneau of EG&G Idaho for their technical review of the manuscript. Appreciation is also extended to D. R. Diercks of ANL for his technical review of the manuscript and providing the additional hot cell photomicrographs of sections of E-6 and G-8.

The author acknowledges the support for this work provided by the Organization for Economic Co-operation and Development Three Mile Island-2 Vessel Investigation Project and the Nuclear Regulatory Commission through Department of Energy Idaho Operations Office Contract DE-AC07-76ID01570.

# Metallographic and Hardness Examinations of TMI-2 Lower Pressure Vessel Head Samples

## INTRODUCTION

During the TMI-2 accident, 15 to 20 metric tons of molten core debris relocated onto the lower pressure vessel head of the reactor, causing a considerable threat to the integrity of the vessel.<sup>1</sup> The temperature of the molten debris is believed to have been of the order of 2,530°C (2,800 K), and therefore it had the potential of melting or considerably weakening the lower head, which is comprised of 136 mm thick A533B pressure vessel steel clad with 5-mm Type 308L stainless steel. The lower head did not melt or fail in high temperature creep, but contained the debris. This indicates that the steel's temperature was considerably below its melting temperature of 1,515°C, though the temperature may have been well within the regime where failure by short term creep could have occurred. Samples were removed from the lower head for examination of the post-accident condition of the steel. Mechanical properties are to be determined and metallographic examinations performed on the samples. The objective of the investigation reported in this document is to determine by metallurgical methods, to the extent possible, the thermal history, especially the peak temperatures reached, of the lower pressure vessel head during the accident so that an assessment of the margin to failure can be performed. Methods of examination used included, but were not limited to, (1) hardness profiles, (2) general microstructure examinations, and (3) interface reactions between the A533B steel and the stainless steel cladding.

## MATERIAL

Slices were taken from each of the "boat samples" that had been removed from the lower head by a metal disintegration machining process. Figure 1 is a schematic showing the relationship of the samples to the lower head. The samples were identified by the core grid position directly above their position on the lower head; the locations and orientations of the samples are shown in Figure 2. From the fifteen boat samples, thirteen full section and two partial section metallography samples were received at the INEL. The metallurgical samples used in this investigation are identified as follows:

### Full Section

D-10(m-2)	E-8(m-3)	E-11(m-3)	F-5(m-3)
F-10(m-3)	H-4(m-3)	H-5(m-2)	H-8(m-2)
K-7(m-3)	K-13(m-3)	L-9(m-3)	M-8(m-3)
M-11(m-3)			

### Partial Section (low alloy steel only)

E-6(m-1)	G-8(m-1)	H-8(x-series) [longitudinal strips]
----------	----------	-------------------------------------

The number in parentheses following the boat sample identification designates the section within the boat (see Appendix A for sectioning details). Boat samples E-6 and G-8 had surface cracks in the stainless steel cladding that contained core debris and decontamination of full cross sections of these two samples with the attached cladding was unsuccessful. Therefore, partial sections of the low alloy steel from these samples were examined at INEL, as well as micrographs taken by ANL during hot cell metallography of E-6 and G-8 sections. The H-8(x-series) samples

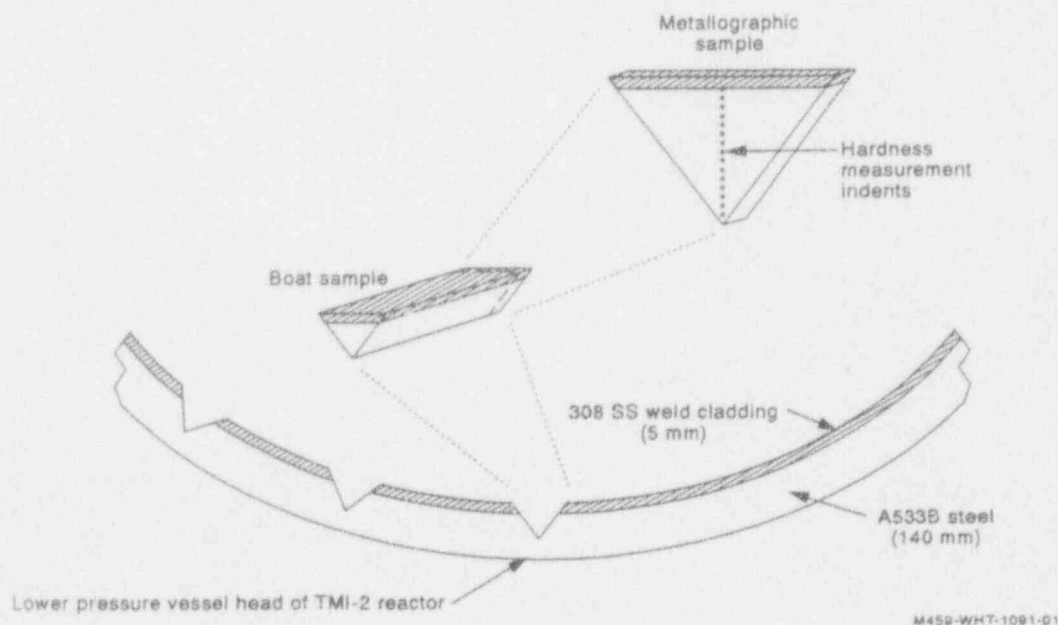
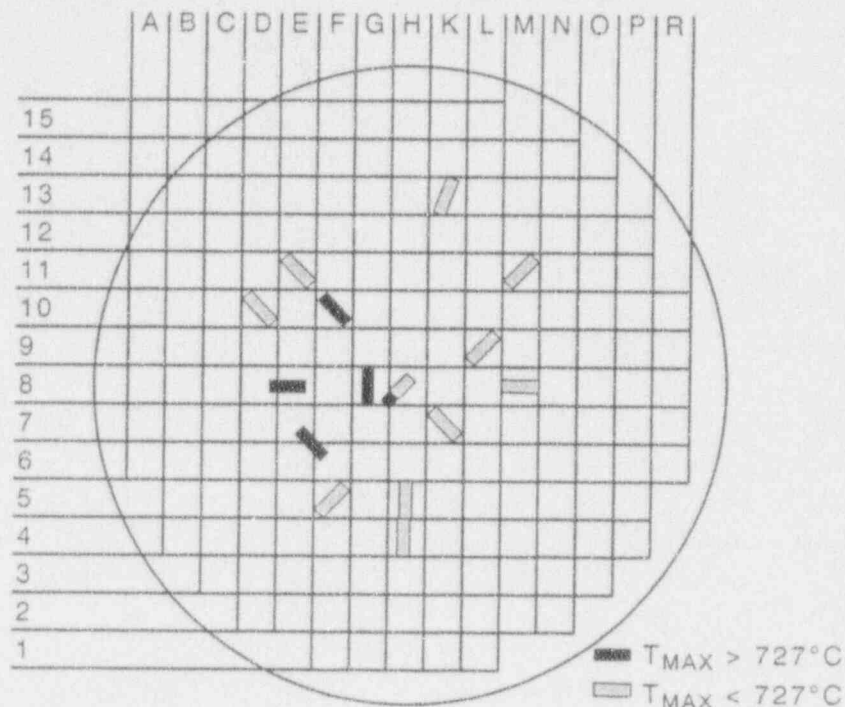


Figure 1. Schematic showing the source of the TMI-2 metallographic samples.



U91 0175

**Figure 2.** Location of lower head boat samples with respect to core positions.

included longitudinal scrap pieces left over after the H-8 boat sample was sectioned into mechanical property test specimen blanks. These strips were from the end of the boat sample closest to G-8, whereas the H-8(m-2) metallurgical sample was a full cross section taken across the nozzle penetration on the opposite end.

When the samples were received at the INEL, some of them were still slightly radioactive, even though all surfaces had been machined after the slicing. The  $\beta$ - $\gamma$  activity at contact ranged from  $<100$  (background level) to 9,000 counts/minute. The primary activity was due to  $Co^{60}$  and  $Cs^{137}$ , but some  $Sb^{125}$  was also observed on two samples. This radioactivity was from surface contamination, primarily in the weld clad area, and was removed by a combination of acid etching and abrasive grinding. All samples, except the two mentioned above, were eventually successfully decontaminated to  $<100$  counts/minute so that they could be handled in the "cold" metallurgical laboratory.

## SAMPLE EXAMINATIONS

Three different methods of examination were used to assess the thermal history of the samples: (1) hardness, (2) microstructure, and (3) metallurgical reactions at the weld clad interface. Hardness profiles were taken of the samples from the weld cladding to the bottom tip of the triangular piece (see Figure 1). Also, to obtain better resolution without having the hardness indents too close to each other, diagonal traces were taken across part of the weld cladding to a depth of 10 to 12 mm from the interface into the A533B steel. All the hardness measurements were taken using the Rockwell B indenter and then converted to DPH values.

Microstructure was examined using standard metallographic practices. Micrographs were taken at several different magnifications of the area from the weld clad interface to a depth in the A533B steel where the heat effects from the weld cladding operation are no longer seen. The optical metallography and hardness profiles of the thirteen full section TMI-2 samples, the two partial section samples, and the Midland archive material<sup>a</sup> are contained in Appendix A.

The third method of examination involved a closer look at any possible metallurgical reactions at the weld cladding/low alloy steel interface. Optical metallography, microhardness, and limited electron microscopy were all used in this investigation.

---

a. The Midland archive material, also A533B steel, was taken from the lower pressure vessel head of the Midland, Michigan reactor built by the same vendor as the TMI-2 reactor and has an almost identical processing history. (See Reference 2 for more details of the Midland archive material and its characterization.)

## RESULTS AND DISCUSSION

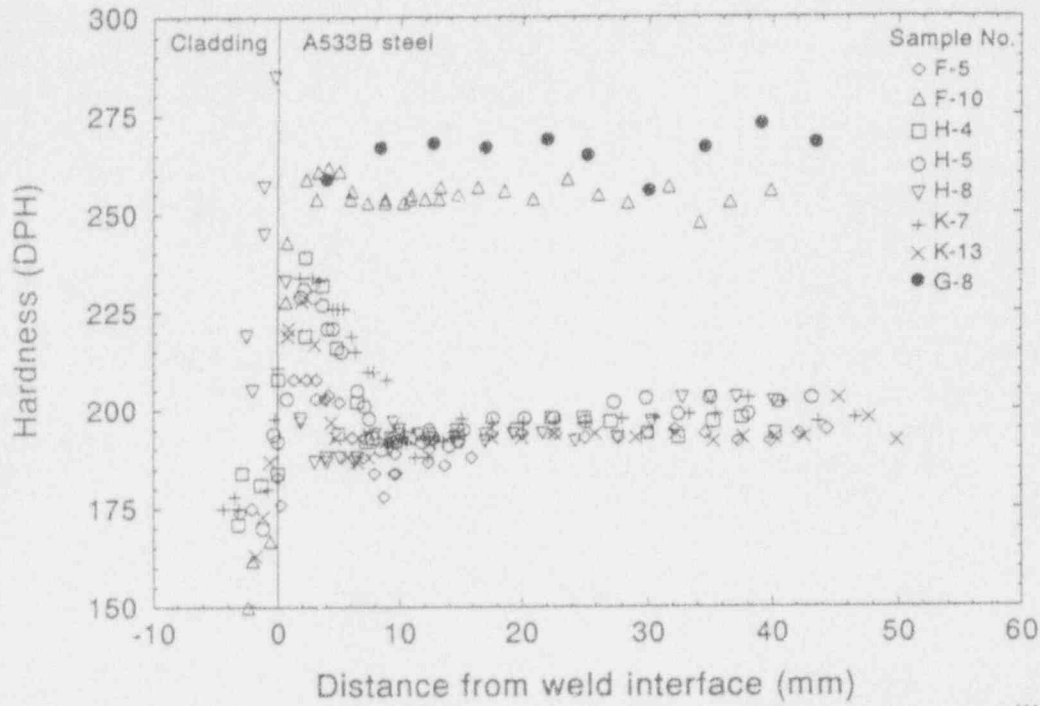
The majority of the samples, including the Midland archive material, exhibited a band 2 to 3 mm below the weld clad interface and 5 to 8 mm wide that could be seen with the naked eye on a polished and etched sample. This band is believed to be due to heat effects from the welding operation. Metallography shows the band to have a very fine-grained structure, and hardness profiles show a marked increase in hardness within the band. Thermal effects from the weld cladding operation would have heated the parent metal to above the ferrite-austenite transformation temperature of 727°C to some depth. The metal would have then been quenched due to the large mass of the lower head plate. This austenitizing and quenching can result in grain refinement and undoubtedly explains the hardening in the band, which could be a Hall-Petch effect from grain refinement, a martensitic transformation, or both. The only full section samples that did not show this band were E-8(m-3) and F-10(m-3), which were shown by hardness measurements to have exceeded the transformation temperature during the accident as will be discussed below.

### Hardness Measurements

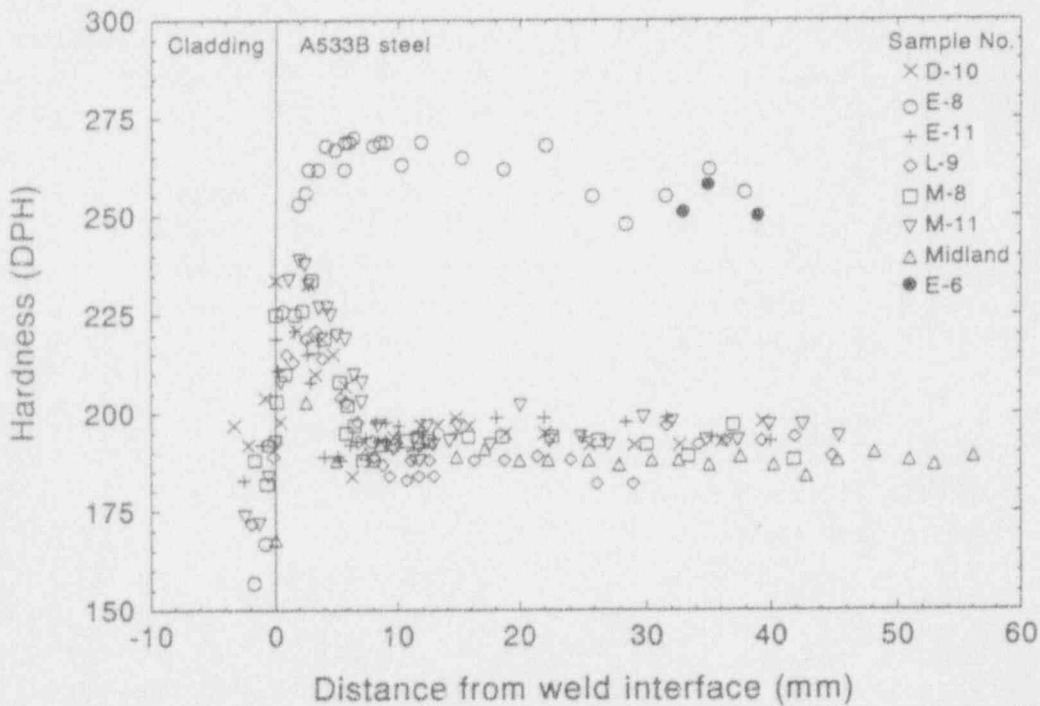
Figure 3 shows the hardness profiles of all the samples. Samples E-8(m-3) and F-10(m-3) have a markedly different hardness profile than the other samples - the characteristic hardness peak in the band with a subsequent drop to as-fabricated levels has changed to a sharp rise to much higher levels that are sustained throughout the full sample depth. Heat-affected bands from the weld cladding are not evident in these two samples, but have been completely eliminated by the thermal effects of the accident. Although a full depth profile is not available for E-6(m-1) or G-8(m-1), their hardness values are plotted in Figure 3 at the approximate location with respect to the weld clad interface. The hardnesses of these two samples are similar to those of F-10(m-3) and E-8(m-3), indicating that they too had exceeded the transformation temperature.

The hardnesses of the H-8(x-series) strips were measured in a longitudinal direction on the several pieces that were large enough to obtain a good reading. The results of these measurements are shown in Figure 4. A hardness increase is evident as the end closest to G-8 is approached. This observation indicates that the ferrite/austenite transformation temperature was reached on the end of H-8 nearest to G-8.

The final hardness of the TMI-2 samples is a strong indicator that the A533B steel transformation temperature of 727°C (1,000 K) was exceeded during the accident, and the discussion to follow shows that the cooling rate back through the phase change was  $\geq 10^\circ\text{C}/\text{minute}$ . Figure 5 compares the final hardnesses of Samples E-8(m-3), F-10(m-3), G-8(m-1), and E-6(m-1) with the results of the cooling rate studies of the Midland archive material. Assuming the Midland material is representative of the TMI-2 lower head material, this figure shows that if the cooling rate had been in the vicinity of  $1^\circ\text{C}/\text{minute}$  or less, then the final hardness would have been approximately the same as that of the as-fabricated parent metal. Therefore, hardness measurements would not have been very helpful in determining the thermal history due to the accident - they would only reveal that the hardness peak from the heat-affected band from the weld cladding was eliminated. However, the final hardness values for E-8(m-3), F-10(m-3), G-8(m-1), and E-6(m-1) are consistent with cooling rates of  $\geq 10^\circ\text{C}/\text{minute}$  and any peak temperature



M459-WHT-1091-02



M459-WHT-1091-03

**Figure 3.** Composite hardness profiles of lower pressure vessel head metallographic samples. Data for eight samples are shown in upper figure; data for remaining seven samples plus Midland archive material are shown in lower figure.

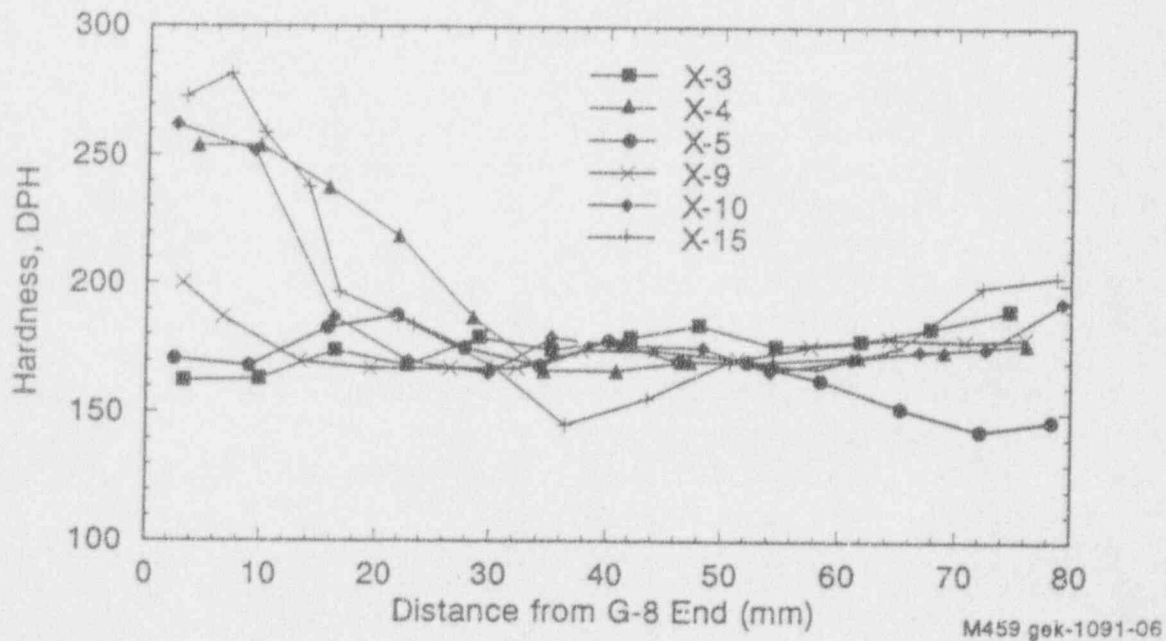
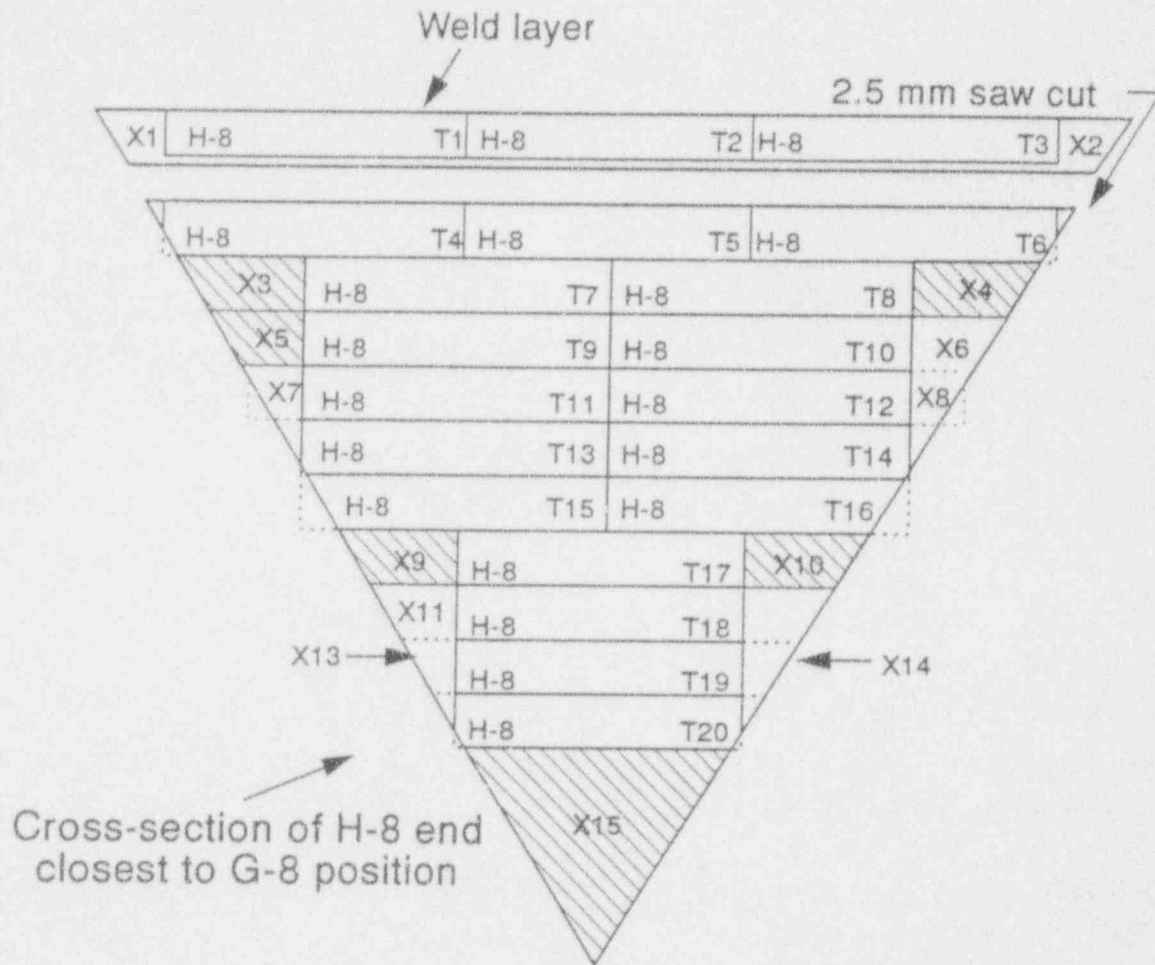
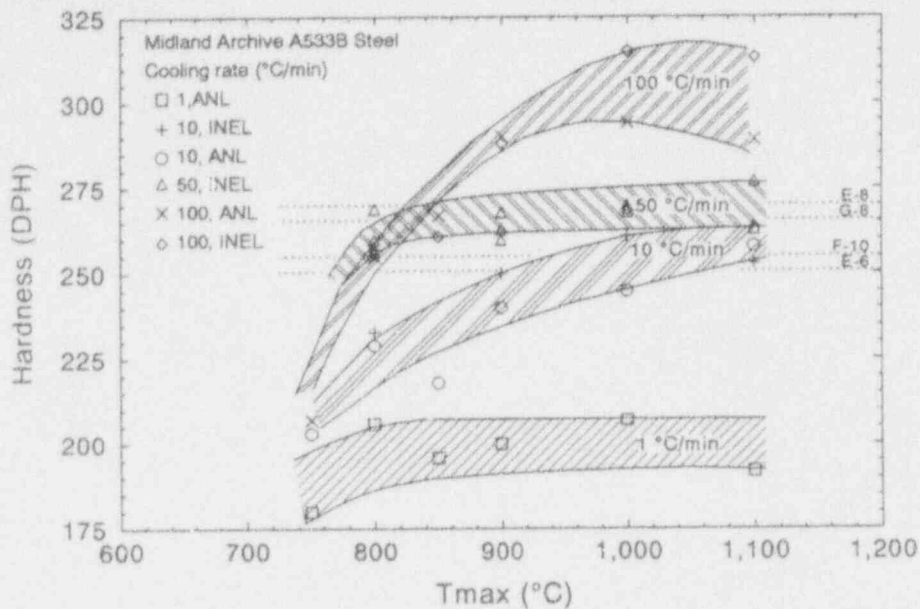


Figure 4. Longitudinal hardness profiles taken from the H-8 (x-series) strips.





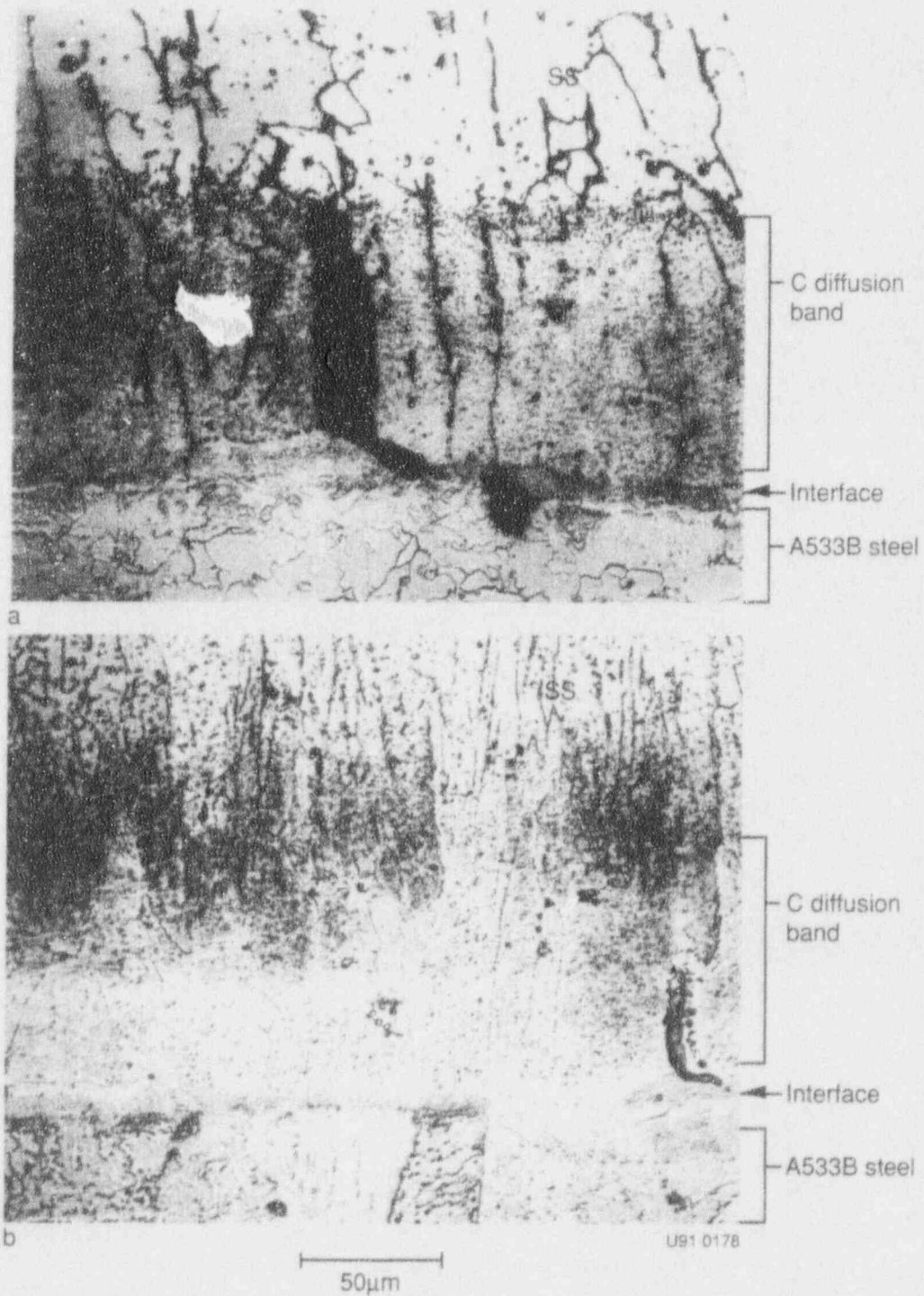
**Figure 5.** Final hardness of Samples E-8(m-3) and F-10(m-3) compared to cooling rate effects/ $T_{\max}$  curves for the final hardness of Midland archive material.

from 800 to 1,100°C (1,073 to 1,373 K). Therefore, hardness values of the TMI-2 samples are indicative of two things: (1) whether or not the material had exceeded the transformation temperature, and (2) if it had, some bounds on the cooling rate. However, hardness values are not very conclusive as to the peak temperatures that may have been reached. Other methods were explored to assess peak temperatures.

## Microstructure

Other indicators that assisted in determining the thermal history of the lower head during the accident include the general microstructure, which would show evidence of grain growth while in the austenitic phase, and heat-induced metallurgical reactions that may have occurred in the stainless steel or at the A533B steel/weld cladding interface. Even though these indicators are metallurgical phenomena for which time and temperature are interrelated, the determination of boundaries is possible. Also, by using several approaches the probability of converging on the thermal history was much greater.

Carbon diffusion from the pressure vessel steel (0.2% C) into the stainless steel (0.03% C) is evident, and this phenomenon is another possible indicator of thermal history. Figure 6 shows the band of carbon diffusion into the weld cladding for Samples F-10(m-3) and E-8(m-3), known heat-affected samples from the accident. The 0.10 to 0.15 mm band in the stainless steel at the interface has been determined by scanning electron microscopy (SEM) microchemical analysis to be due to carbon diffusion. Figure 7 shows more details of the band of F-10(m-3). Microhardness measurements reveal that the band is very hard (up to 500 DPH) and microcracks, seen in Figures 6a and 7, show that the band is brittle. Although TMI-2 samples believed to not have been affected by the accident exhibited some evidence of carbon diffusion (by microhardness



**Figure 6.** Stainless steel/low alloy steel interface of Samples F-10(m-3) (a) and E-8(m-3) (b) illustrating the band of carbon diffusion into the stainless steel.

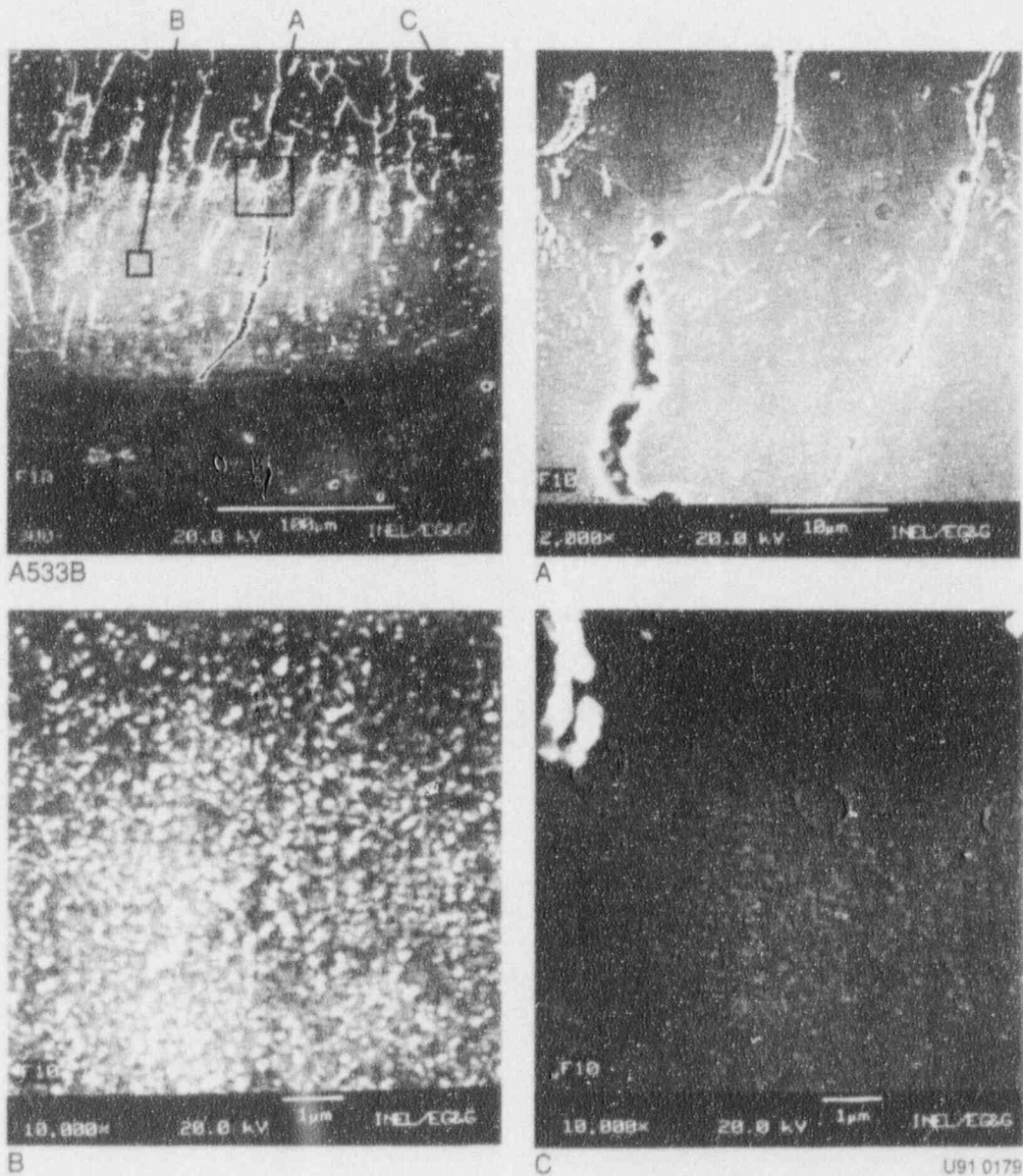


Figure 7. SEM micrographs of the carbon diffusion band [Sample F-10(m-3)].

measurements only, not revealed by metallography) into the stainless steel from the welding operation, it is not so prominent nor as deep as observed in F-10(m-3) and E-8(m-3). Figure 8 illustrates the time/temperature/distance relationship of carbon diffusion into austenite based on theoretical calculations using diffusion coefficients found in the literature.<sup>3</sup> This figure shows that the 0.10 to 0.15 mm diffusion distance observed on Samples F-10(m-3) and E-8(m-3) could have resulted from conditions ranging from 2 minutes at 1,100°C to 90 minutes at 800°C. Thus, the carbon diffusion distance by itself is not conclusive in determining peak temperatures of the lower head, but it is of value in confirming other indications.

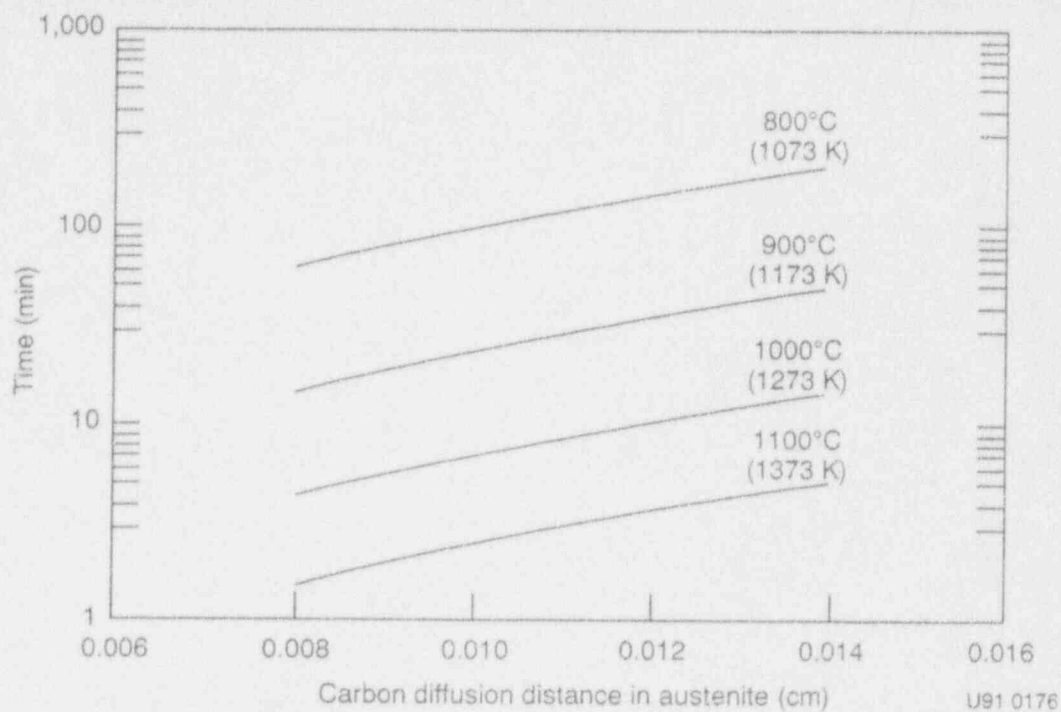


Figure 8. Calculated time/temperature/distance relationship of carbon diffusion into austenite.

### Comparison with Standards

To provide a basis for comparison with the TMI-2 samples, Midland archive standards with known thermal histories were prepared. By making the best possible match between the standards and the TMI-2 samples of the combination of hardness, microstructure, carbon diffusion distance, and any interface reactions, an estimate of the TMI-2 sample thermal history was made. The first series of Midland archive standards was prepared by resistively heating flat bars (3 x 25 x 80 mm) in the Gleeble machine to the thermal histories shown in Figure 9. Initially, twelve standards were prepared with  $T_{max}$  values of 800, 900, 1,000, and 1,100°C and dwell times of 1, 10, and 100 minutes. The heating rate of 40°C/minute was chosen arbitrarily, but the cooling rate of 50°C/minute was selected because it produced a final hardness similar to that observed in Samples F-10(m-3), E-8(m-3), G-8(m-1), and E-6(m-1). Microstructures from the stainless steel weld cladding, the A533B vessel steel, and the stainless steel/low alloy steel interface from the twelve Midland archive standards are shown in Appendix B. One of the first things that is apparent from this set of standards is a dark feathery line at the interface on the as-received archive sample and all those exposed to temperatures not exceeding 800°C. This line is still partially present on the samples exposed to 900°C for 1 and 10 minutes, but has disappeared (dissolved or dissipated) on all samples exposed for longer times or at higher temperatures. This same dark feathery line, although variable in thickness, is visible at the stainless steel/low alloy steel interface of all TMI-2 samples except F-10(m-3) and E-8(m-3). The stainless steel/low alloy steel interface area was not available for Samples G-8(m-1) and E-6(m-1,) but the ANL micrographs of other sections of G-8 and E-6 that did contain the interface had no evidence of the dark feathery line.

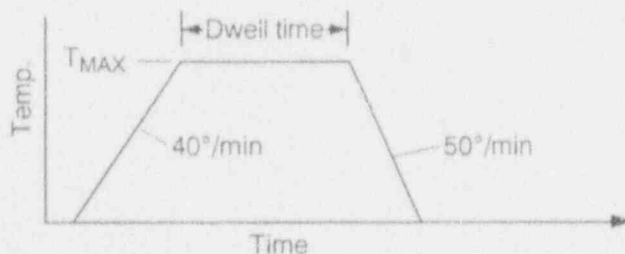
Dwell time at $T_{MAX}$ , min	$T_{MAX}$ , maximum temperature, °C			
	800	900	1000	1100
1	TMIG-15	TMIG-16	TMIG-17	TMIG-18
10	TMIG-19	TMIG-20	TMIG-21	TMIG-22
100	TMIG-23	TMIG-24	TMIG-25	TMIG-26

All samples

U91 0177

Heat up rate = 40°C/min

Cool down rate = 50°C/min

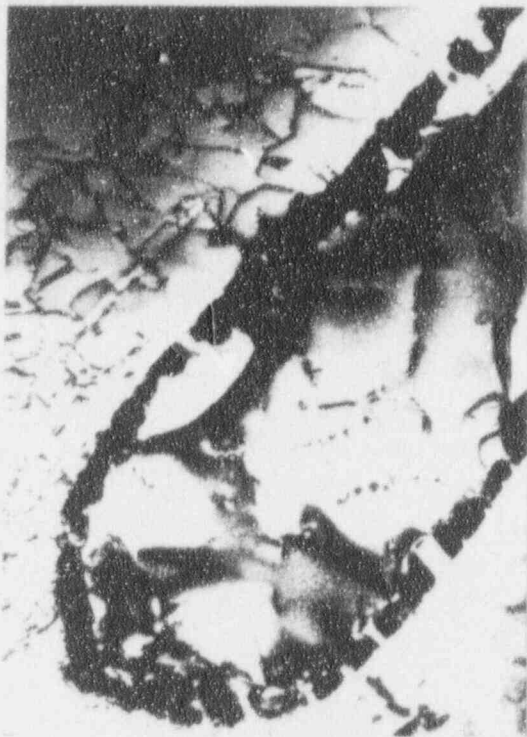


**Figure 9.** Thermal histories of the 12 Midland archive standards (first series).

It was also noted that in the standards the prior austenitic grain size of the A533B vessel steel some distance away from the interface starts to change quite dramatically after 1,000 and 1,100°C exposures. Directly adjacent to the interface, the low alloy steel grains go through a morphology change. Initially, they have a typical ferritic structure (slightly enlarged due to the weld cladding heat effects). As the temperature increases, the grains are refined with an equiaxed shape and, as the temperature continues to increase, eventually consumed by the growth of the larger austenitic grains.

The morphology of the  $\delta$ -ferrite islands in the stainless steel weld cladding also changes. At thermal exposures of 1,100°C for 10 minutes and also at 1,000°C for 100 minutes, the  $\delta$ -ferrite islands begin to lose their slender branch-like interdendritic morphology and become more spherical in shape. It was suspected that this spheroidizing was due to the dissolution of  $M_{23}C_6$  carbides, which decorate the austenite-ferrite boundaries at lower temperatures and thus tend to stabilize the shape of the islands. After the carbides dissolve, the  $\delta$ -ferrite would become more spherical shape to minimize the surface energy. The limited transmission electron microscopy (TEM) examinations performed appear to confirm this speculation. Figure 10 shows the presence of carbides at the austenite-ferrite interface in the stainless steel cladding for Samples K-13(m-3)<sup>b</sup>

b. Boat Sample K-13 is the sample most likely to be unaffected by the accident since it was not covered by core debris, and therefore should represent the TMI-2 lower head in the as-fabricated condition.



(a) K-13 (m-3)



(b) F-10 (m-3)

1  $\mu\text{m}$



(c) E-8 (m-3)

U91 0180

**Figure 10.** TEM micrographs showing the presence of carbides at the austenite-ferrite interface in the stainless steel weld cladding: (a) Sample K-13, as-fabricated condition, (b) Sample F-10(m-3), carbides starting to dissolve, and (c) Sample E-8(m-3), carbides have dissolved from austenite-ferrite boundary.

and F-10(m-3), but none were observed at an austenite/ferrite boundary for E-8(m-3). The carbides shown in Figure 10 for Sample F-10(m-3) appear to be partially dissolved when compared to those in K-13(m-3). The presence or absence of carbides correlates well with the spheroidizing of the  $\delta$ -ferrite islands.

Comparing all three areas (stainless steel cladding, interface, and low alloy steel), F-10(m-3) was found to most closely match the Midland archive standard given the 1,000°C exposure for 10 minutes, and the best match for E-8(m-3) falls between the standards exposed at 1,100°C for 10 and 100 minutes.

In an attempt to further refine the time/temperature history of F-10(m-3) and E-8(m-3), additional Midland archive samples were prepared and heat treated at times and temperatures between those of the previous series. For this second series the samples were approximately 3 x 6 x 25 mm in size and were heat treated in a quartz lamp infrared radiant furnace. Using these standards, the best match with sample F-10(m-3) is the archive sample exposed at 1,050°C for 30 minutes and E-8(m-3) falls between the archive samples exposed at 1,100°C for 10 and 30 minutes.

After the time/temperature history of F-10(m-3) and E-8(m-3) had been narrowed down using archive material, 3 x 6 x 25 mm slices were cut from TMI-2 metallurgical Samples H-4(m-3) and M-11(m-3), which are believed to represent the as-fabricated condition. These slices were heat treated as shown in Table 1. These thermal exposures are the same times and temperatures as the second series of Midland archive standards. This action was taken to eliminate, as much as possible, subtle thermal response differences that might exist between the Midland archive material and actual TMI-2 lower head material. Microstructures from the heat-treated slices of H-4(m-3) and M-11(m-3) are shown in Appendix C. Microstructures of F-10(m-3) and E-8(m-3) with the same etch and magnifications and the micrographs from the ANL hot cell sections G-8 (408P-3) and E-6 (402A-1) are also shown in this appendix.

**Table 1.** Sample identification and heat treatments given the slices from H-4(m-3) and M-11(m-3) samples.

Dwell time ( $T_{max}$ , minutes)	Maximum temperature ( $T_{max}$ , °C)			
	950	1,000	1,050	1,100
10	H4-1	H4-2	H4-3	H4-4
30	H4-5	H4-6	M11-1	M11-2
100	M11-3	M11-4	M11-5	M11-6

Heatup rate = 200°C/minute, cooling rate = 50°C/minute

The H-4(m-3) and M-11(m-3) heat treatments showed that the structural changes in the TMI-2 material were very similar to those of the Midland archive material. Some subtle differences in the prior austenite grain size and morphology were noted with the TMI-2 material. Therefore, the heat treated slices from Samples H-4(m-3) and M-11(m-3) were used for the final time/temperature history determinations for F-10(m-3), E-8(m-3), G-8(m-1), and E-6(m-1), even though the final conclusions were the same as those based on the Midland archive material.

In an attempt to illustrate the various metallurgical observations from the prepared standards of Midland archive material and the heat treated TMI-2 material, the diagram shown in Figure 11 was constructed. Since the vessel was stress relieved at 610°C after the weld cladding, no thermal effects from the accident could be detected at or below this temperature and, therefore, the diagram only shows metallurgical observations for temperatures above this point. The lowest temperature indicator, above the stress relief temperature, was the ferrite-austenite transformation, which starts at 727°C and is complete by 810°C. Variations in hardness will be evident when this threshold is exceeded. The next indicator is the dissolution of the dark feathery band at the interface; this occurs between 800 and 925°C, depending on the time. The next indicator of increasing temperature is the appearance of small equiaxed grains in the A533B steel adjacent to the interface that form between 850 and 900°C and disappear between 1,025 and 1,100°C as they are consumed by grain growth in the low alloy steel. The dissolution of the dark feathery band and the formation of the equiaxed grains are believed to be associated with carbon diffusion into the stainless steel cladding. The dark feathery band appears to be some sort of carbide that disperses as the carbon diffuses into the stainless steel. The equiaxed grains, which are not typical for a low alloy steel, appear to be devoid of cementite, undoubtedly due to a loss of carbon into the stainless steel. Grain growth in the A533B steel becomes significant above approximately 950 to 1,075°C, depending on the time involved. The highest temperature indicator shown on the diagram is the spheroidizing of the  $\delta$ -ferrite islands in the stainless steel cladding, which occurs in the approximate range of 975 to 1,000°C at 100 minutes or 1,100 to 1,125°C at 10 minutes.

The thermal histories were determined by applying the above observations to microstructural examinations of areas near the stainless steel/low alloy steel interface (within 2.5 mm). The temperature gradient through the thickness of the lower vessel head wall was estimated by two methods. First, since the high level of hardness of the four affected samples persisted to the full depth of the boat samples (50 mm from the inside surface, 45 mm from the weld clad interface, see Figure 3), it could be concluded that the temperature at that depth was greater than the 727°C transformation temperature. Secondly, since it had been established that the thermal excursion on the lower head due to the accident was of the order of 30 minutes, prior austenite grain size at the bottom-most tip of the heat-affected samples was compared with the prepared standards given the 30-minute heat treatments. The results of this rough analysis indicated that the temperature 50 mm from the inside surface (45 mm from the stainless steel/low alloy steel interface) was approximately 50 to 150°C lower than the peak temperatures determined previously for the region near the interface. There is a fair amount of uncertainty in the gradient estimate since only average prior austenite grain size was used for the determination and that measurement cannot be made with precision. Also, the assumption that 30 minutes was the actual time at peak temperatures 50 mm into the thickness may be in error.



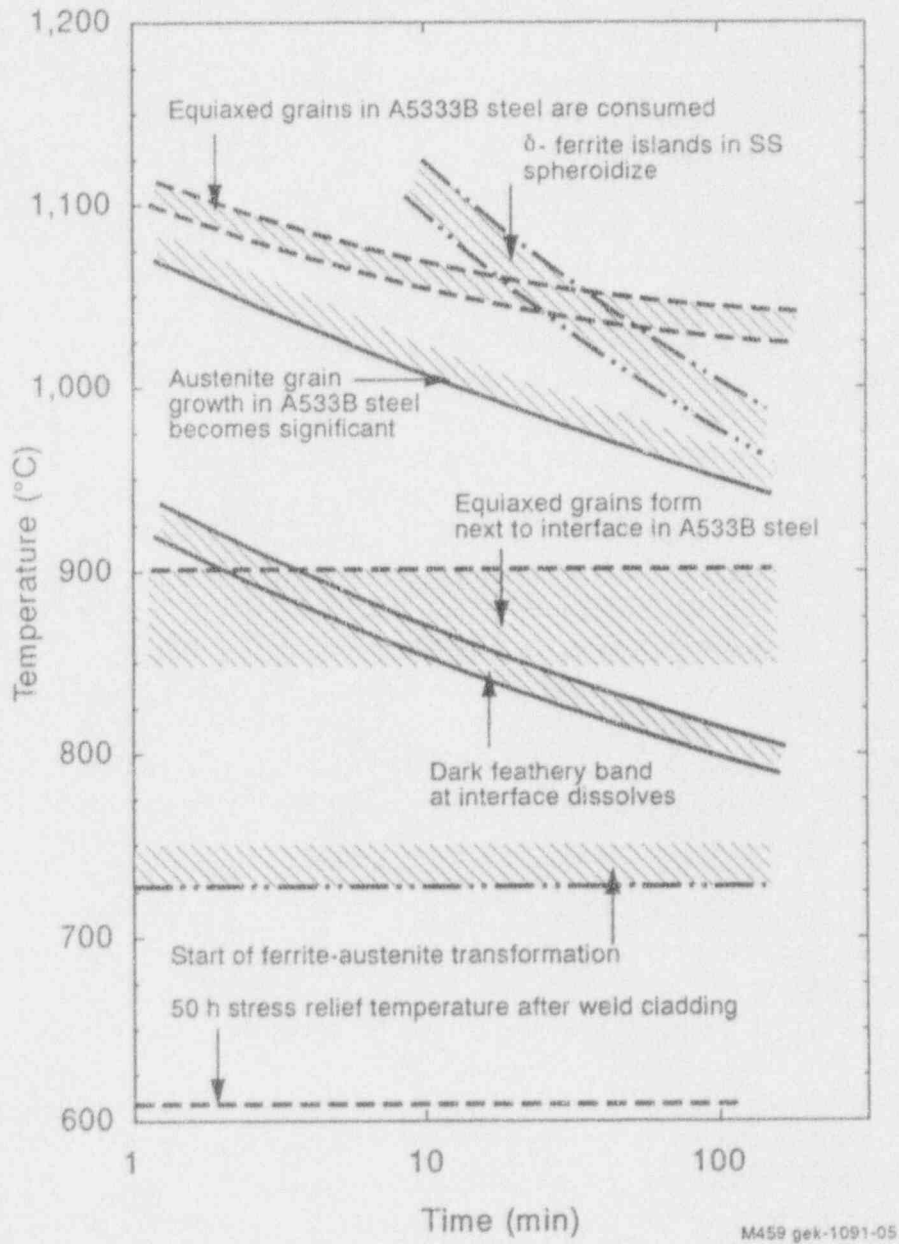


Figure 11. Diagram of time/temperature observations of A533B pressure vessel steel clad with Type 308L stainless steel.

## CONCLUSIONS

From this investigation, the following conclusions can be made concerning the thermal history of the TMI-2 steel samples extracted from the lower head.

1. Of the thirteen full section and two partial section samples received at the INEL, only F-10(m-3), E-8(m-3), G-8(m-1), and E-6(m-1) have shown hardness values indicative of having exceeded 727°C (1,000 K).
2. A feathery dark band right at the low alloy steel/stainless steel cladding interface starts to dissolve at temperatures of the order of 900°C at times of 10 minutes and longer. Carbon diffusion from the low alloy steel into the stainless steel weld cladding is evident in the accident heat-affected samples and is undoubtedly the mechanism for the dissolution of the feathery carbide band and the formation of the cementite-devoid equiaxed grains.
3. At a temperature of 1,000°C and a time of 100 minutes, or a temperature of 1,100°C and a time of 10 minutes, the carbides dissolve at the ferrite/austenite boundaries of the stainless steel cladding and the  $\delta$ -ferrite islands change their morphology.
4. Grain growth of the low alloy steel starts to become significant at 1,000°C and, therefore, prior austenite grain size is another indicator of temperature.
5. Using the combination of microstructure from the stainless steel weld cladding, the pressure vessel steel austenite grain size and morphology, the interface, and carbon diffusion distance into the stainless steel, comparisons with standards of known thermal histories showed that Sample F-10(m-3) experienced 1,040 to 1,060°C for approximately 30 minutes during the accident and Sample E-8(m-3), 1,075 to 1,100°C for about 30 minutes.
6. Although the stainless steel cladding and the stainless steel/low alloy steel interface were not available at INEL for Samples G-8(m-1) and E-6(m-1), based on the hardness and microstructure of the low alloy steel and the hot cell micrographs from ANL that did show the interface and stainless steel cladding, the G-8 position experienced temperatures during the accident approximately the same as F-10(m-3) (1,040 to 1,060°C for 30 min) and the E-6 position was approximately the same as E-8(m-3) (1,075 to 1,100°C for 30 minutes).
7. The temperatures at 50 mm from the inside surface (45 mm from the weld cladding/low alloy steel interface) were estimated to be  $100 \pm 50^\circ\text{C}$  lower than the peak temperatures.

## REFERENCES

1. R. L. Moore and E. L. Tolman, *Estimated TMI-2 Vessel Thermal Response Based on the Lower Plenum Debris Configuration*, Joint AIChE/ASME Heat Transfer Conference, High Melt Attack Phenomena Session, Houston, TX, July 27, 1988.
2. D. R. Diercks, *TMI-2 Vessel Investigation Project (VIP) Metallurgical Program*, Progress Report January-September 1989, NUREG/CR-5224, ANL-90/2, Vol. 1, March 1990.
3. R. Tricot and R. Castro, "Study of the Isothermal Transformations in 17% Cr Stainless Steels," *The Metallurgical Evolution of Stainless Steels*, ed. by F.B. Pickering, American Society for Metals, Metals Park, Ohio (1979), p. 256.

## **Appendix A**

### **Microstructure and Hardness Profiles of All TMI-2 Metallurgical Samples and Midland Archive Material**

**(In alphabetical order)**

Sectioning diagram of TMI-2 lower head sample D-10 and hardness profile of INEL subsection D-10 (m-2)

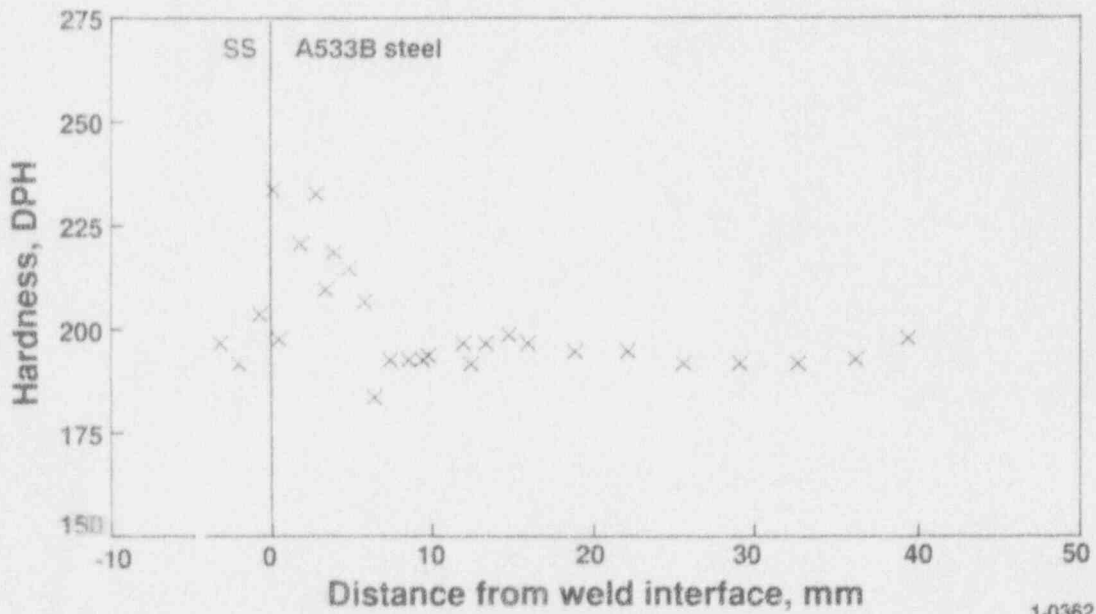
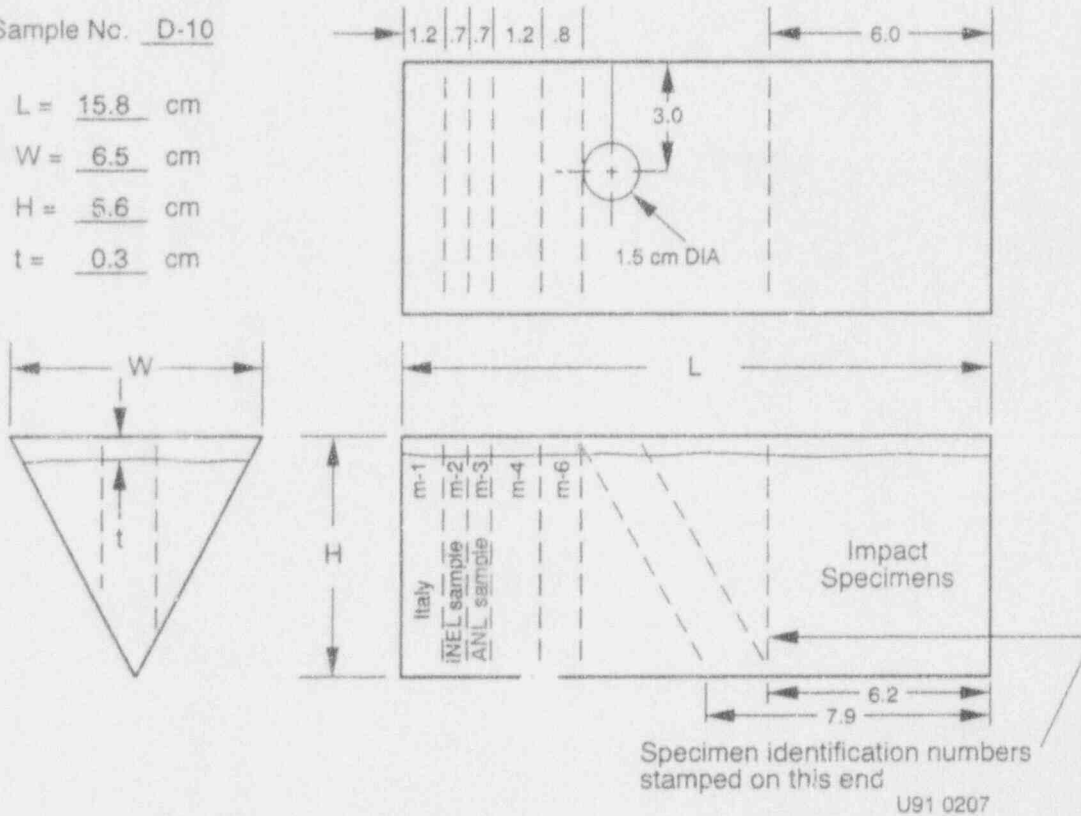
Sample No. D-10

L = 15.8 cm

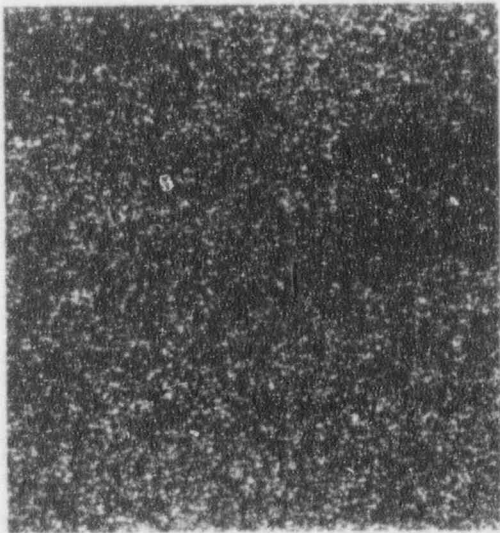
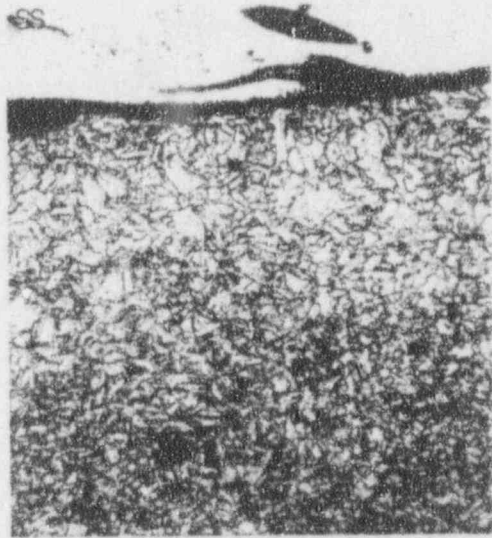
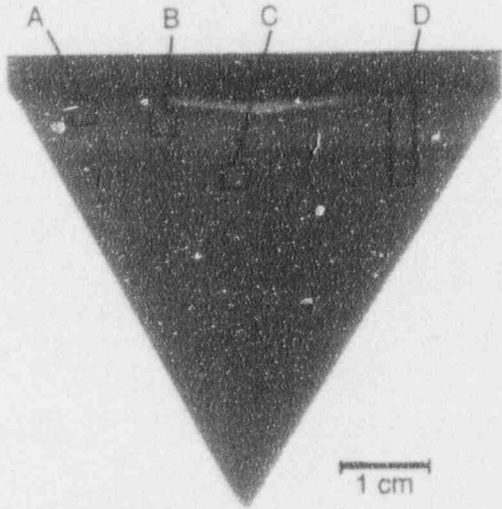
W = 6.5 cm

H = 5.6 cm

t = 0.3 cm



Sample D-10 (m-2)



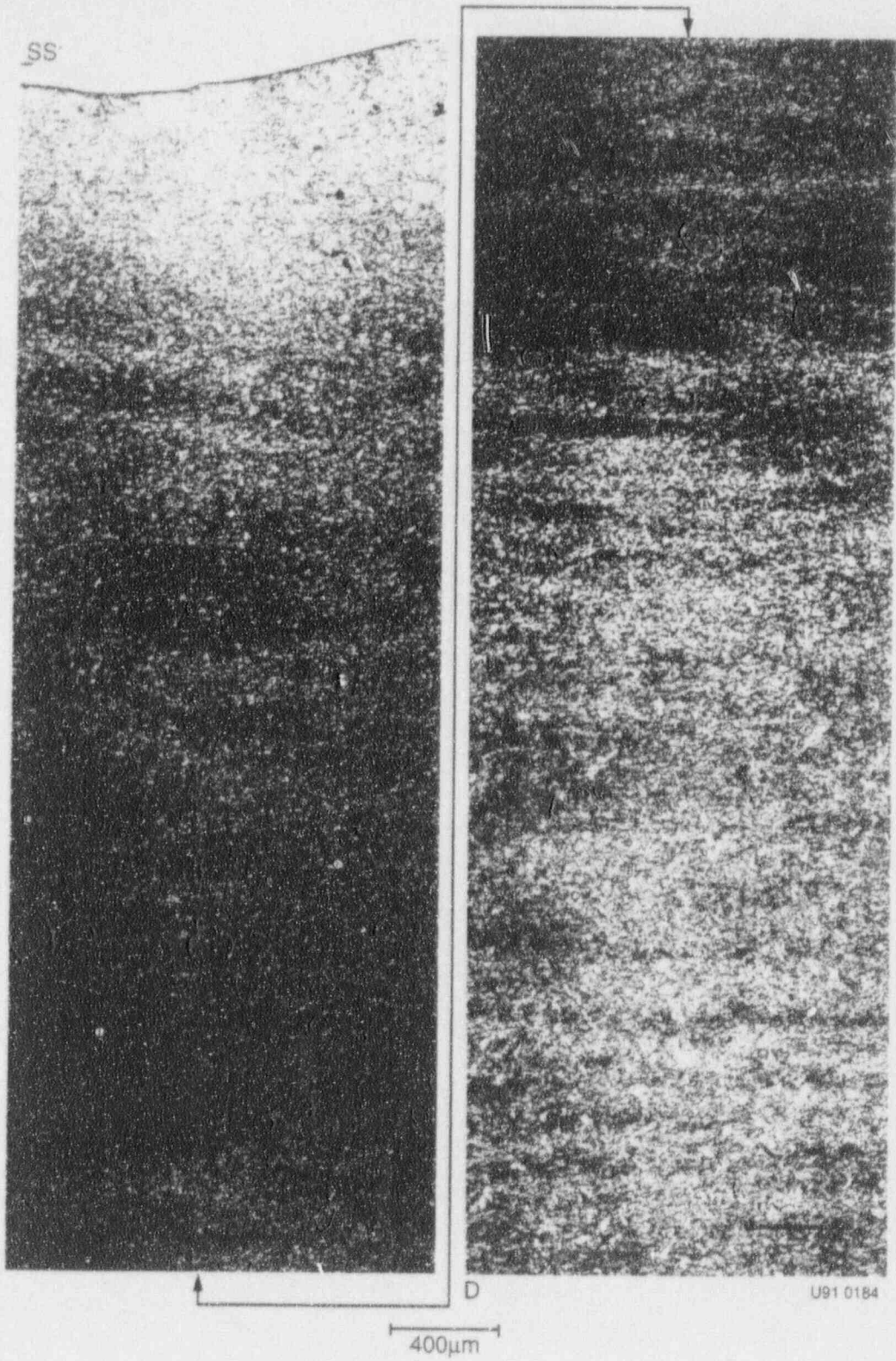
B

A

U91 0183

100µm

Sample D-10 (m-2)



Sample D-10 (m-2)  
(backside)

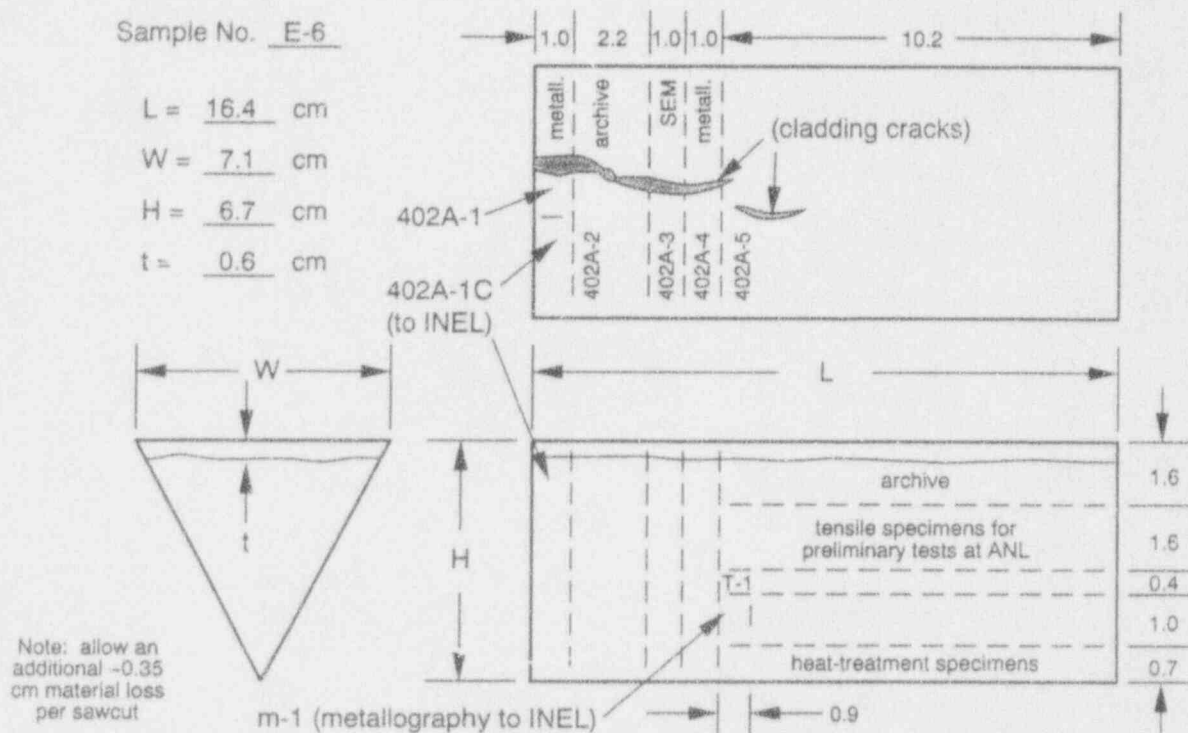


Near nozzle penetration

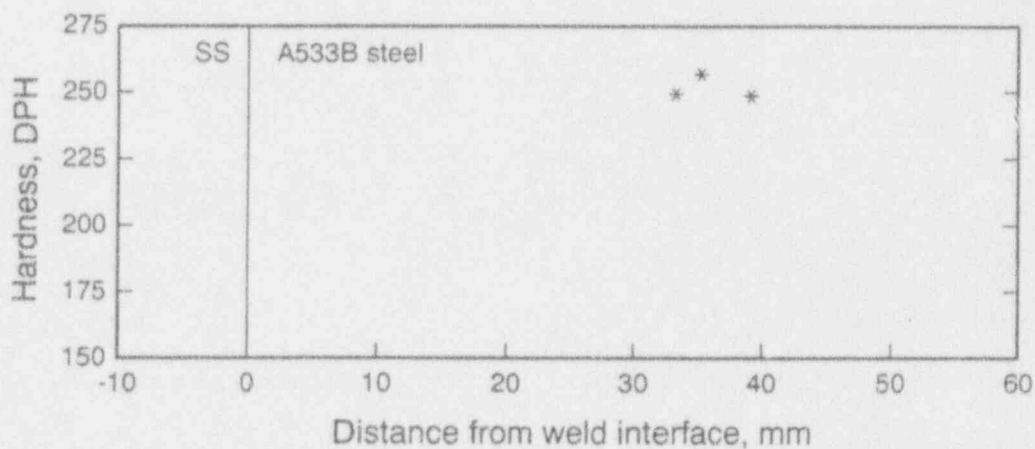
U91 0185



Sectioning diagram of TMI-2 lower head sample E-6 and hardness profile of INEL subsection E-6 (m-1)



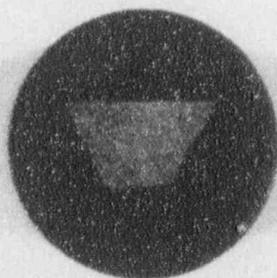
U91 0208



U91 0209

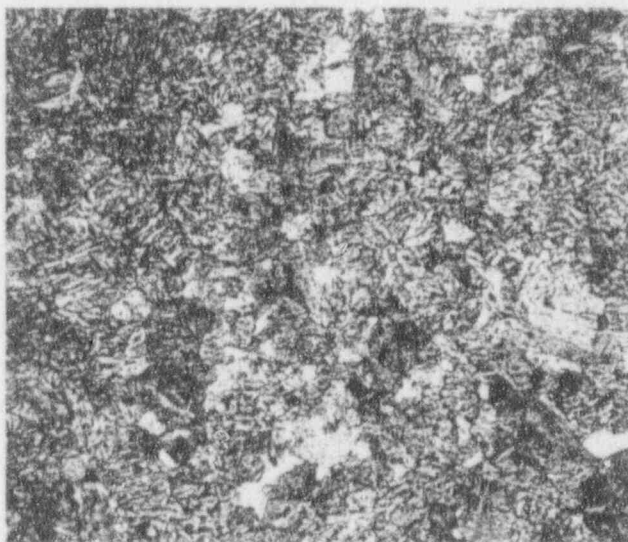
1-0363

Sample E-6 (m-1)



(In met. mount)

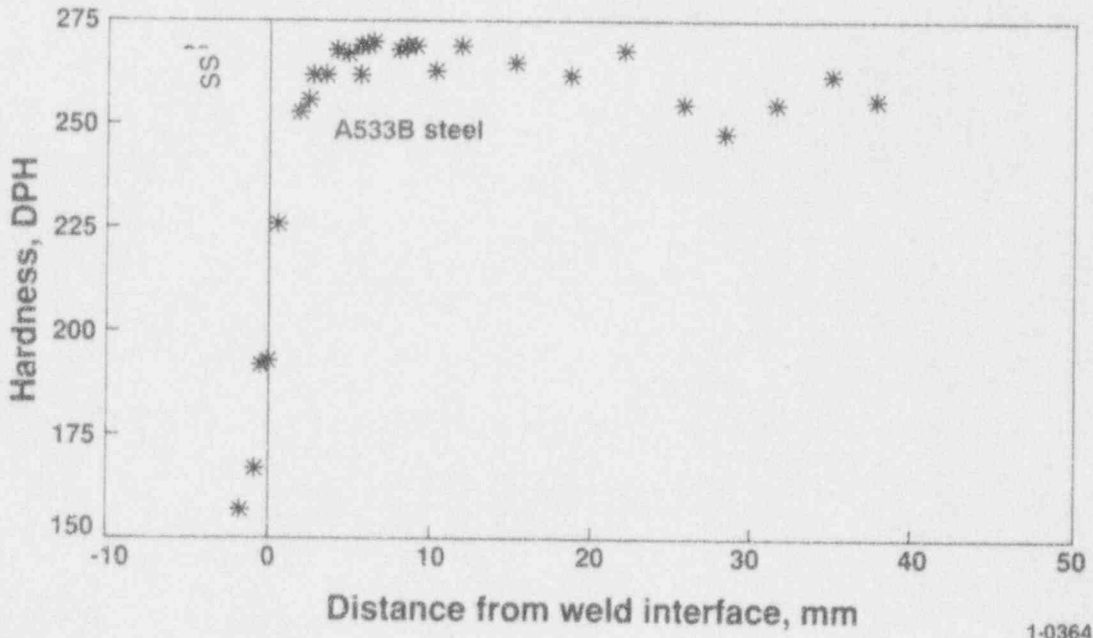
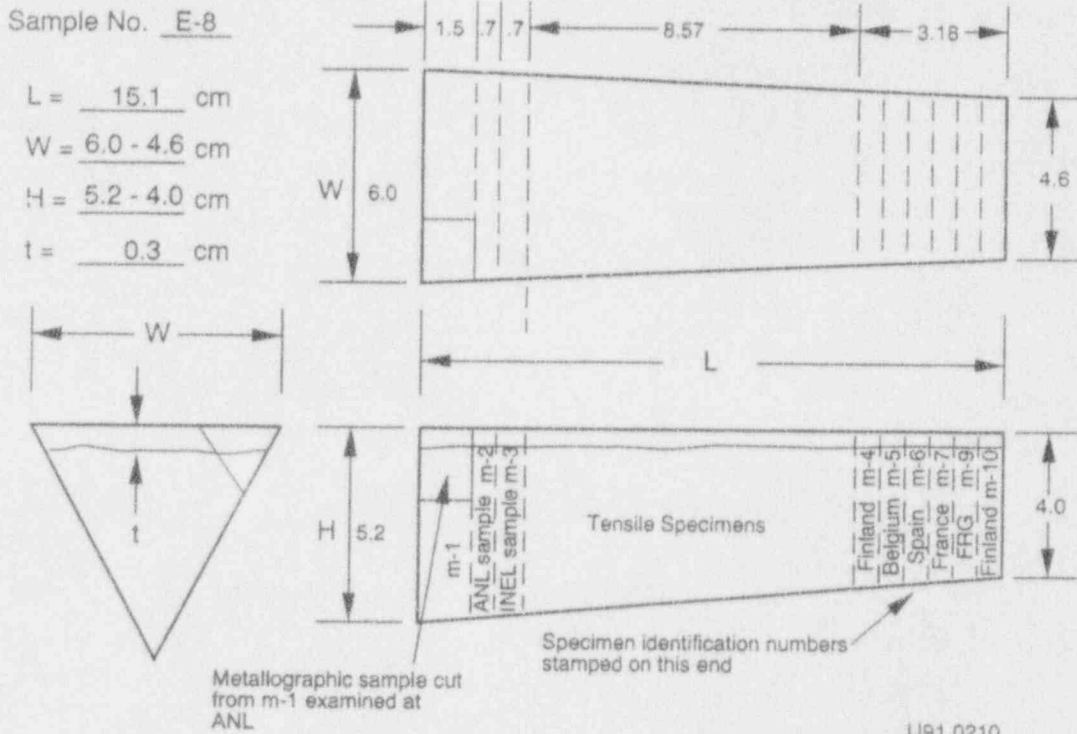
1 cm



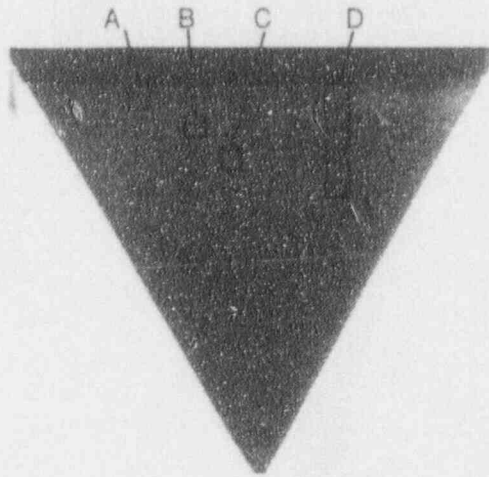
100µm

U91 0186

Sectioning diagram of TMI-2 lower head sample E-8 and hardness profile of INEL subsection E-8 (m-3)

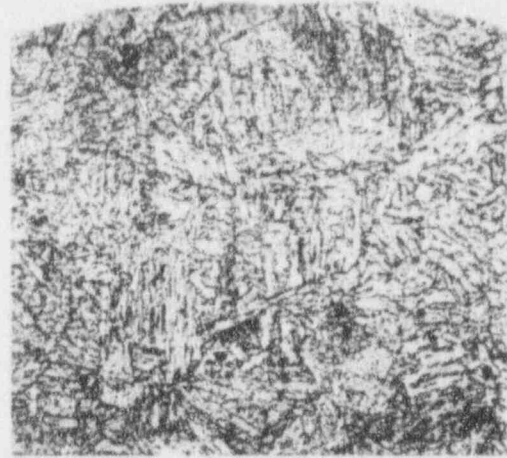


Sample E-8 (m-3)

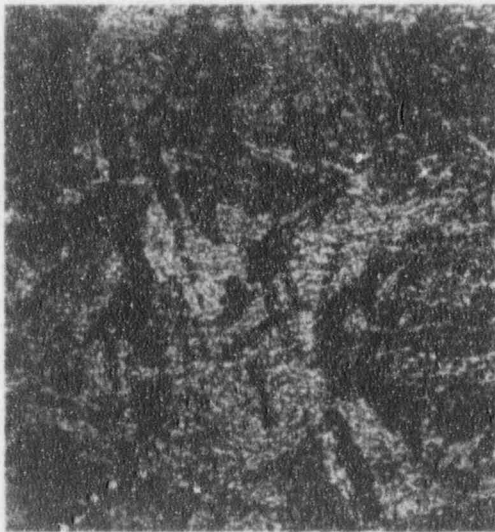


1 cm

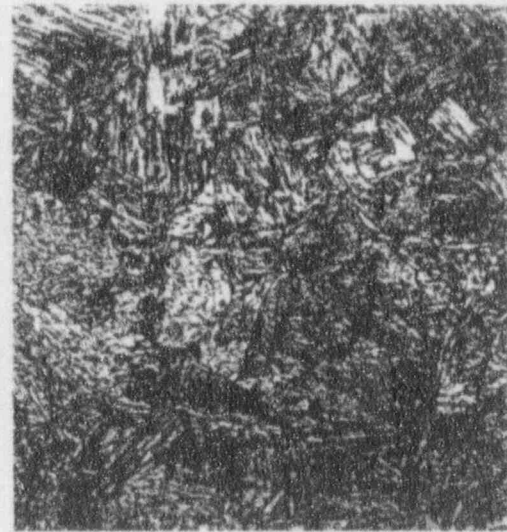
SS



A



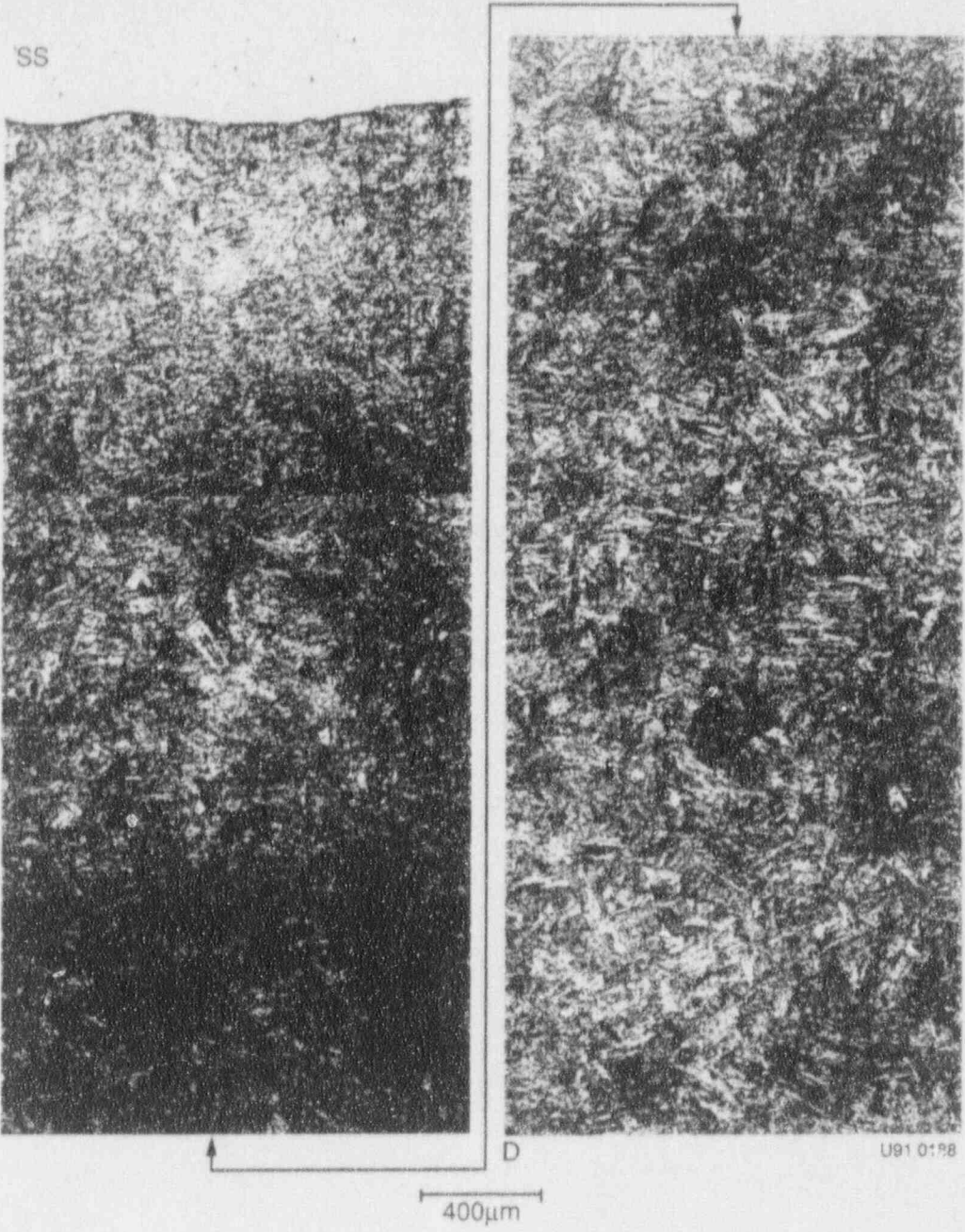
B



C

U91 0187

100μm



Sectioning diagram of TMI-2 lower head sample E-11 and hardness profile of INEL subsection E-11 (m-3)

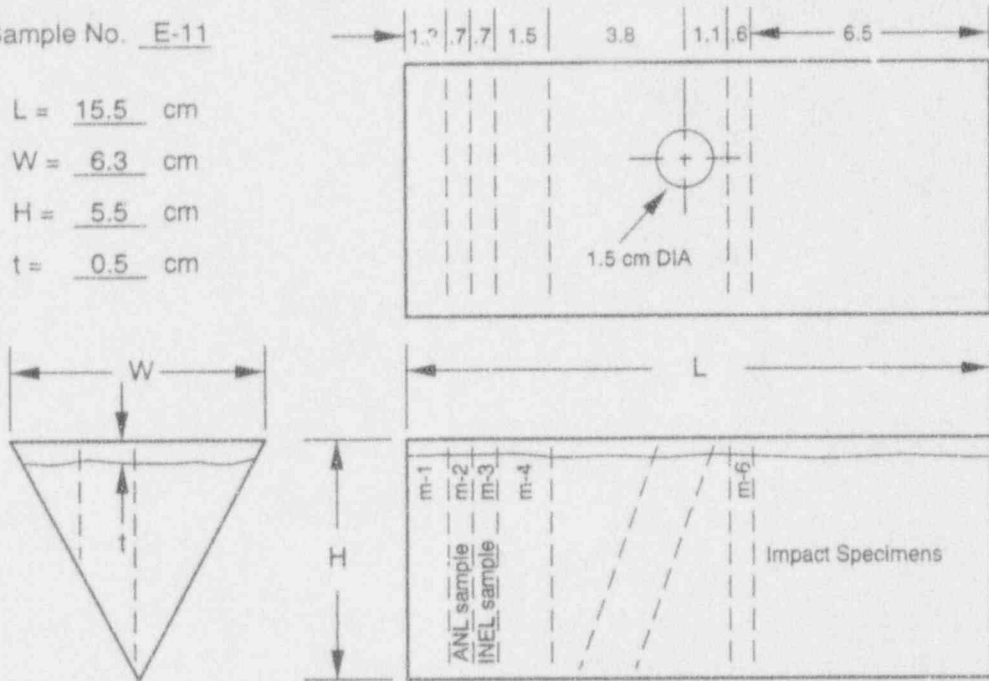
Sample No. E-11

L = 15.5 cm

W = 6.3 cm

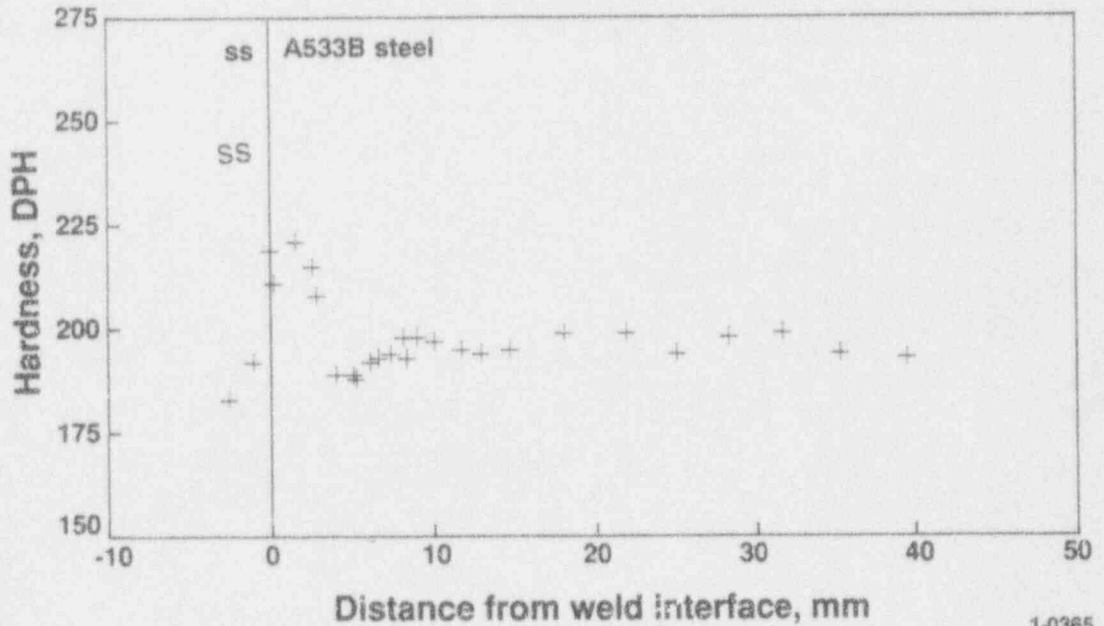
H = 5.5 cm

t = 0.5 cm



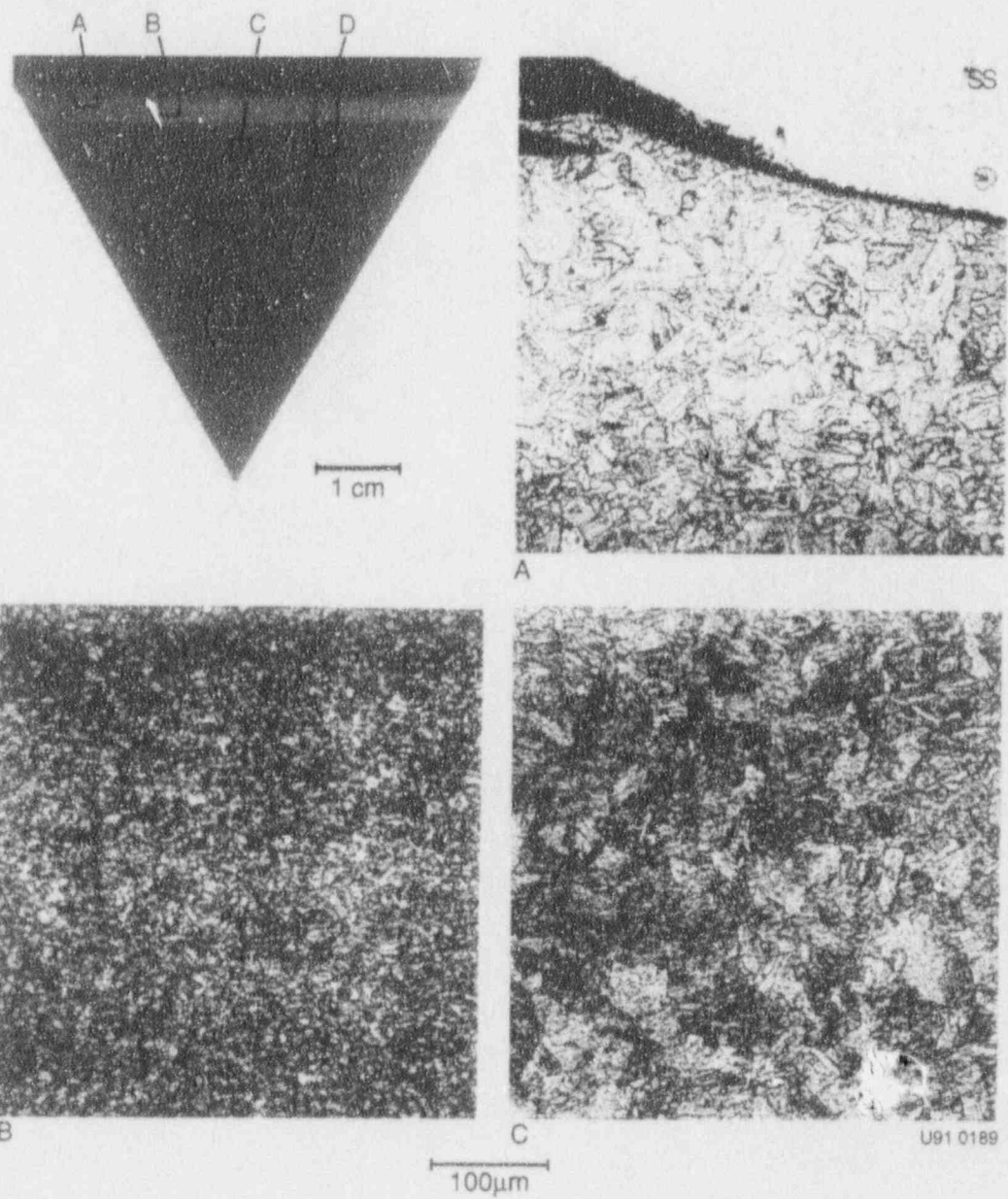
Specimen identification numbers stamped on this end

U91 0211

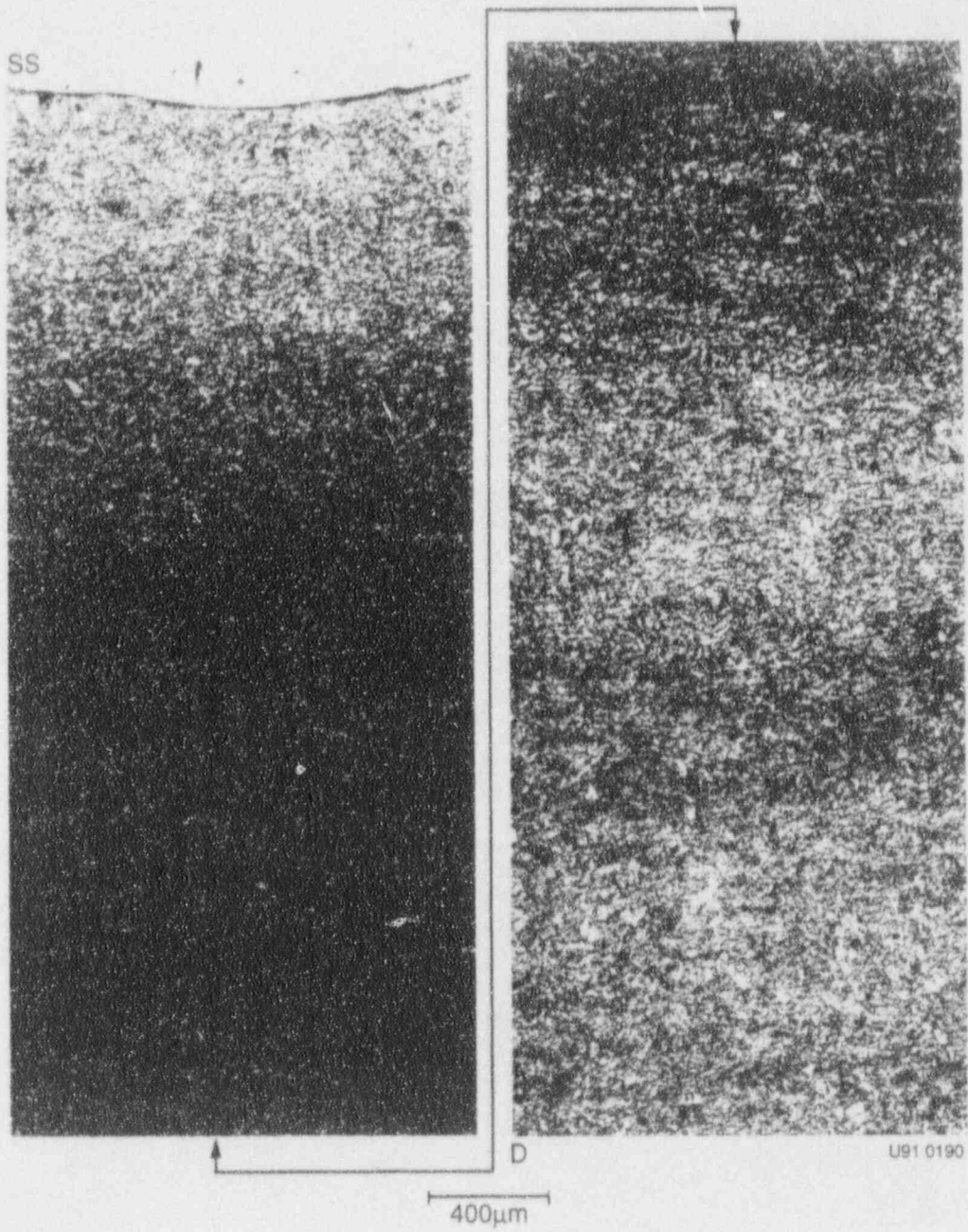


1-0365

Sample E-11 (m-3)



Sample E-11 (m-3)





Sectioning diagram of TMI-2 lower head sample F-5 and hardness profile of INEL subsection F-5 (m-3)

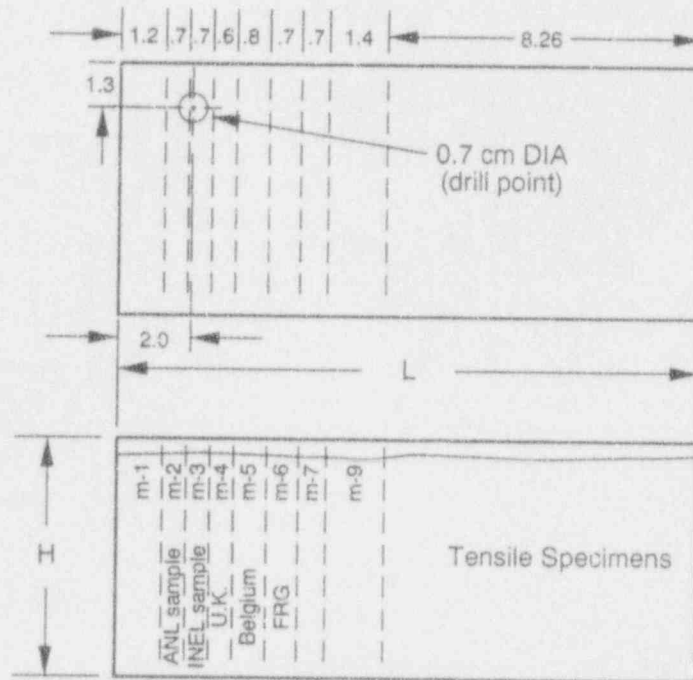
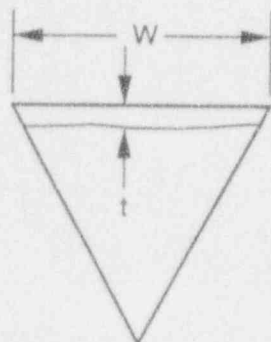
Sample No. F-5

L = 15.8 cm

W = 6.7 cm

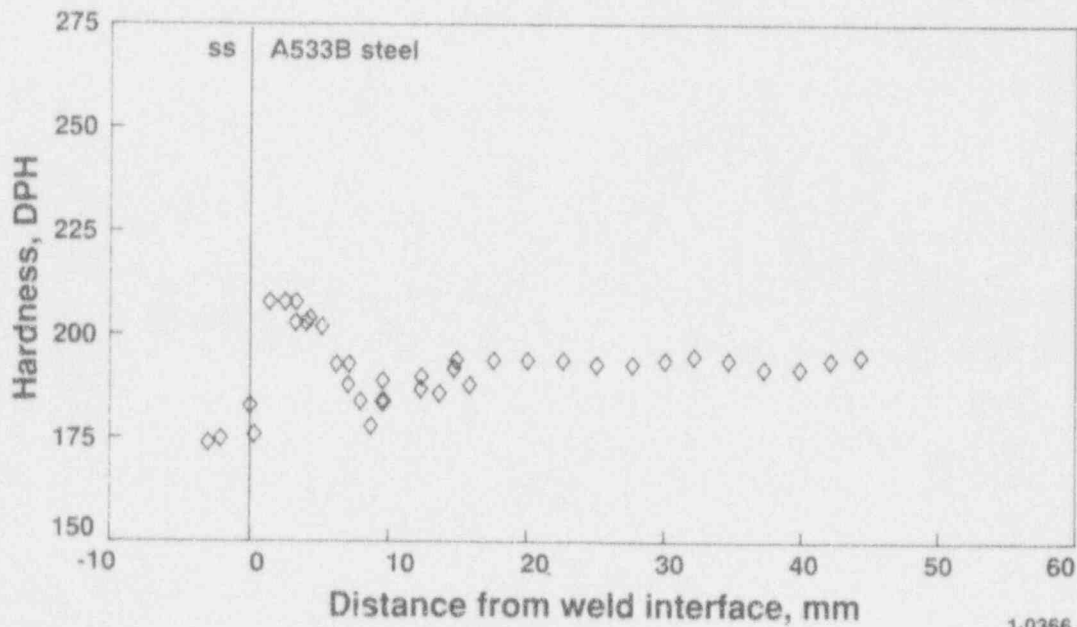
H = 5.7 cm

t = 0.5 cm



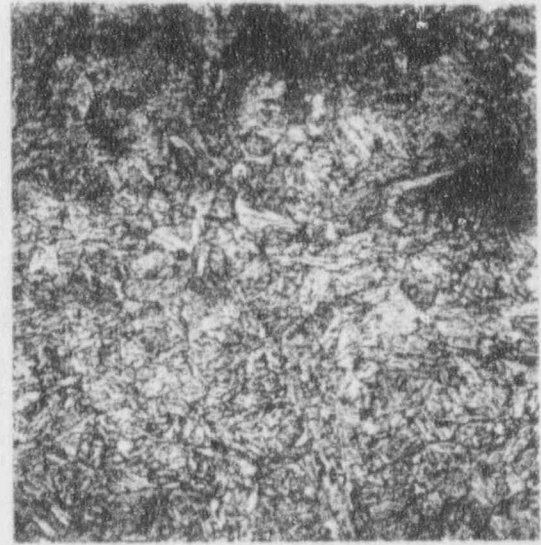
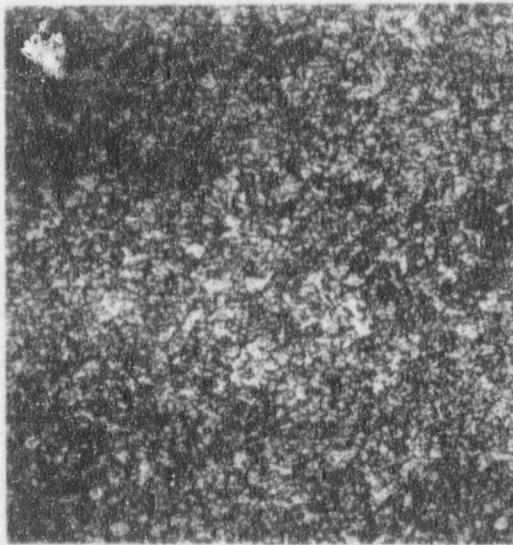
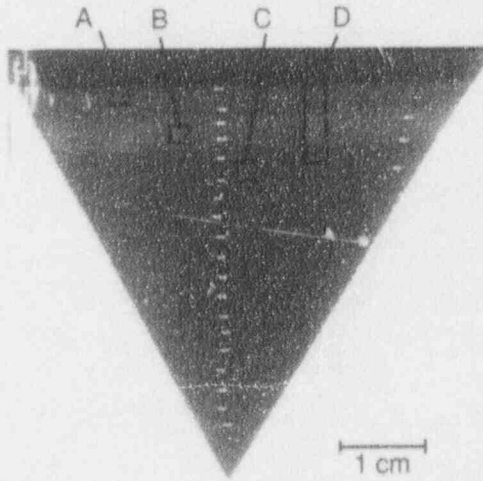
Specimen identification numbers stamped on this end

U91 0212



1-0366

Sample F-5 (m-3)



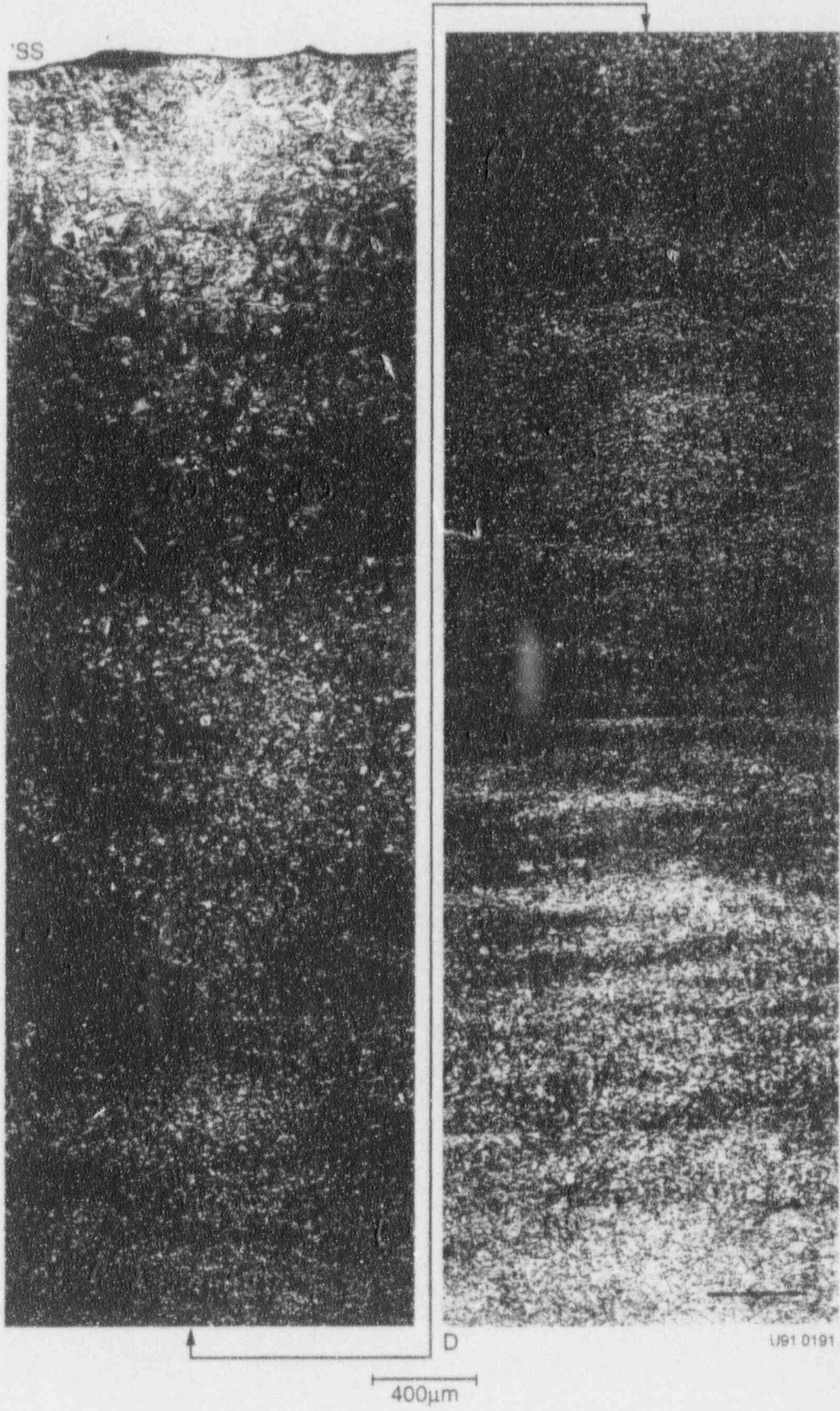
B

C

U91 0181

100µm

Sample F-5 (m-3)



Sectioning diagram of TMI-2 lower head sample F-10 and hardness profile of INEL subsection F-10 (m-3)

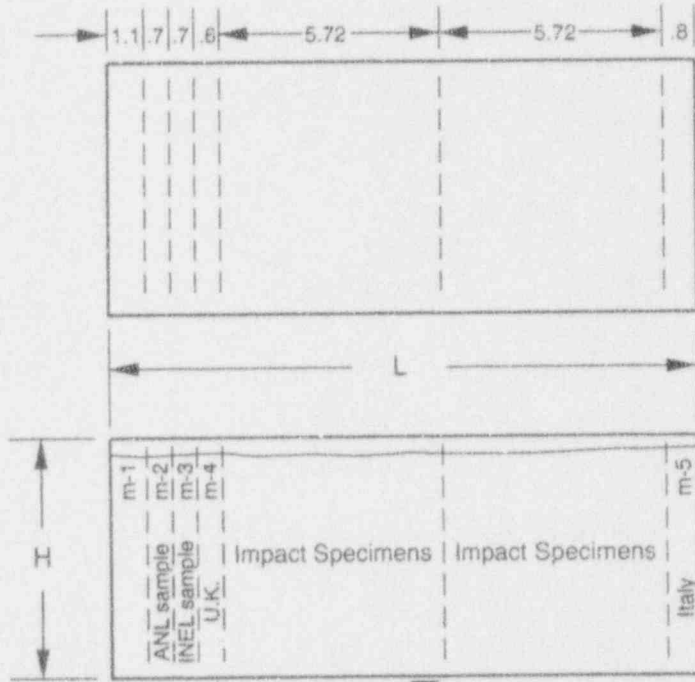
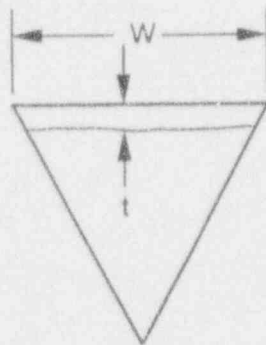
Sample No. F-10

L = 15.7 cm

W = 6.1 cm

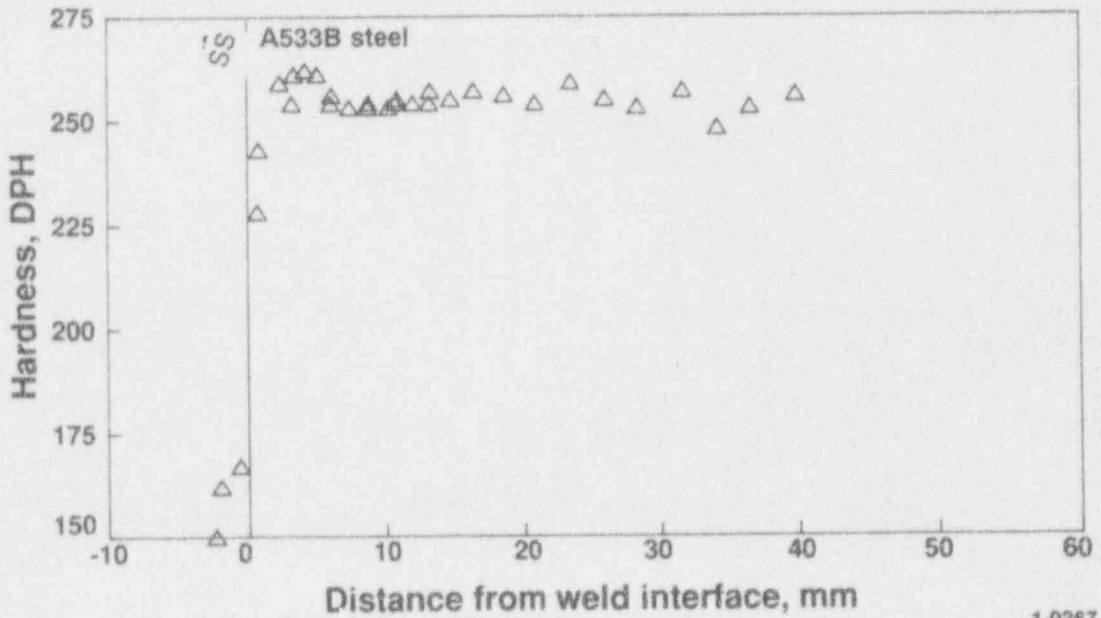
H = 5.3 cm

t = 0.3 cm



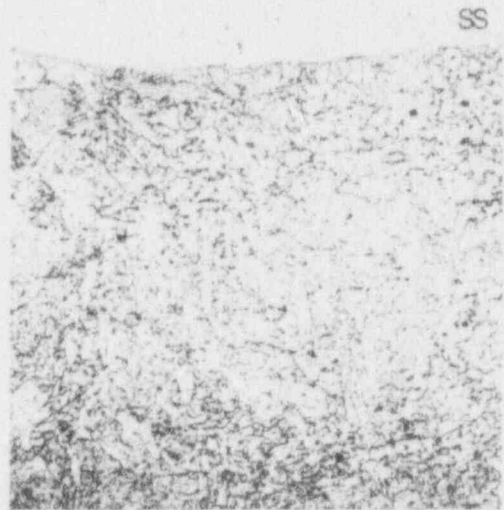
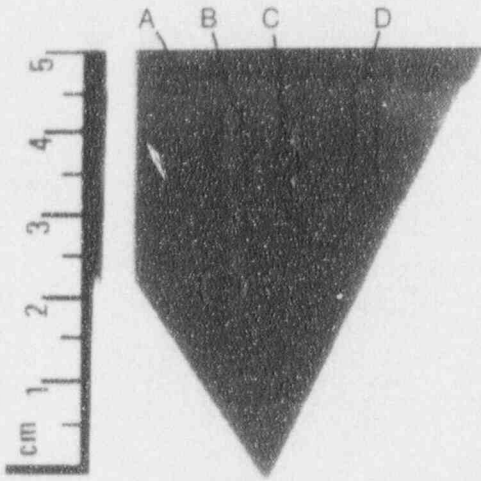
Specimen identification numbers stamped on this end

U91 0213



1-0367

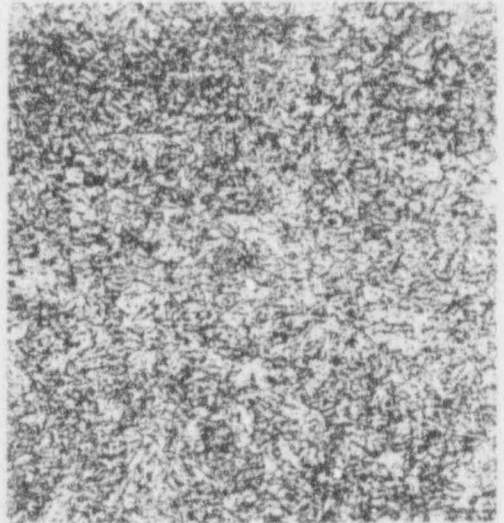
Sample F-10 (m-3)



A



B

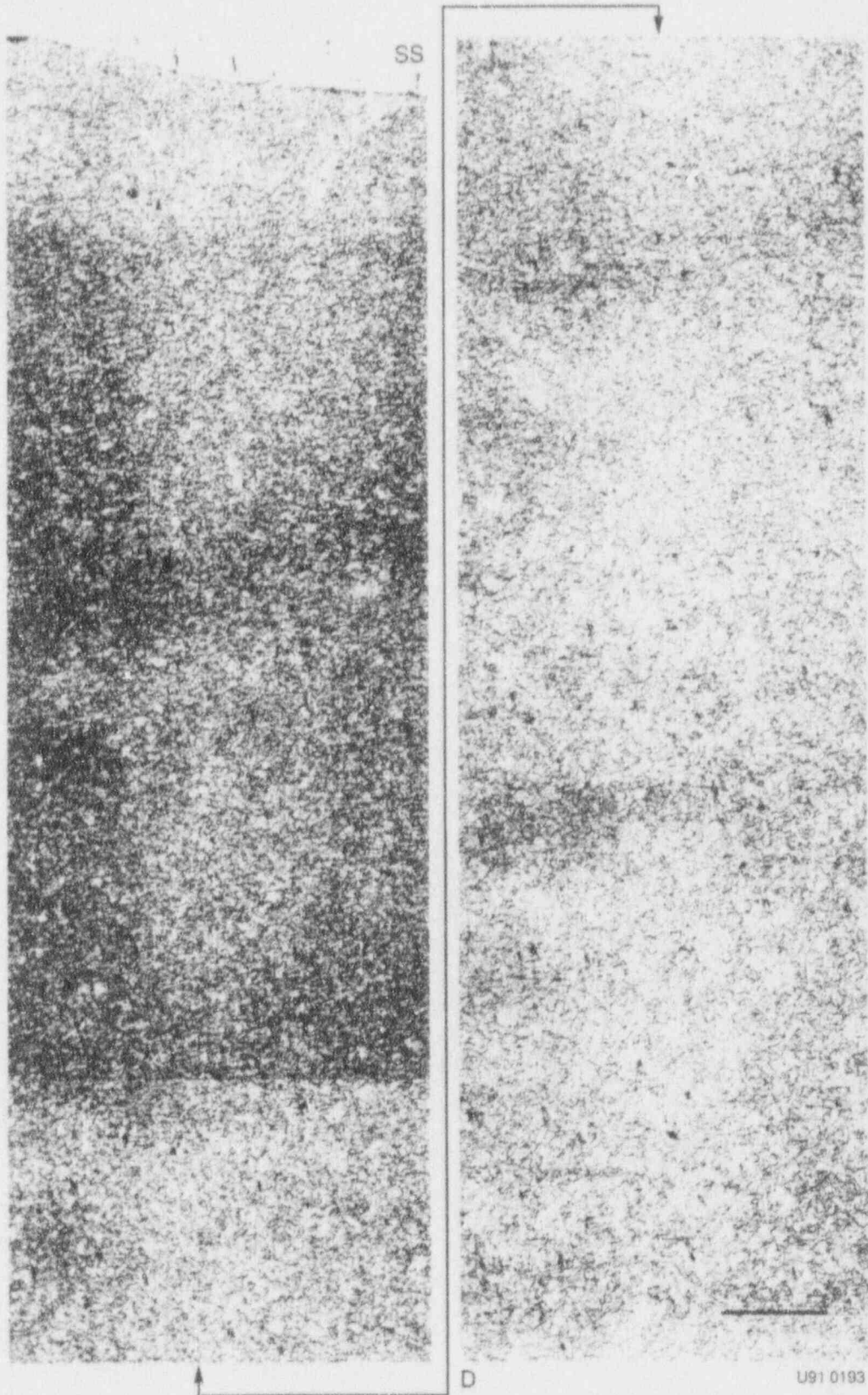


C

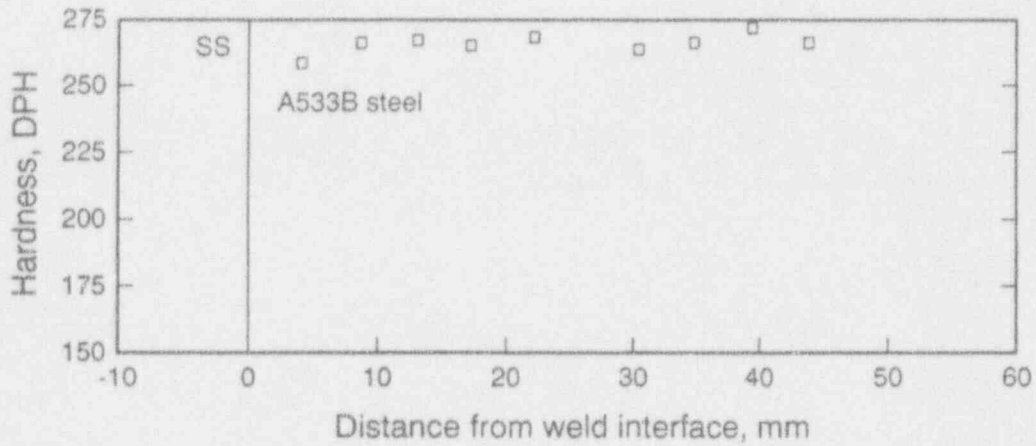
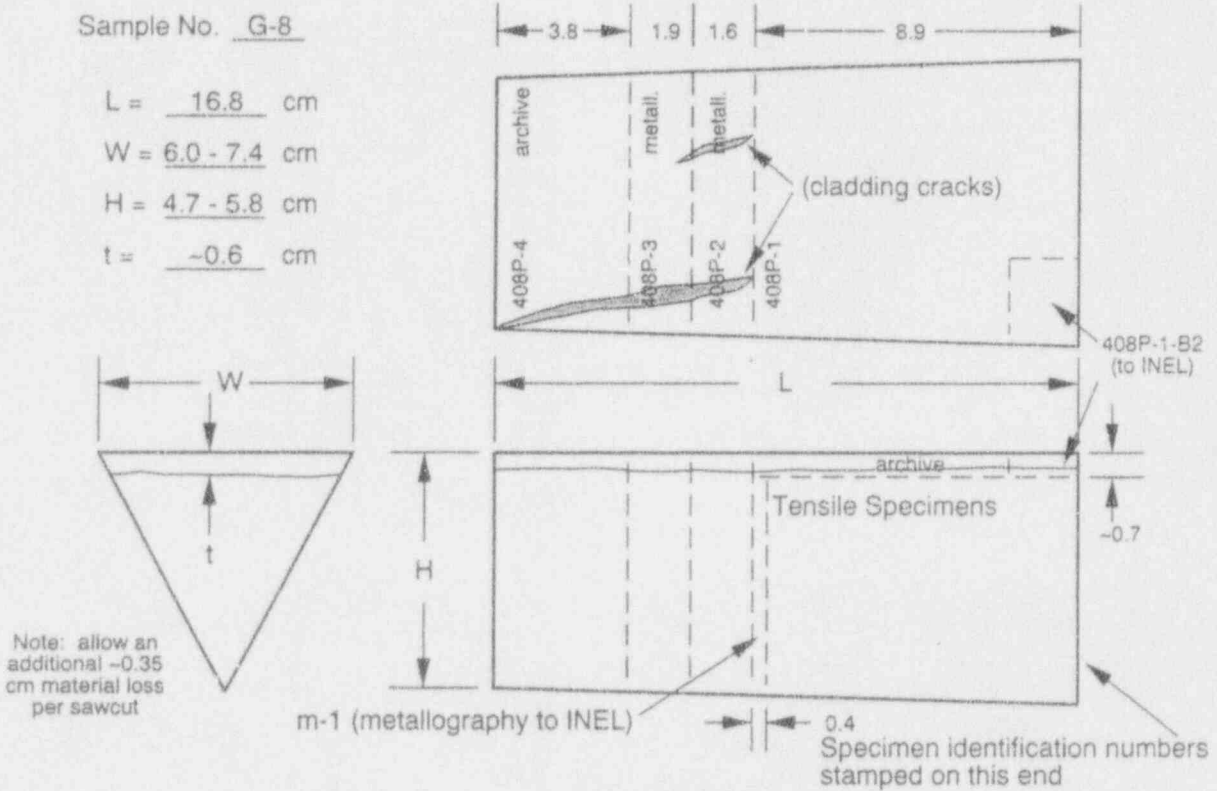
U91 0192

100μm

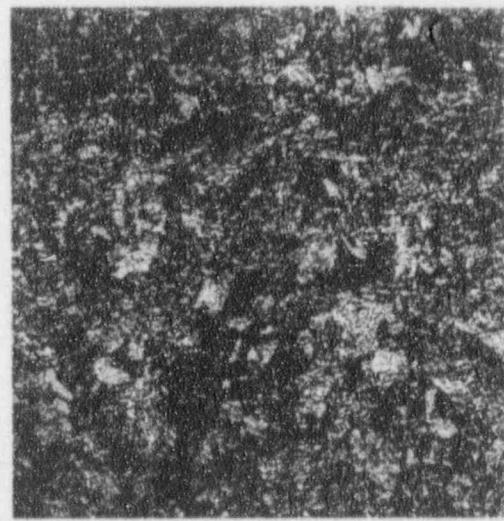
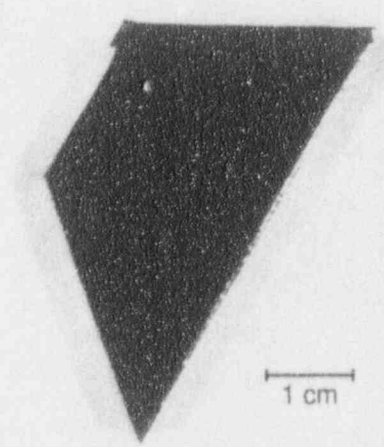
Sample F-10 (m-3)



Sectioning diagram of TMI-2 lower head sample G-8 and hardness profile of INEL subsection G-8 (m-1)



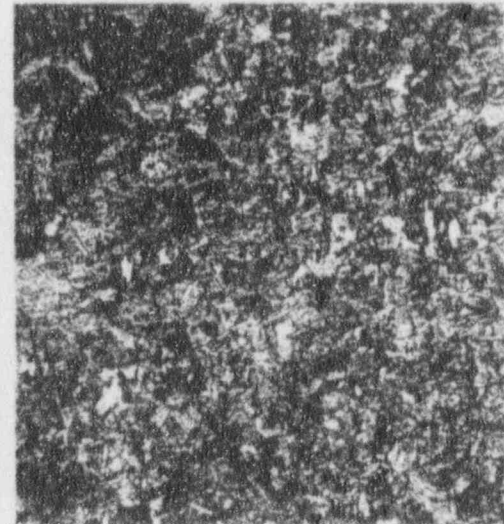
Sample G-8 (m-1)



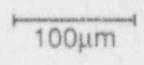
Near top



Midway



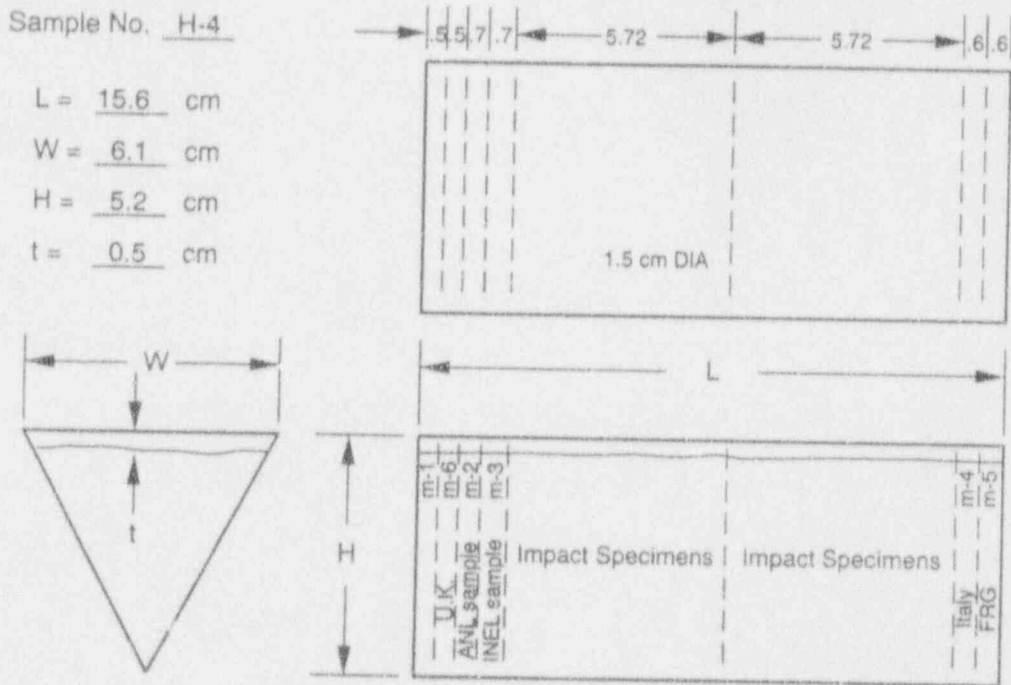
Near bottom



U91 0194

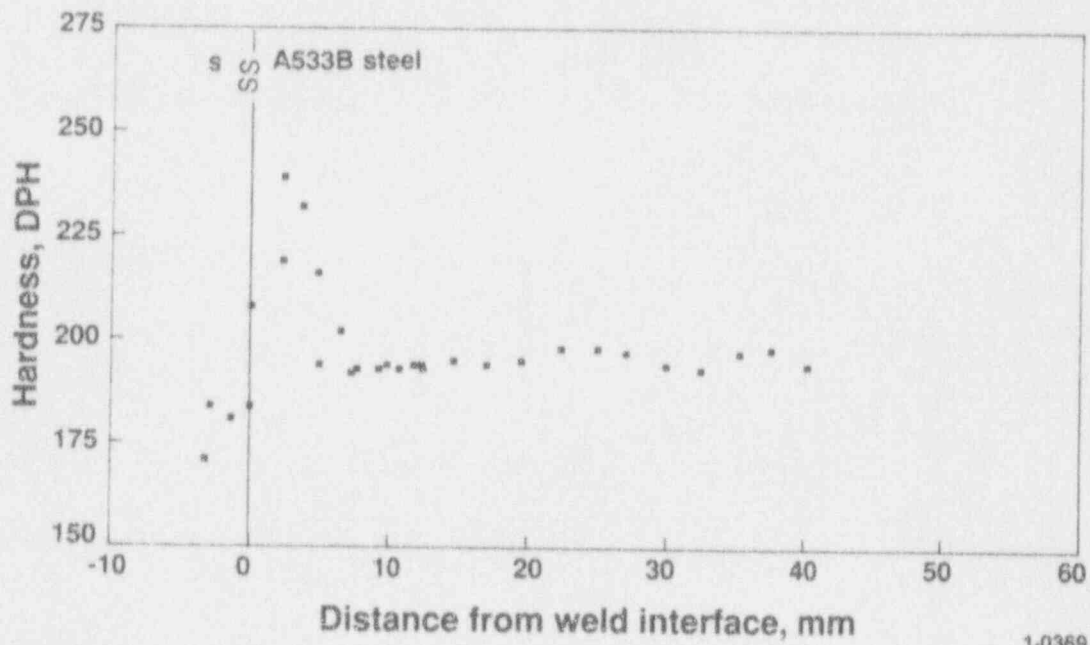


Sectioning diagram of TMI-2 lower head sample H-4 and hardness profile of INEL subsection H-4 (m-3)



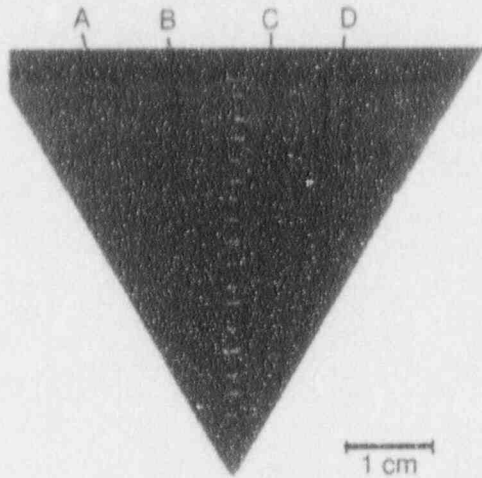
Specimen identification numbers stamped on this end

U91 0216



1-0369

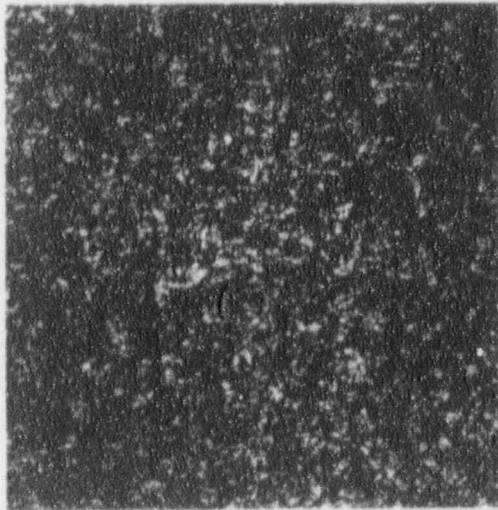
Sample H-4 (m-2)



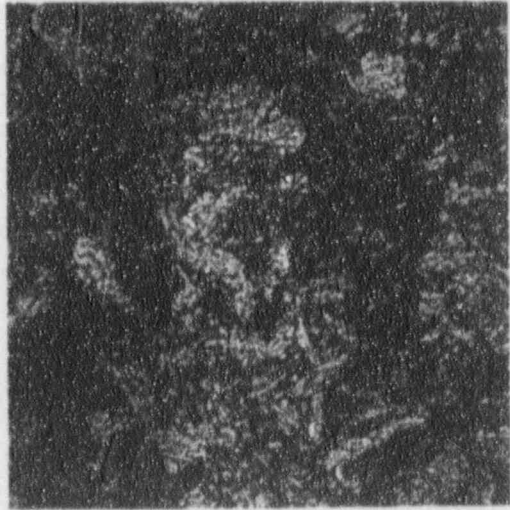
1 cm



A



B

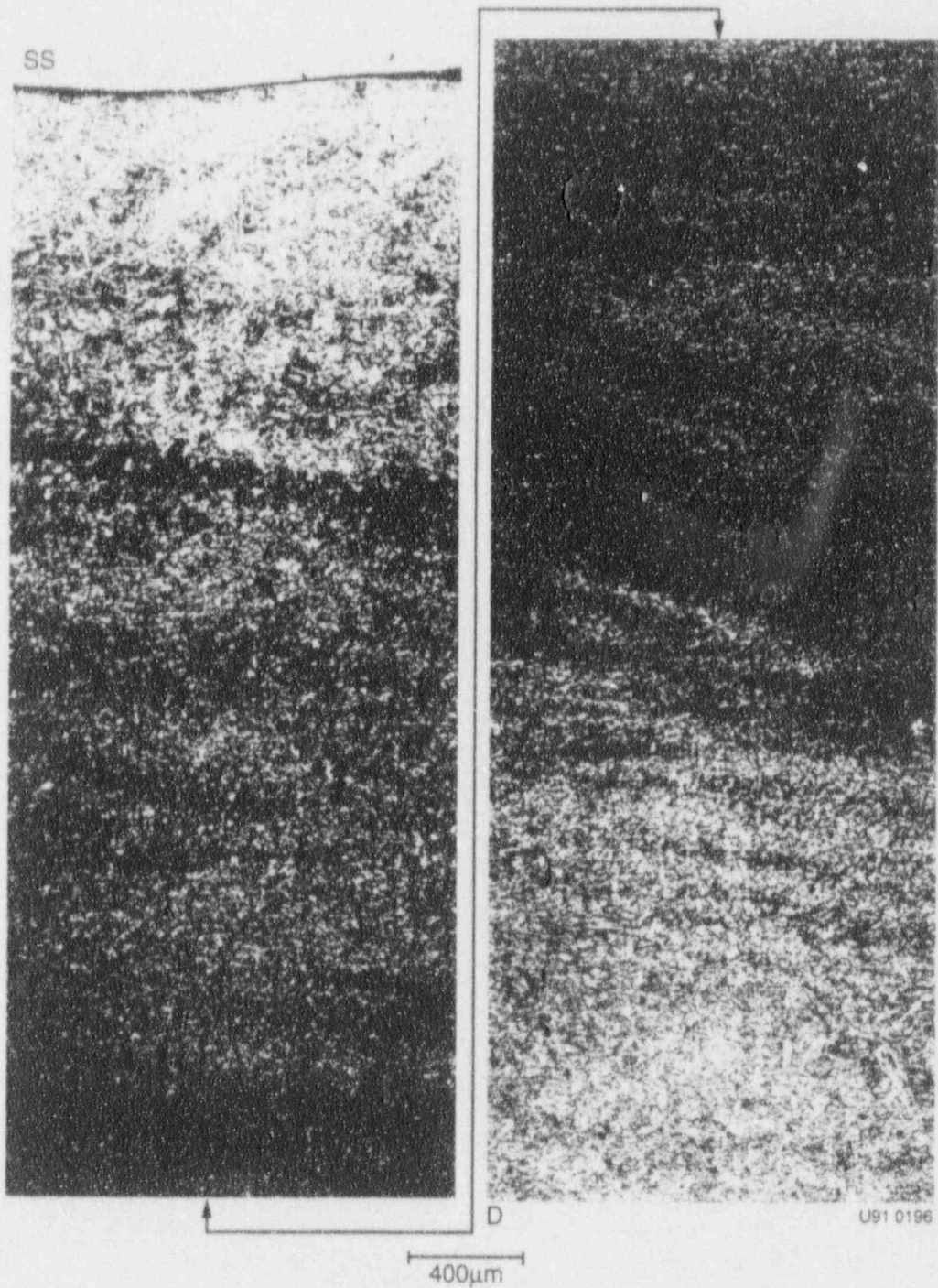


C

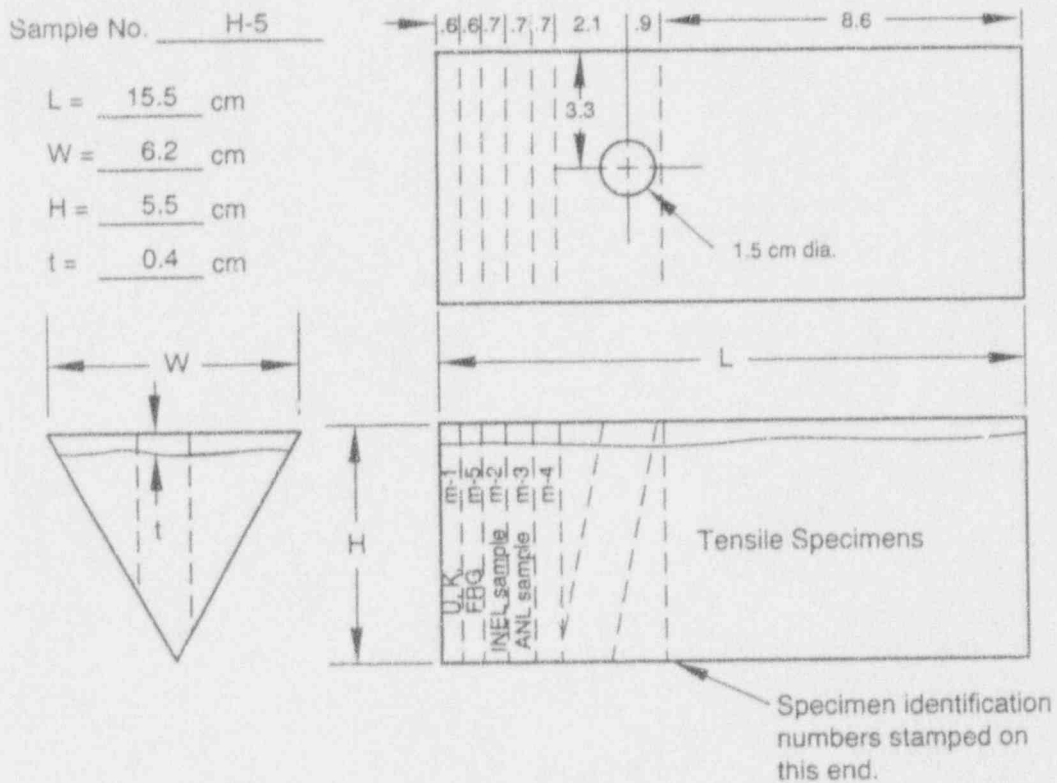
U91 0195

100µm

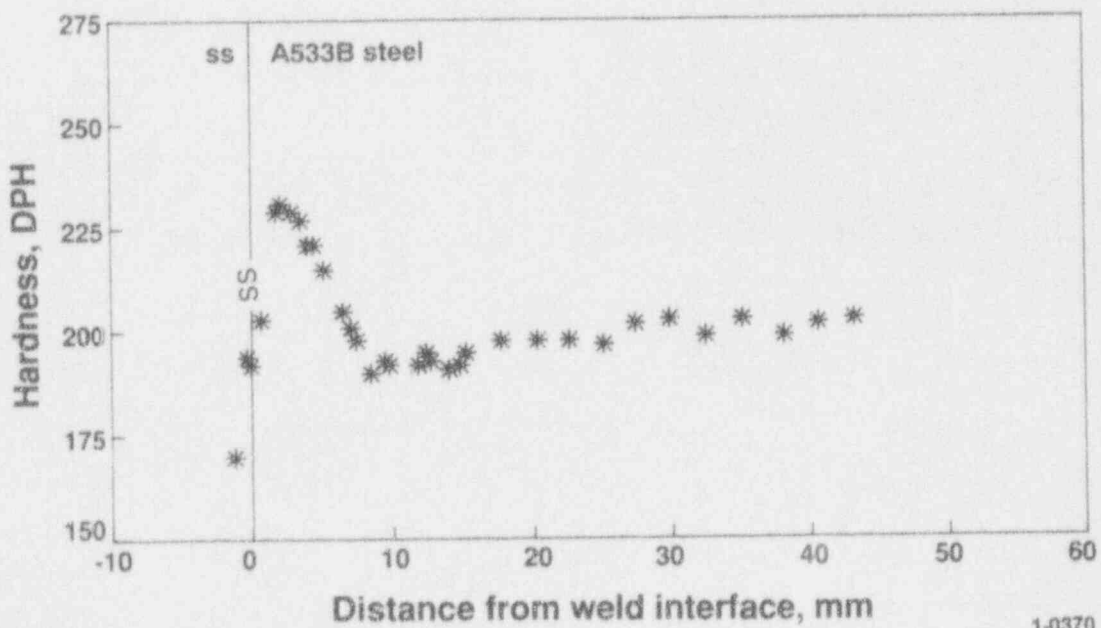
Sample H-4 (m-3)



Sectioning diagram of TMI-2 lower head sample H-5 and hardness profile of INEL subsection H-5 (m-2)

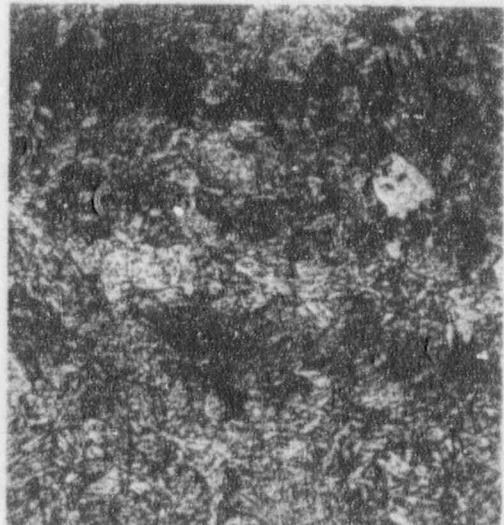
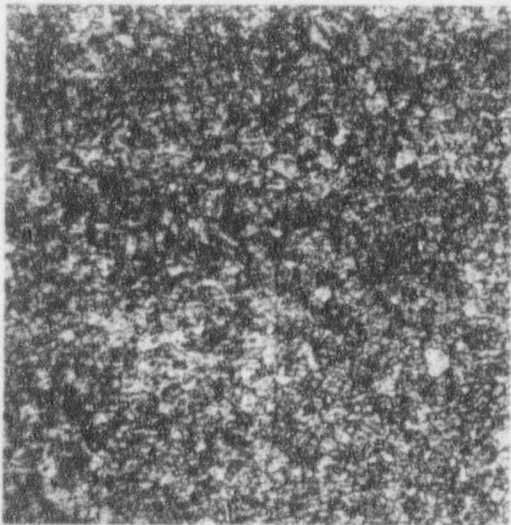
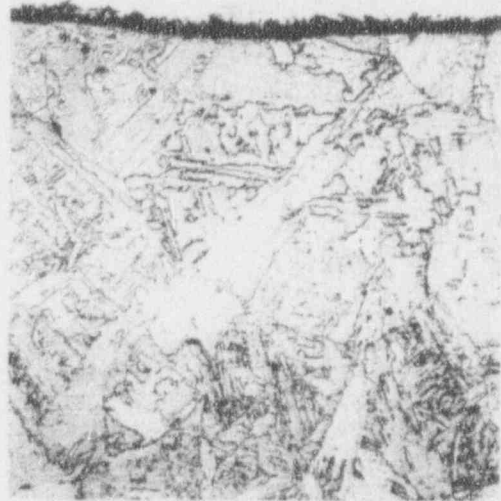
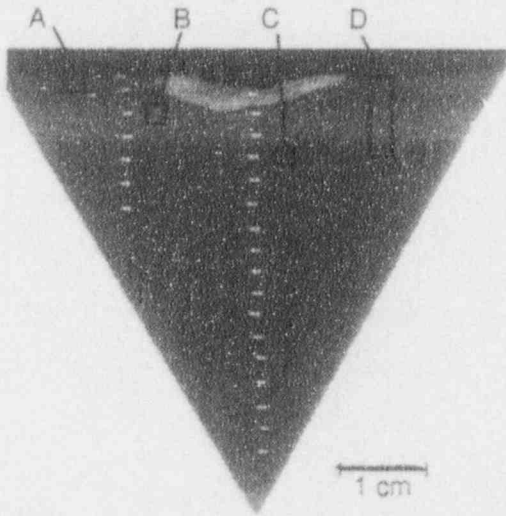


U91 0217



1-0370

Sample H-5 (m-2)



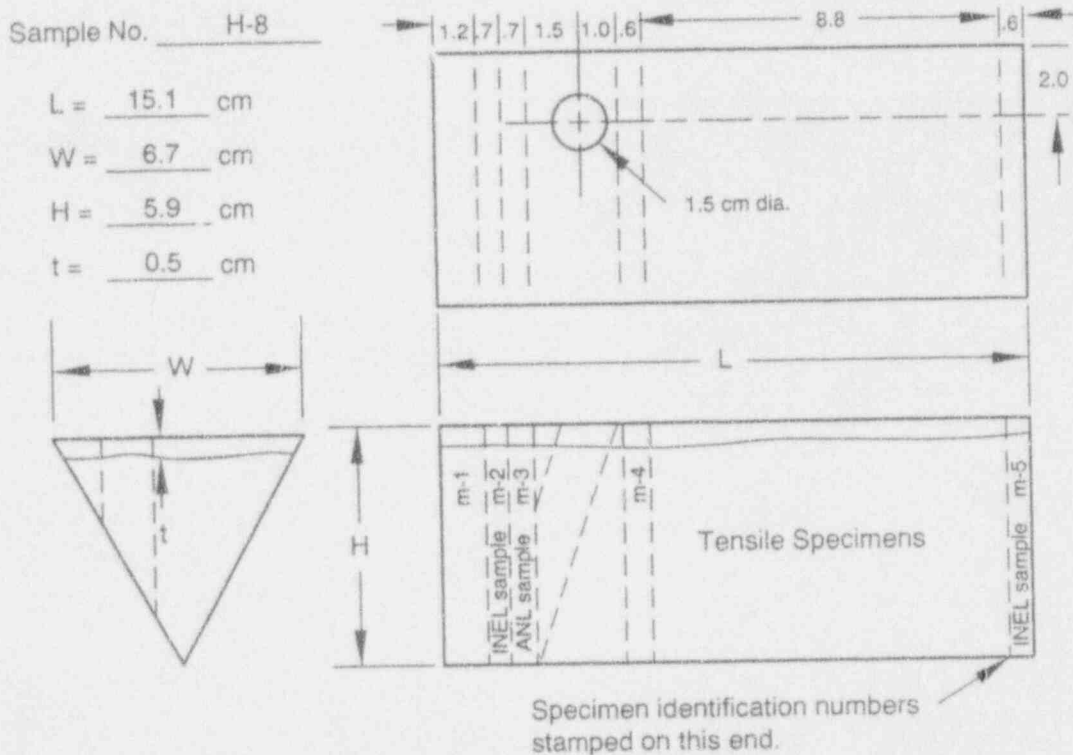
B

C

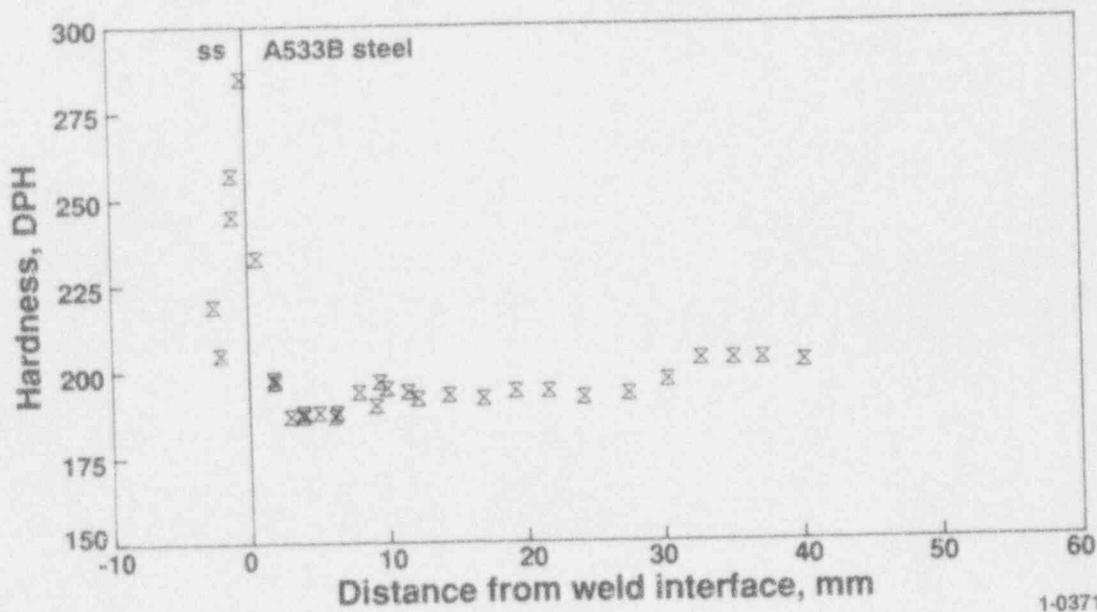
U91 0197

100μm

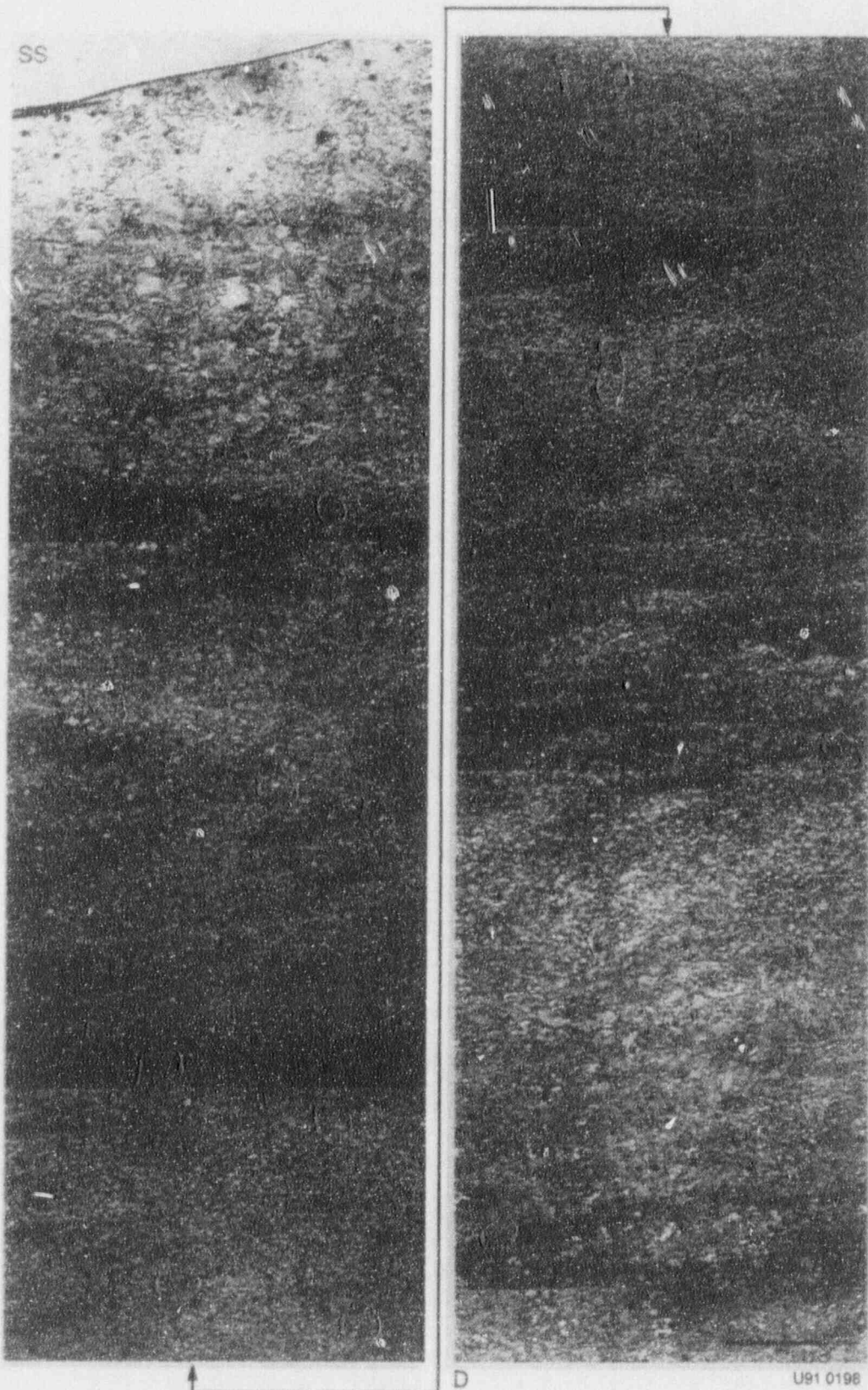
Sectioning diagram of TMI-2 lower head sample H-8 and hardness profile of INEL subsection H-8 (m-2)



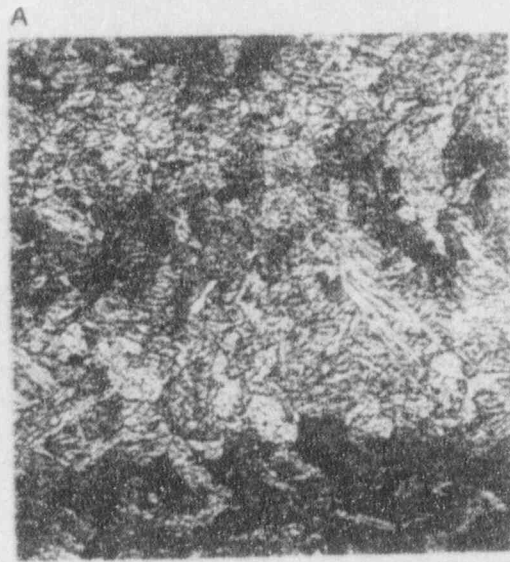
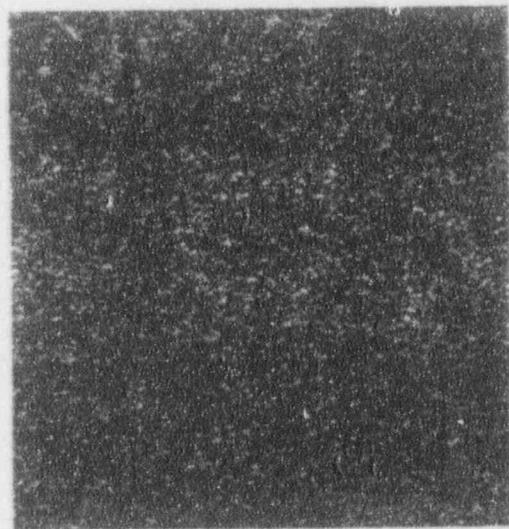
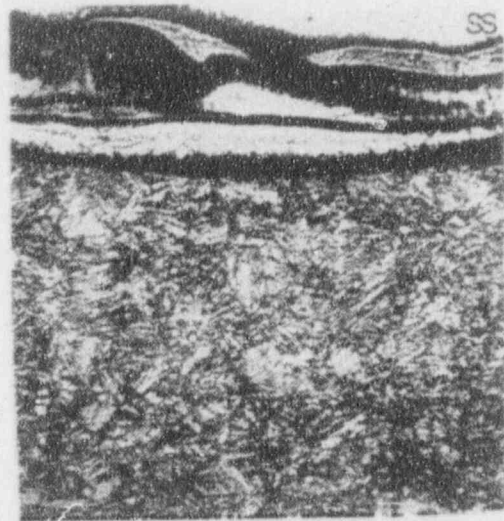
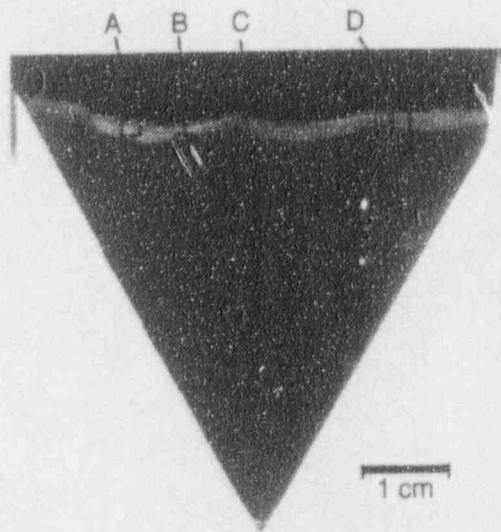
U91 0218



Sample H-5 (m-2)



Sample H-8 (m-2)

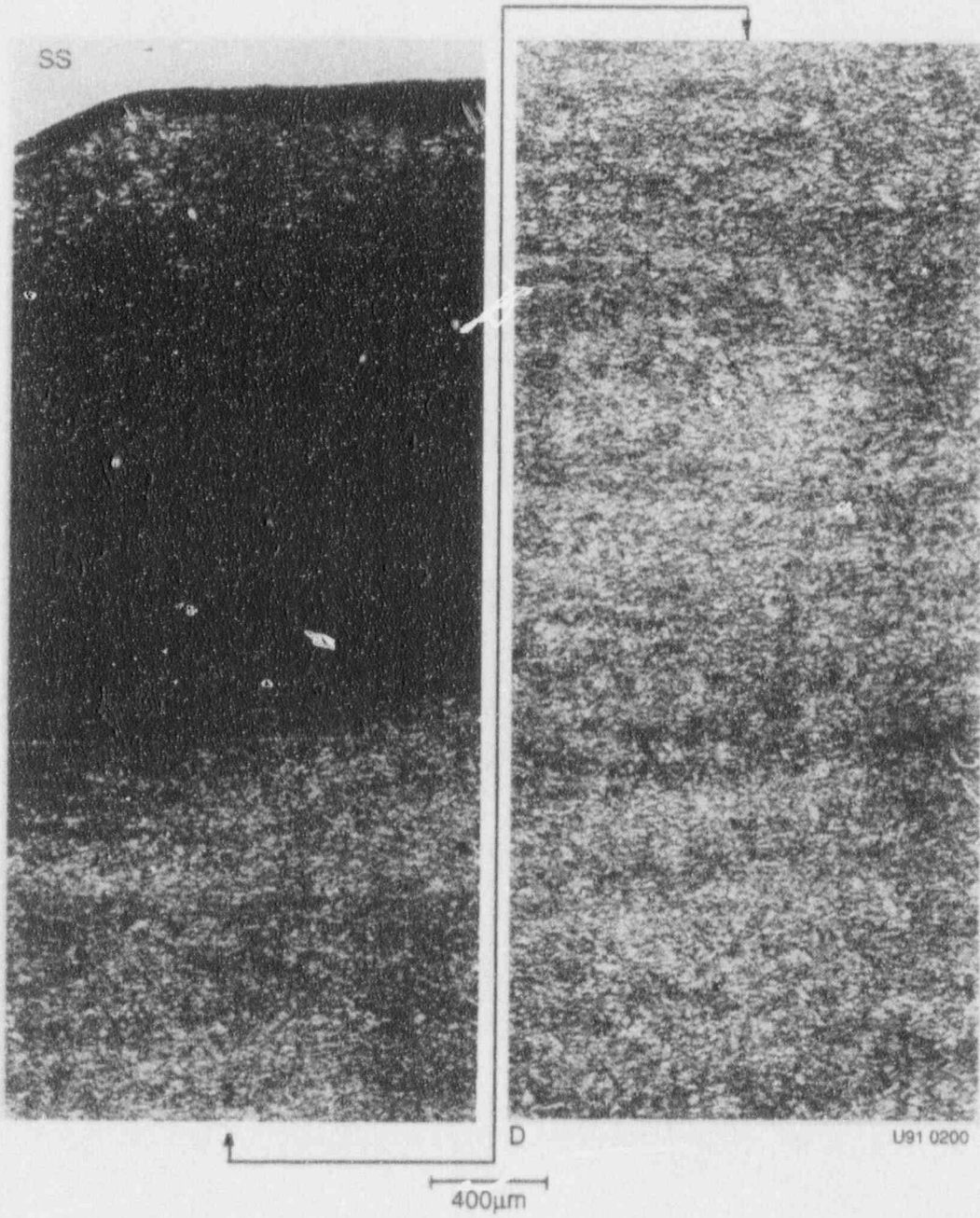


U91 0199

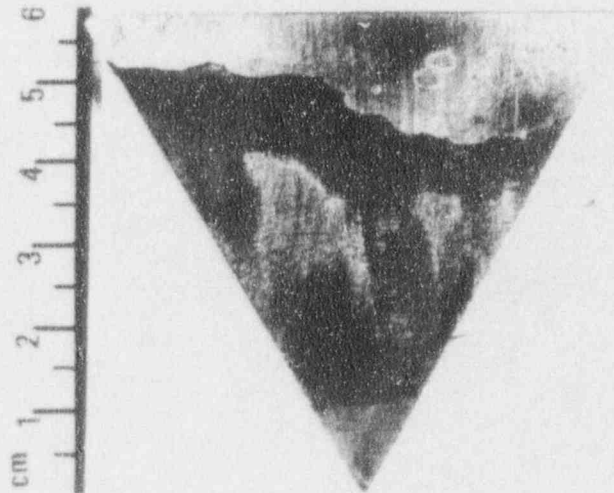
100μm



Sample H-8 (m-2)



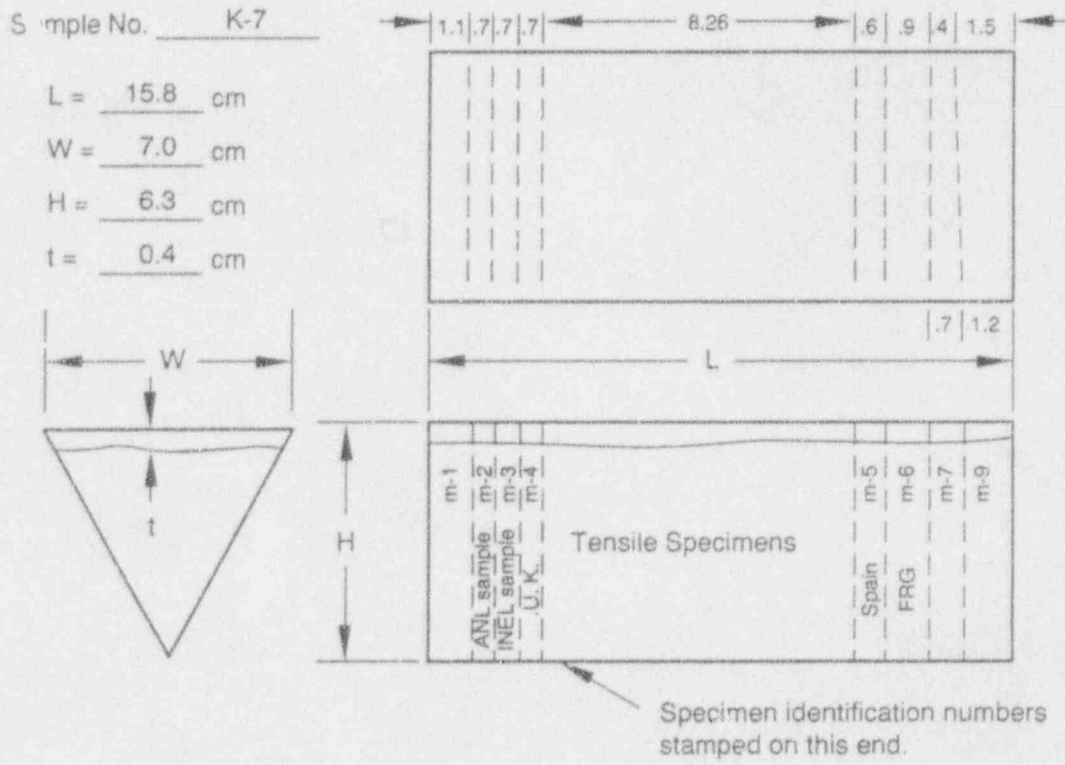
Sample H-8 (m-2)  
(Backside)



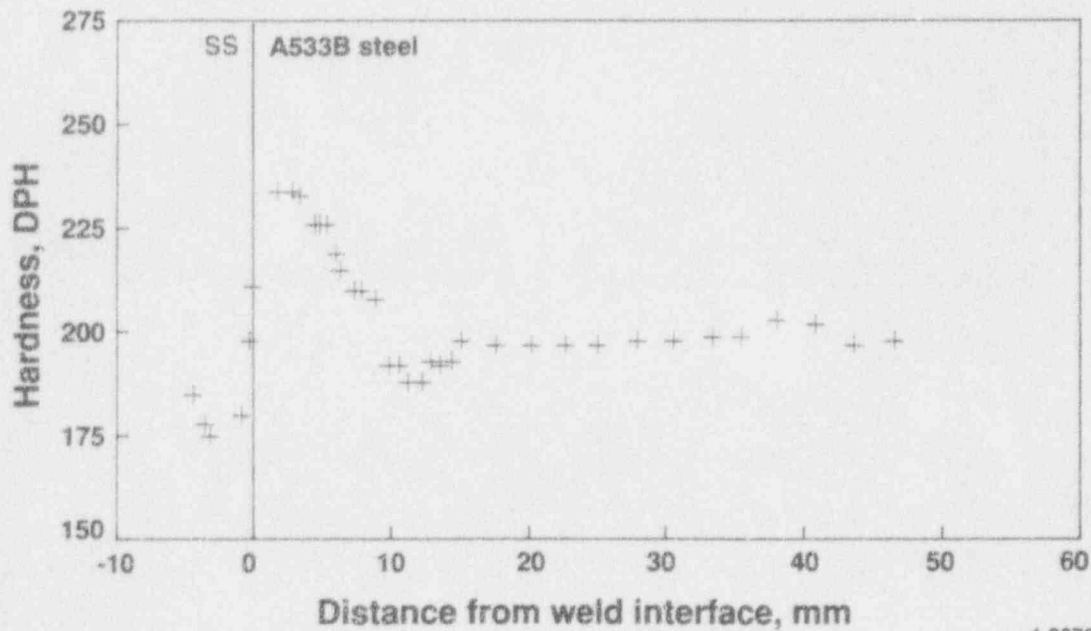
Near Nozzle Penetration

U81 0201

Sectioning diagram of TMI-2 lower head sample K-7 and hardness profile of INEL subsection K-7 (m-3)

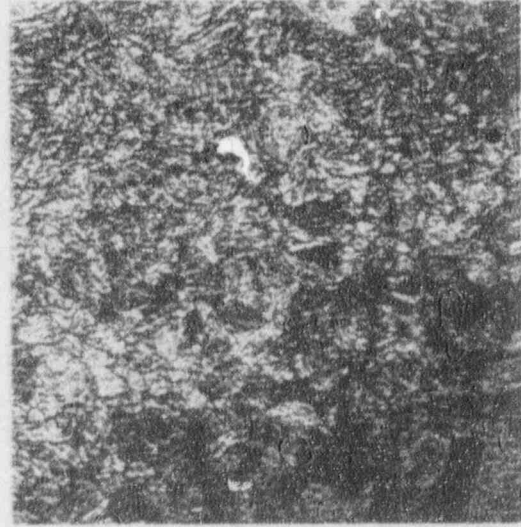
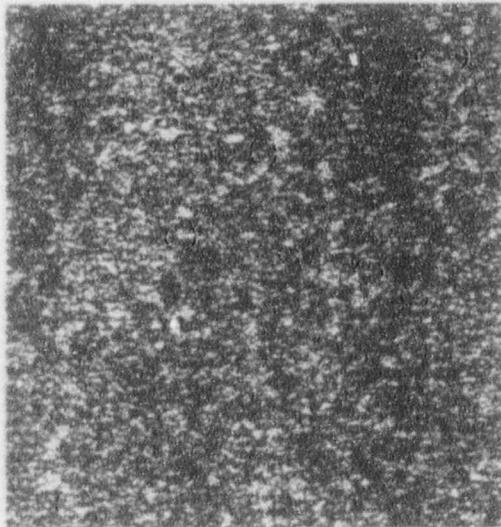
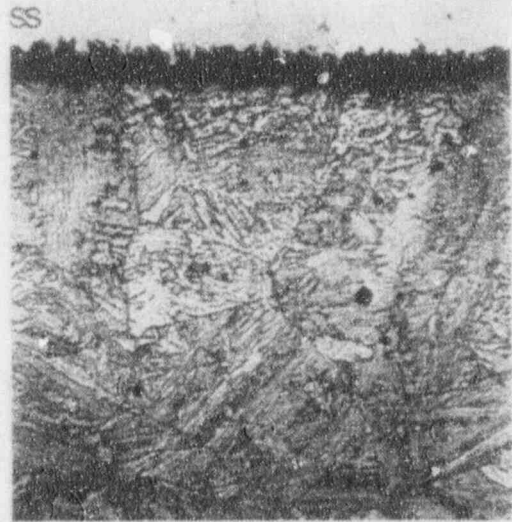
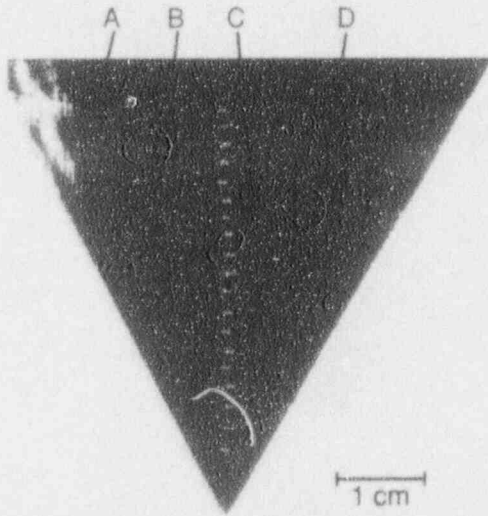


U91 0219



1-0372

Sample K-7 (m-3)



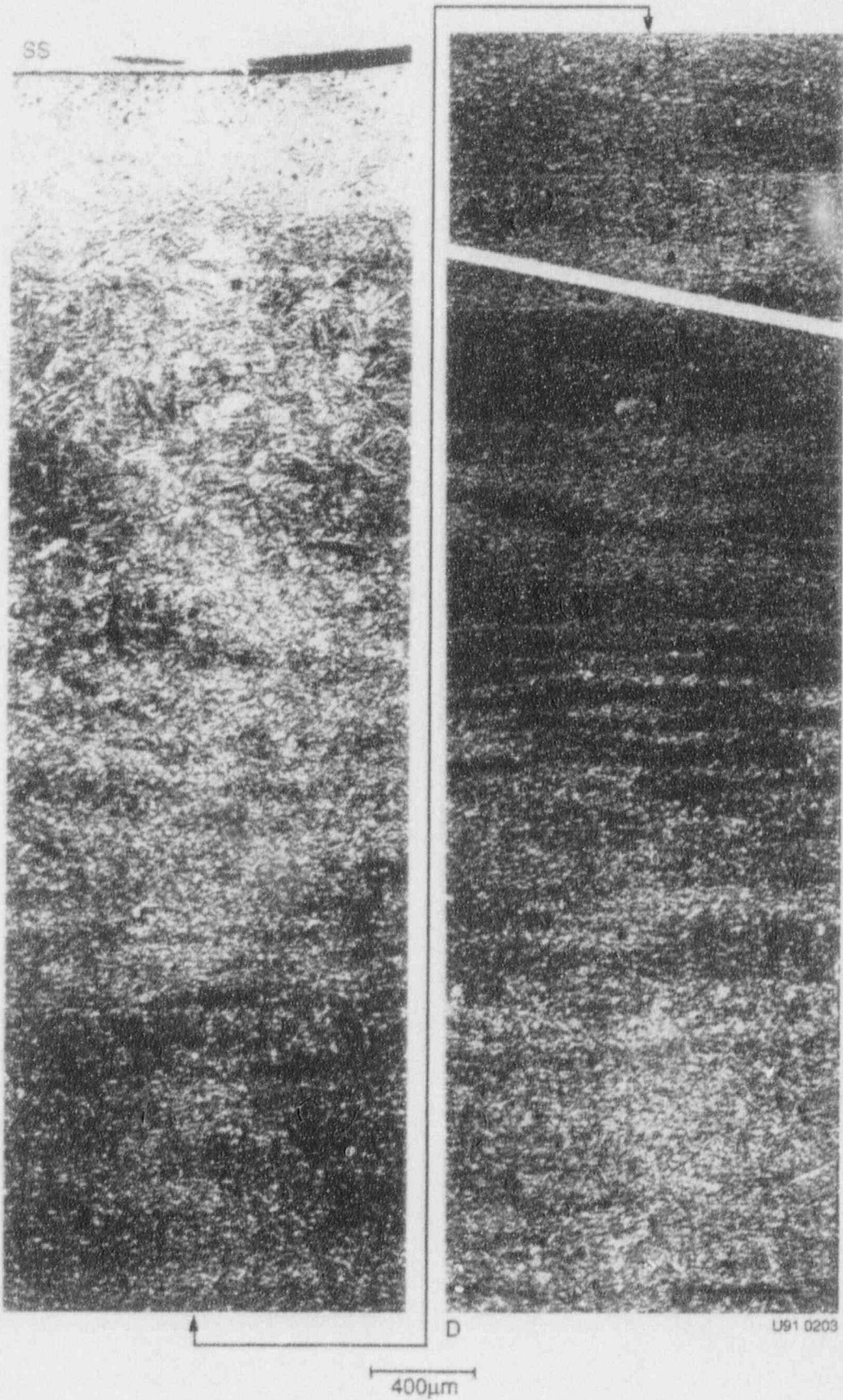
B

C

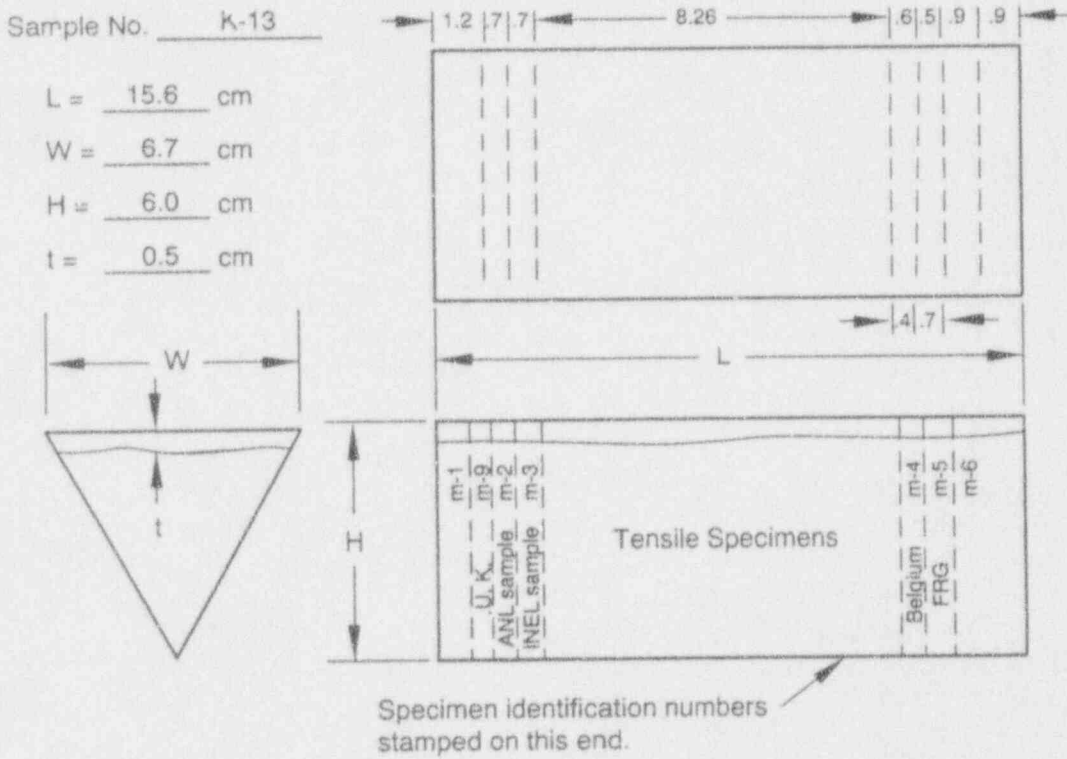
U91 0202

100μm

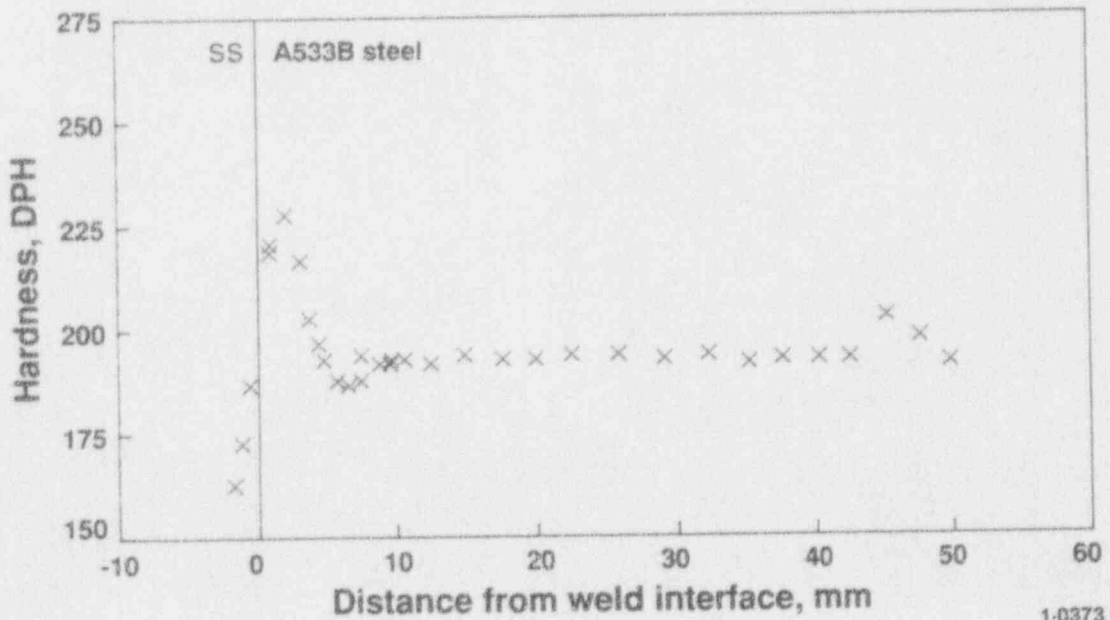
Sample K-7 (m-3)



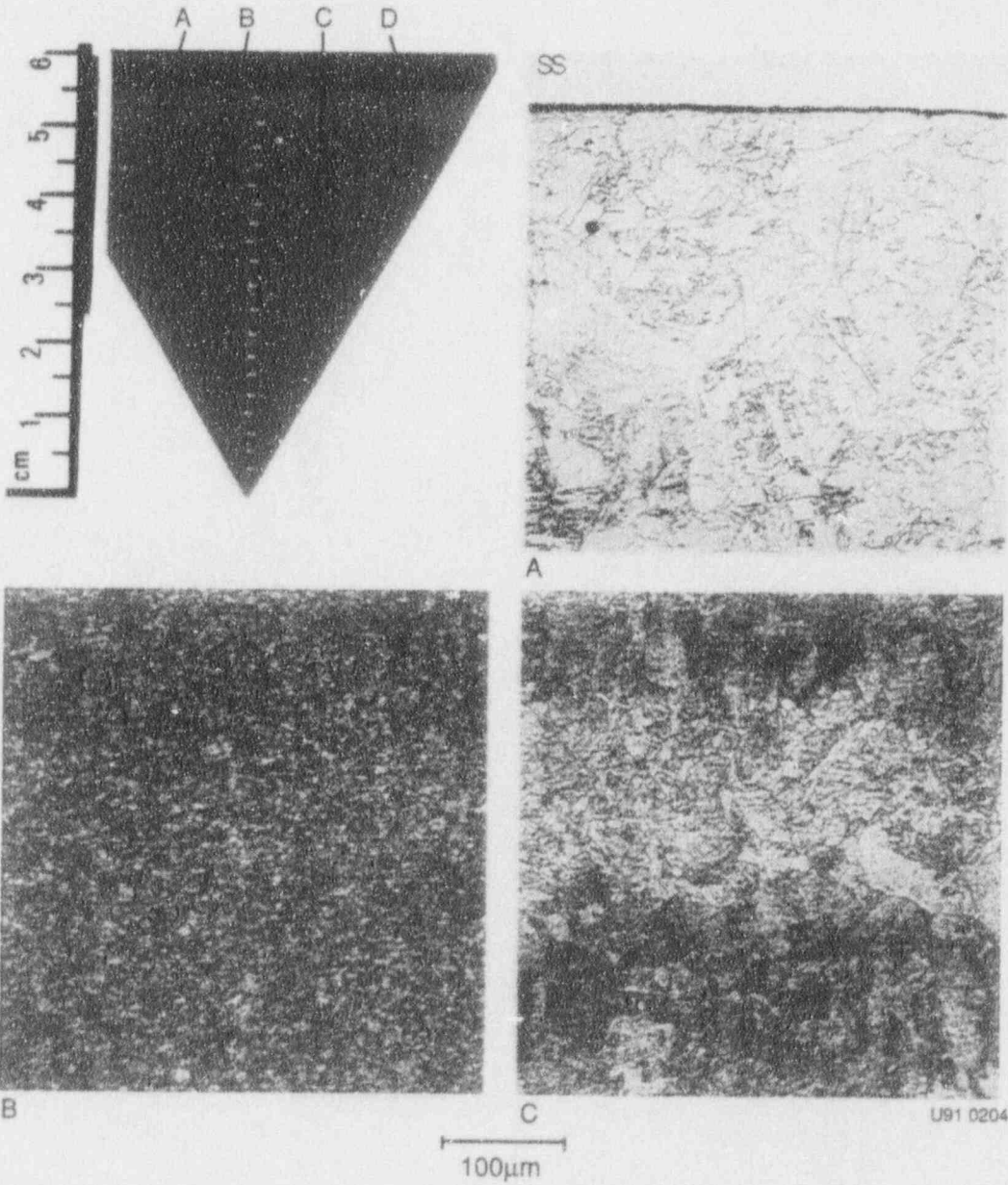
Sectioning diagram of TMI-2 lower head sample K-13 and hardness profile of INEL subsection K-13 (m-3)



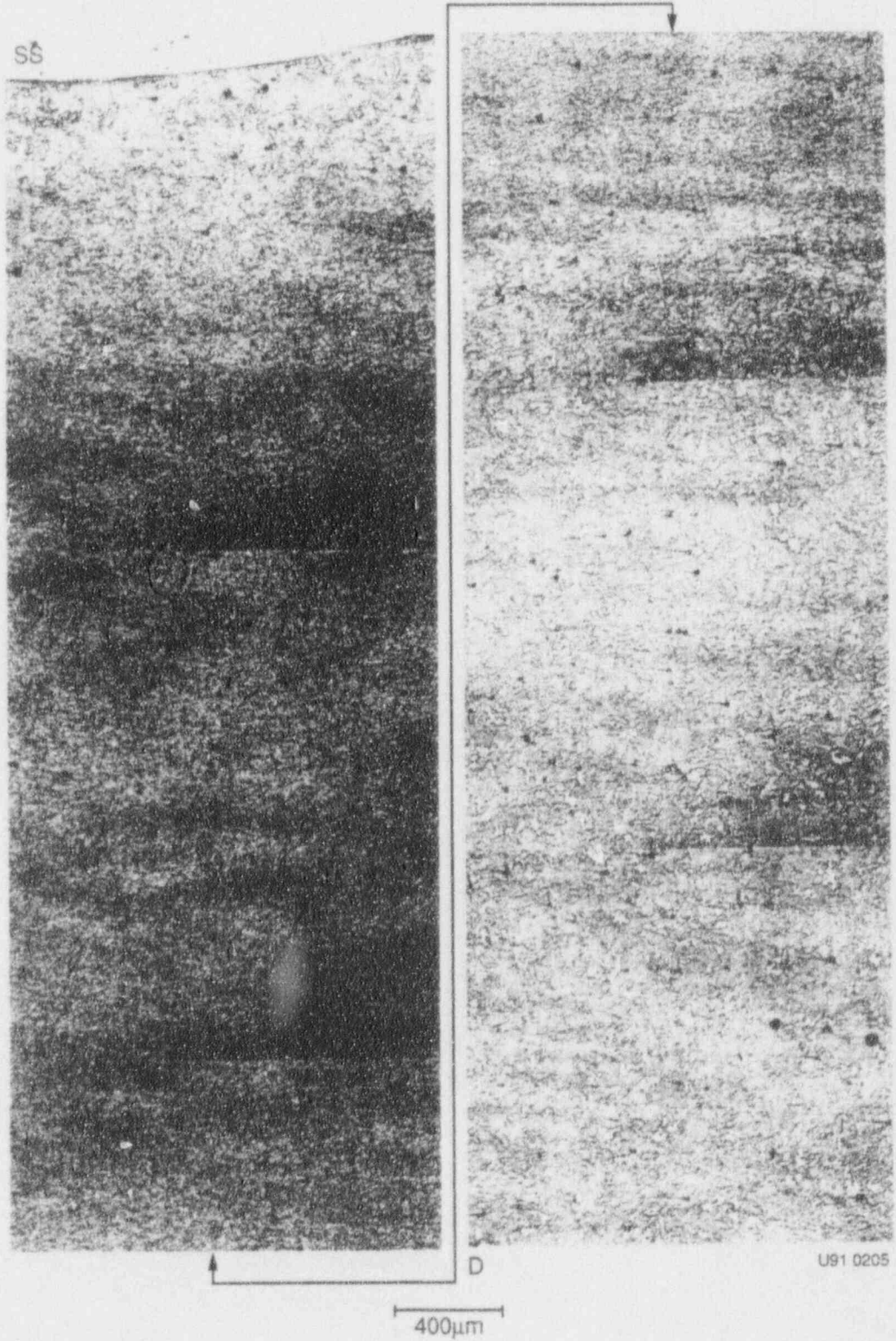
U91 0220



Sample K-13 (m-3)

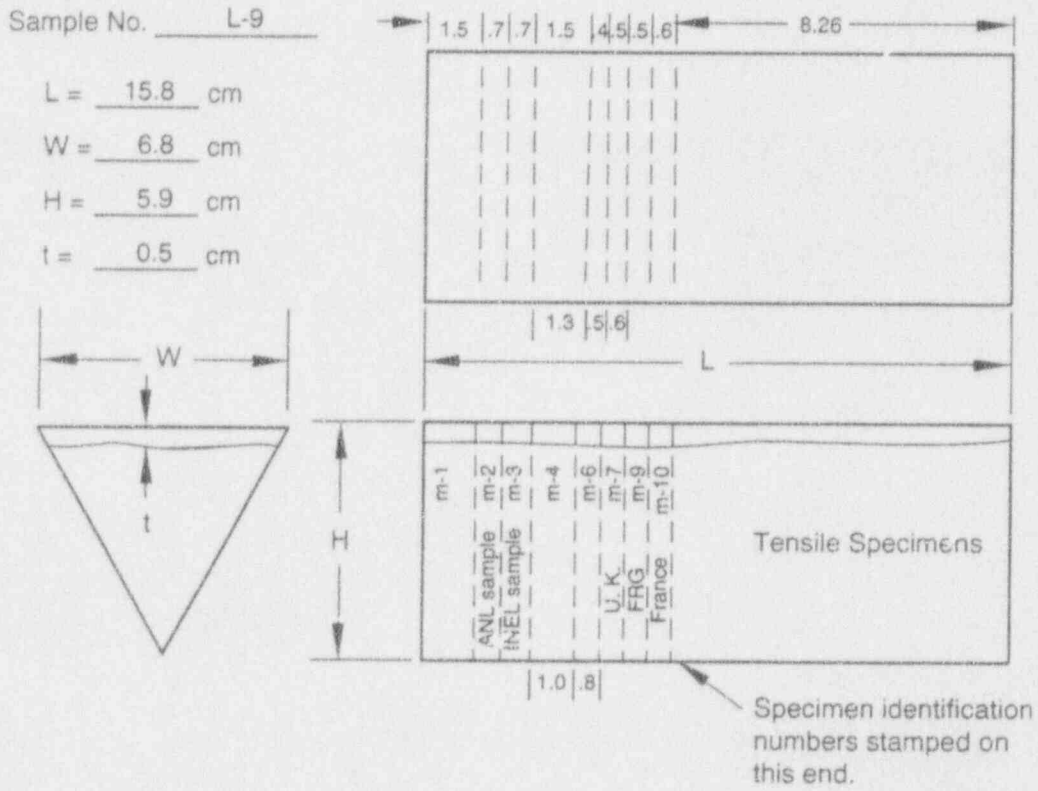


Sample K-13 (m-3)

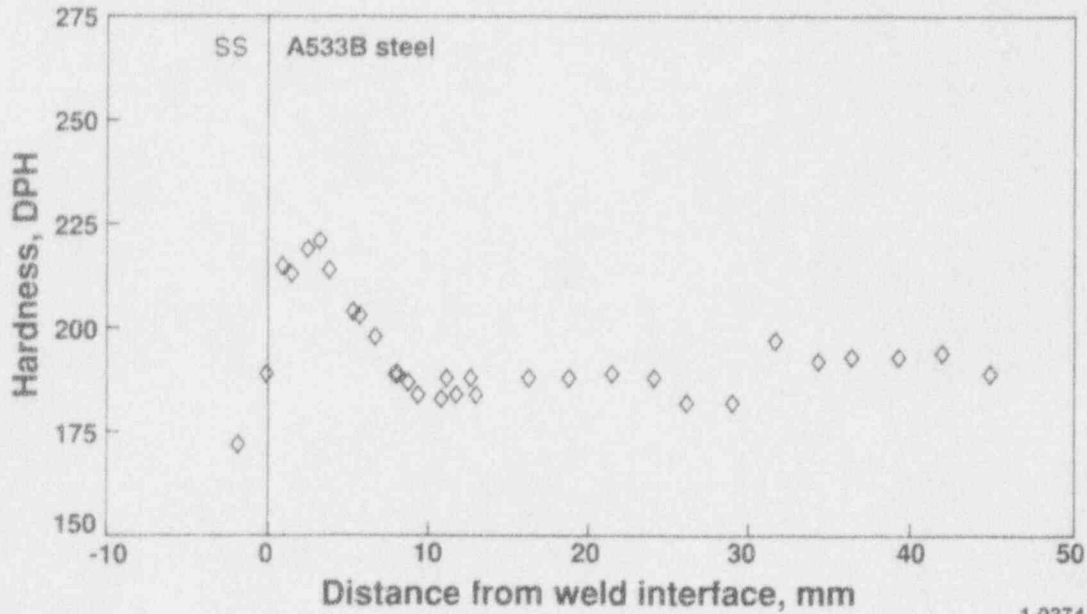




Sectioning diagram of TMI-2 lower head sample L-9 and hardness profile of INEL subsection L-9 (m-3)

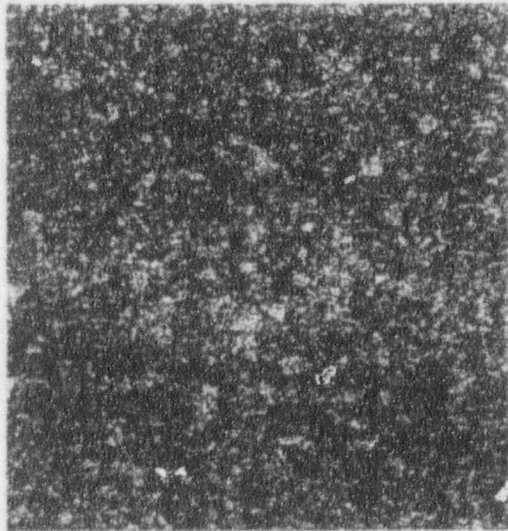
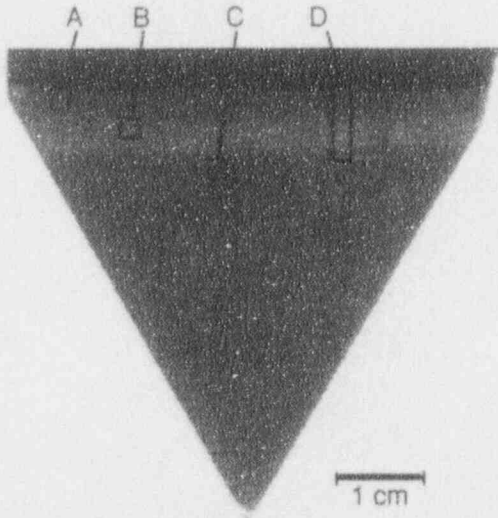


U91 0221



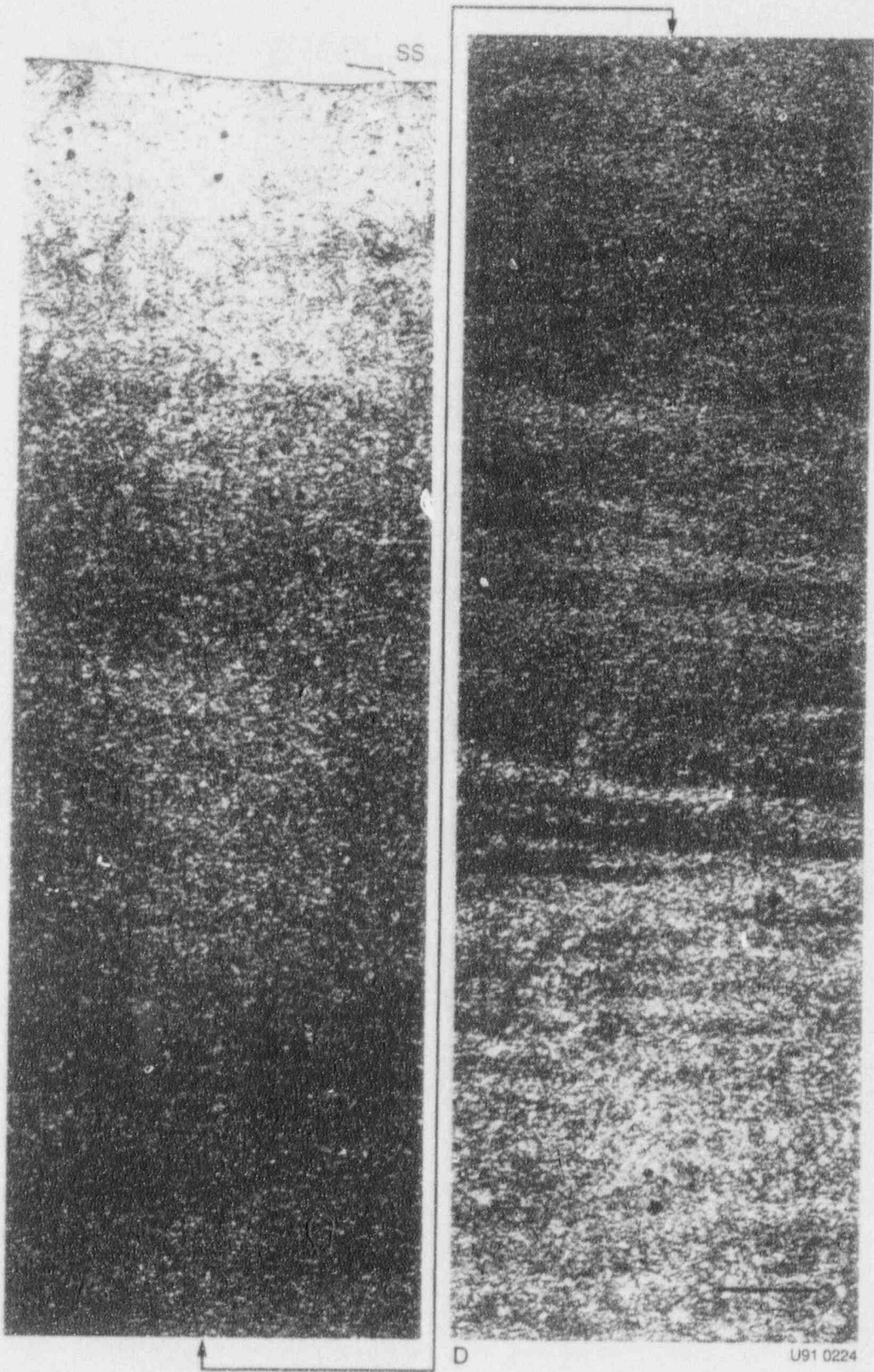
1-0374

Sample L-9 (m-3)

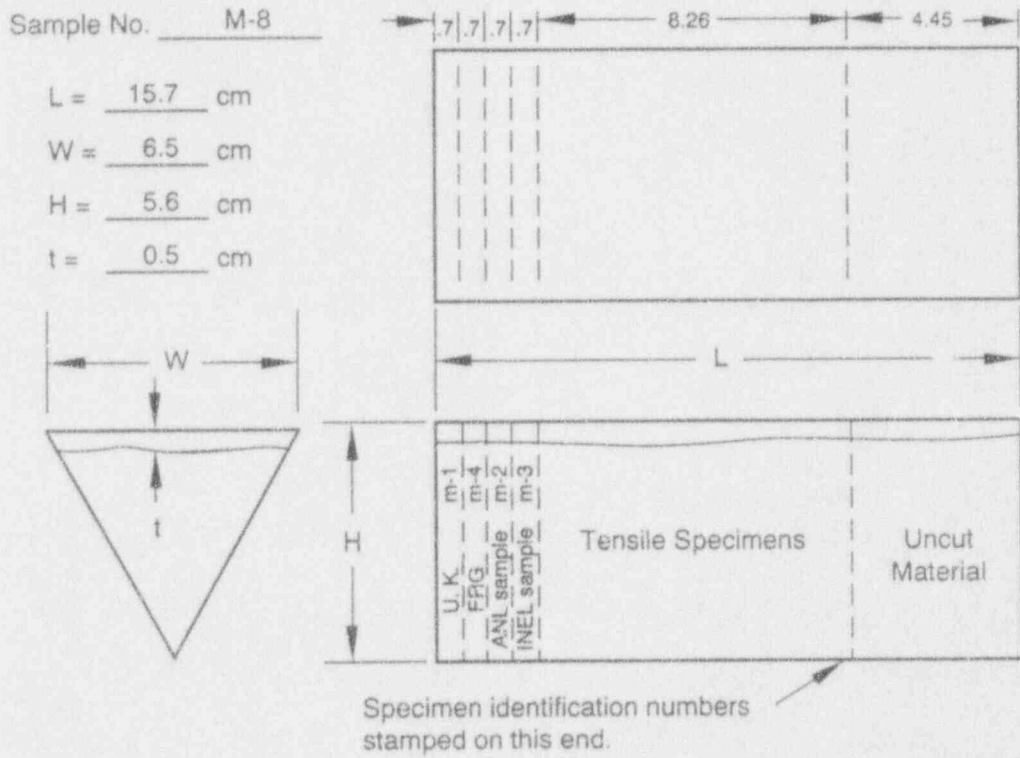


100µm

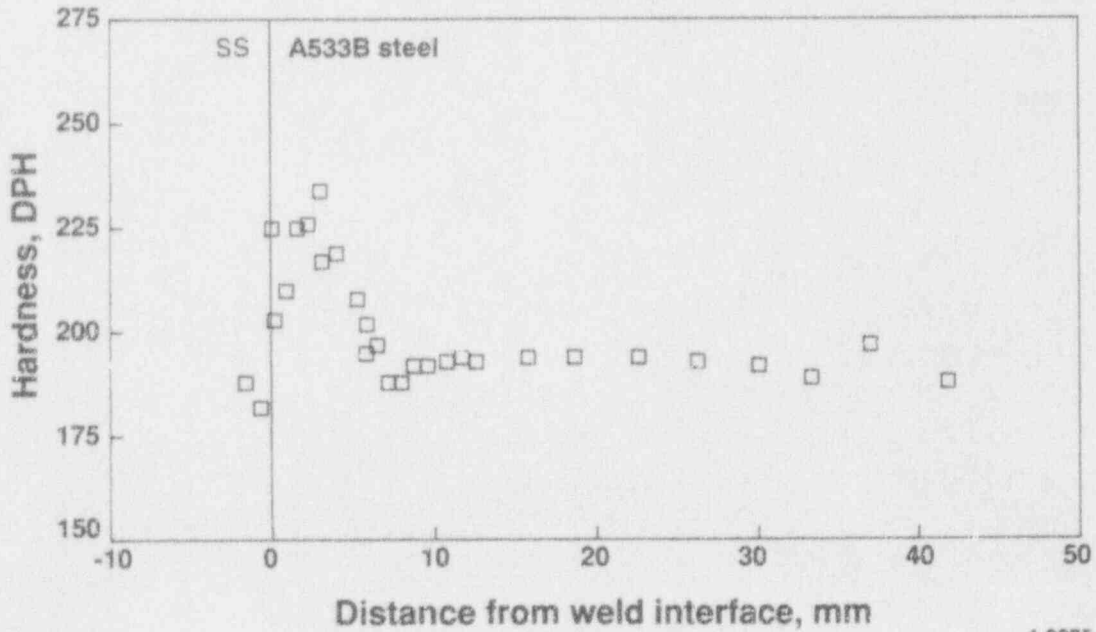
Sample L-9 (m-3)



Sectioning diagram of TMI-2 lower head sample M-8 and hardness profile of INEL subsection M-8 (m-3)

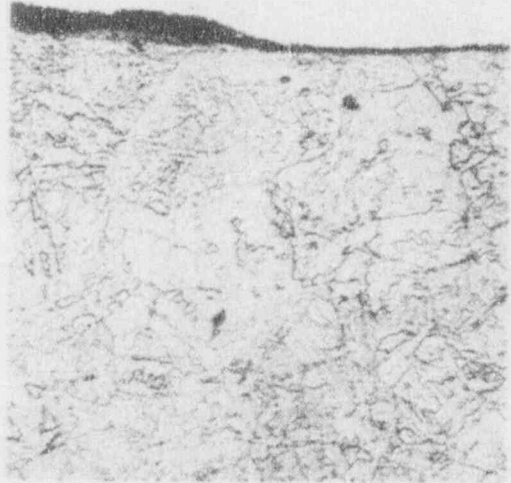
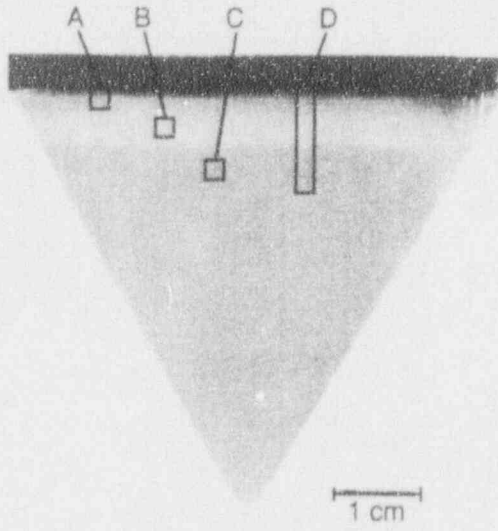


U91 0222

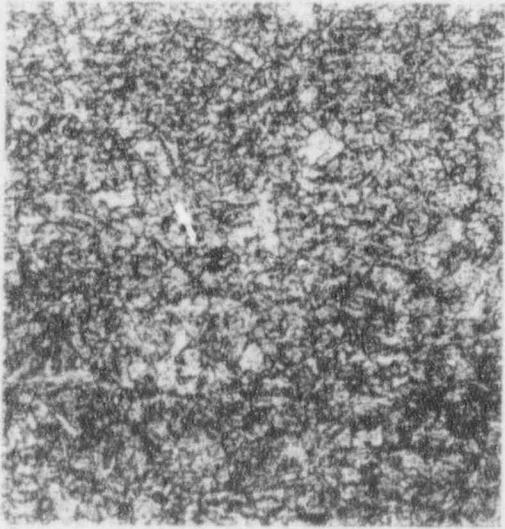


1-0375

Sample M-8 (M-3)



A



B

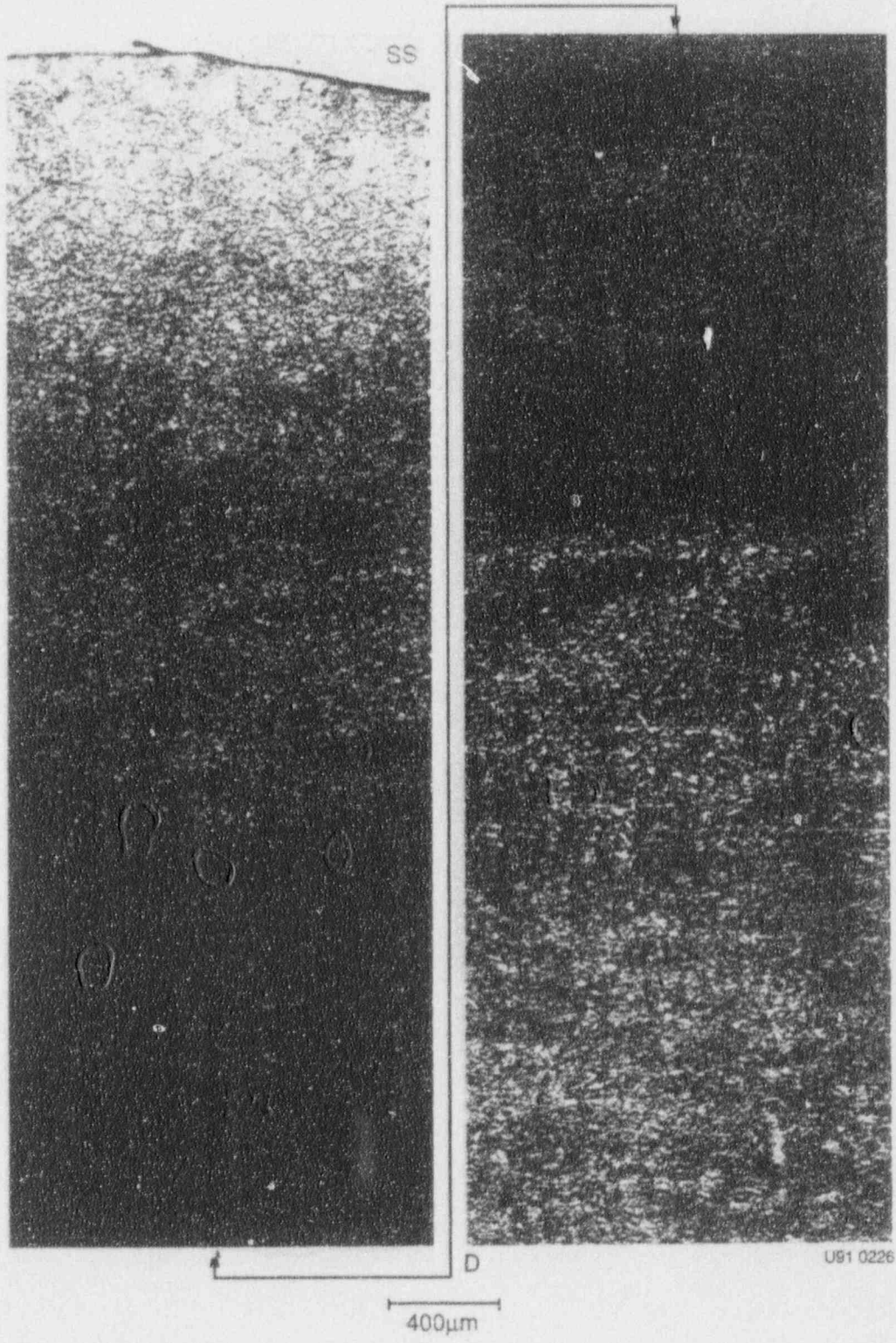


C

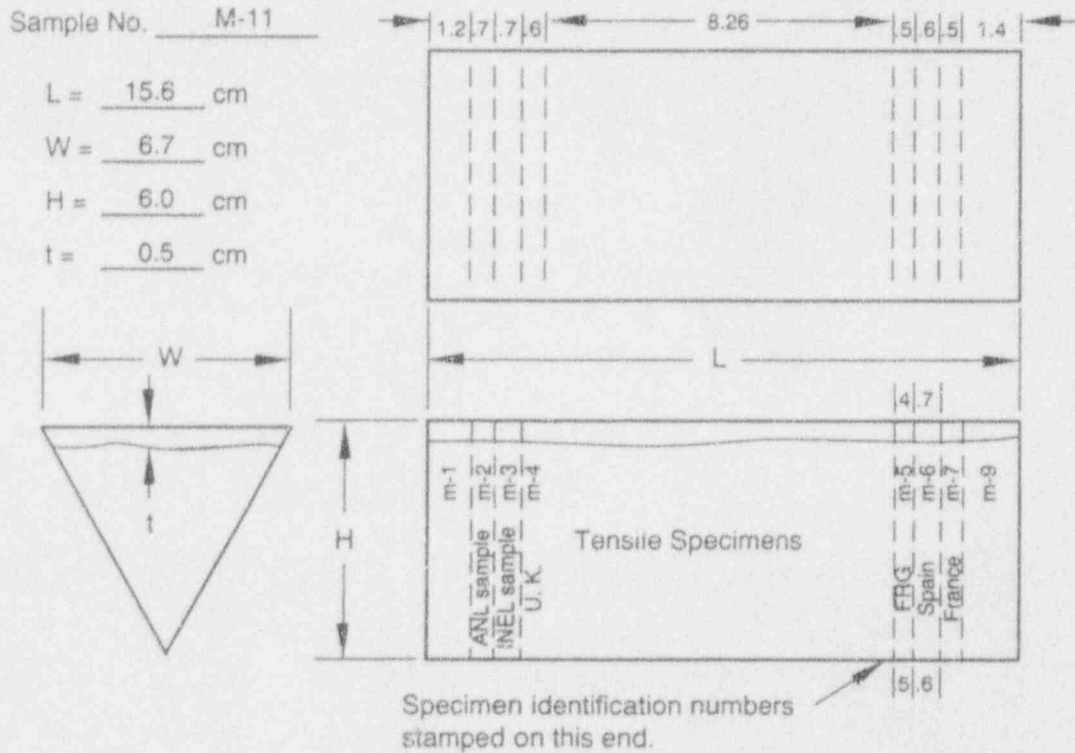
100 $\mu$ m

U91 0225

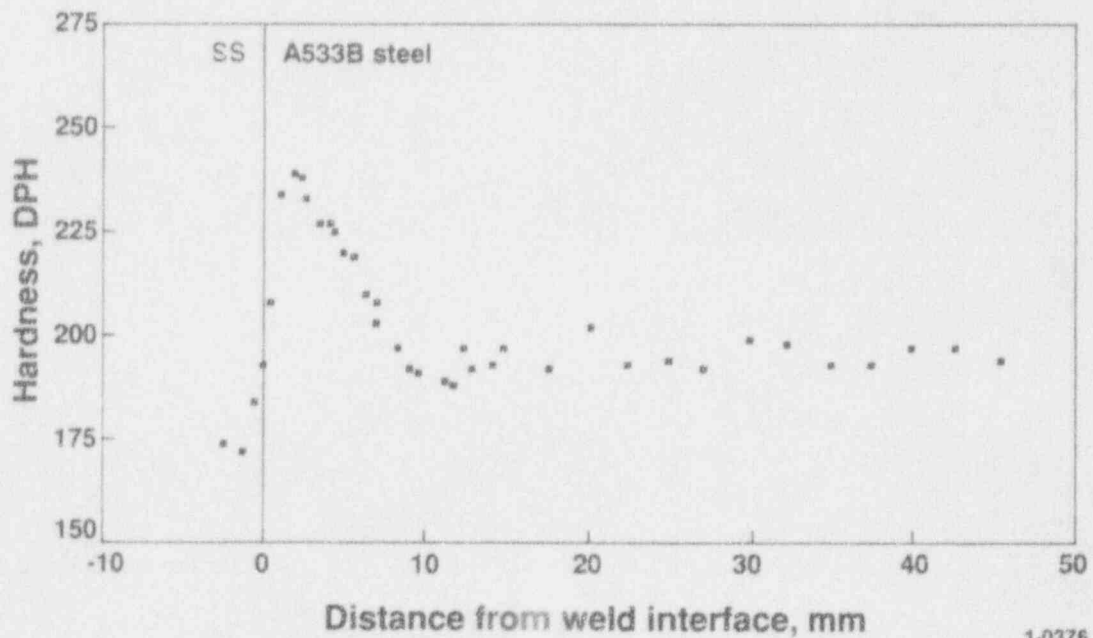
Sample M-8 (m-3)



Sectioning diagram of TMI-2 lower head sample M-11 and hardness profile of INEL subsection M-11 (m-3)

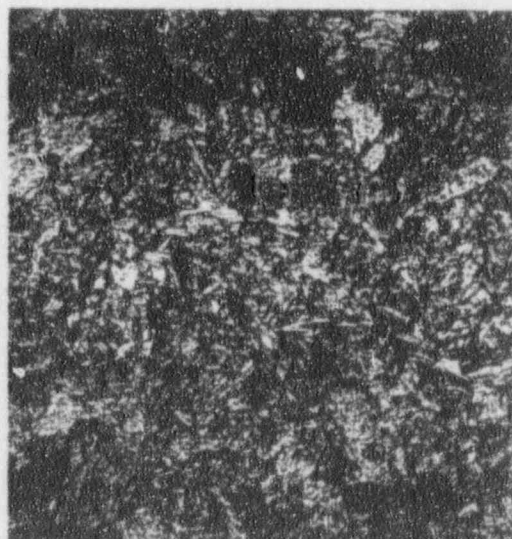
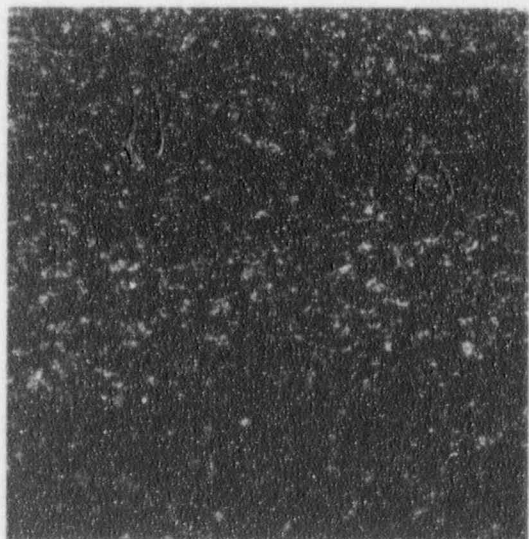
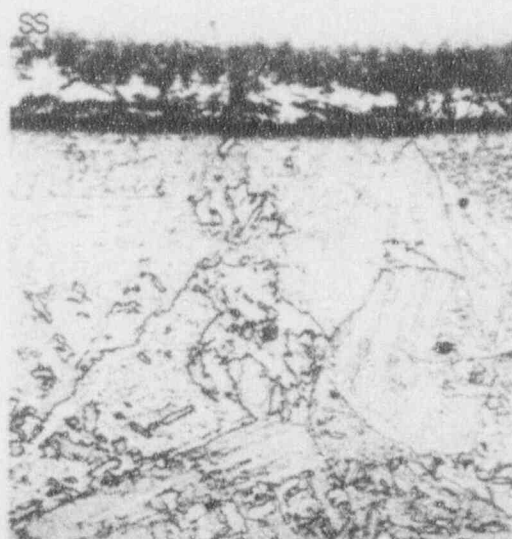
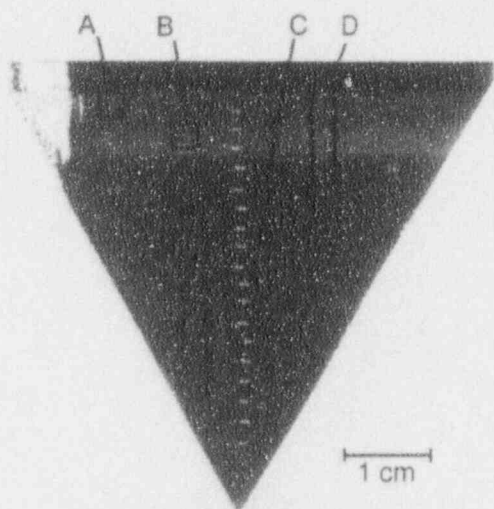


U91 0223



1-0376

Sample M-11 (m-3)

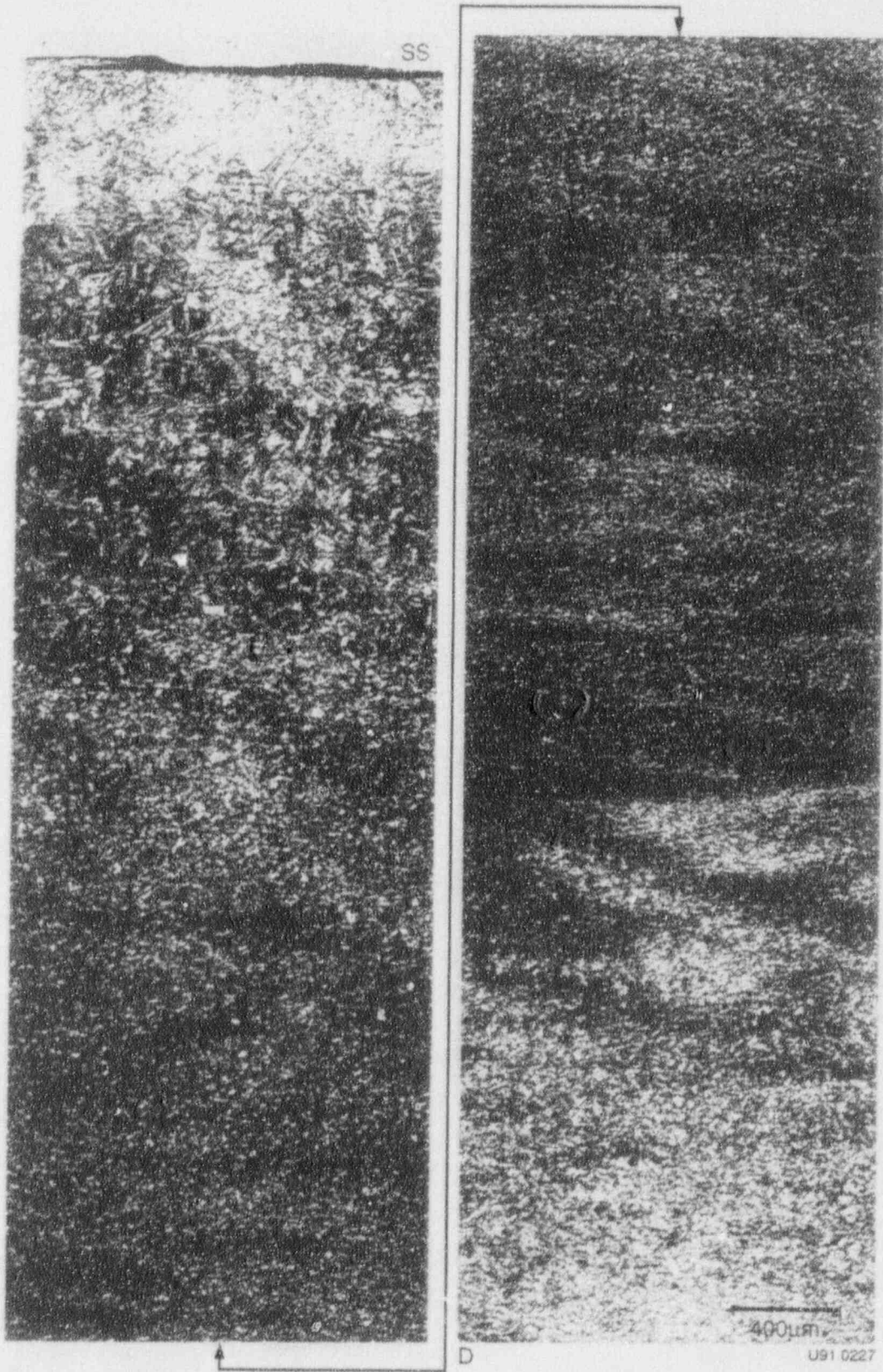


100μm

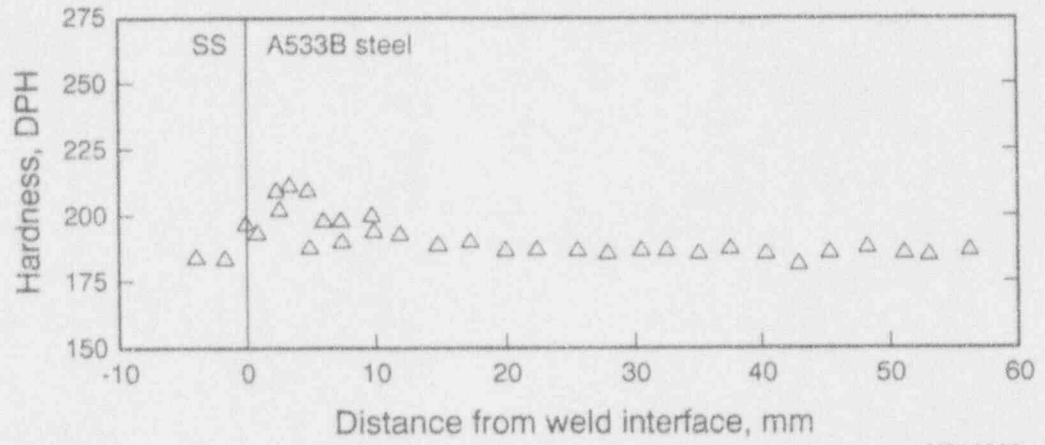
U91 0182



Sample M-11 (m-3)

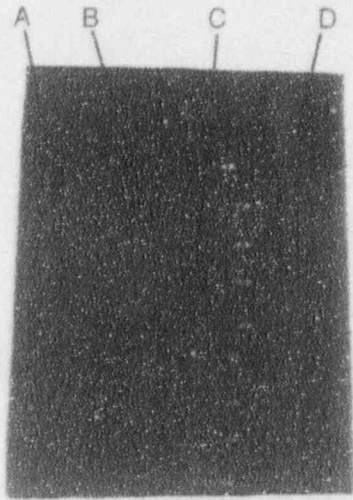


Hardness profile of Midland Archive material



U91 0265

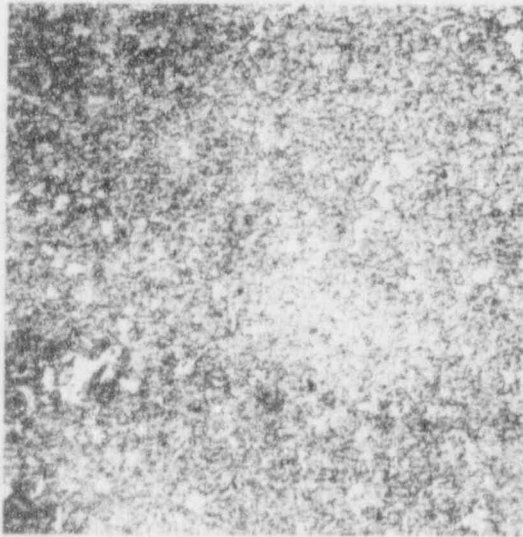
Midland Archive Material



1 cm



A



B

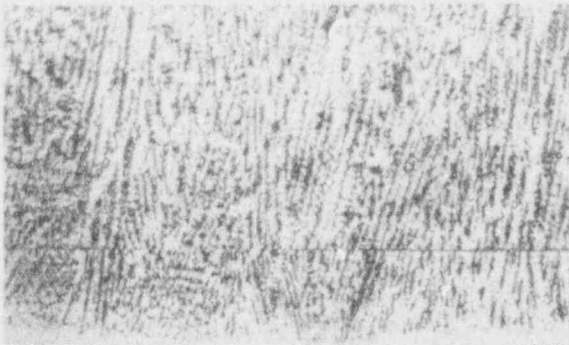


C

100µm

U91 0228

Midland Archive Material



D

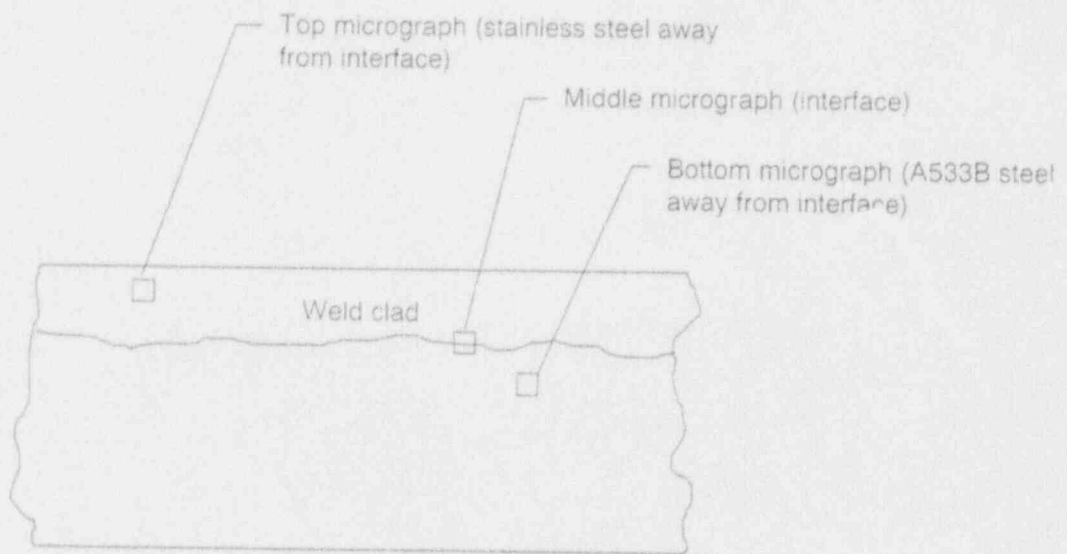
400µm

A-50

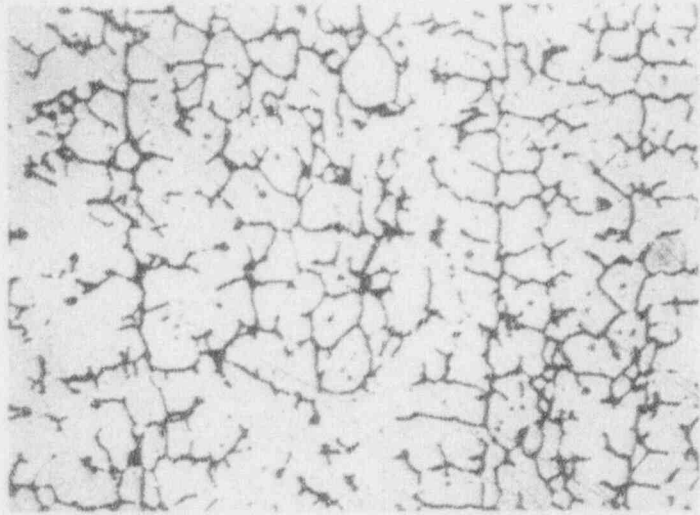
U91 0264

## **Appendix B**

### **Microstructure of First Series of Midland Archive Samples Given Accident-Simulated Heat Treatments**



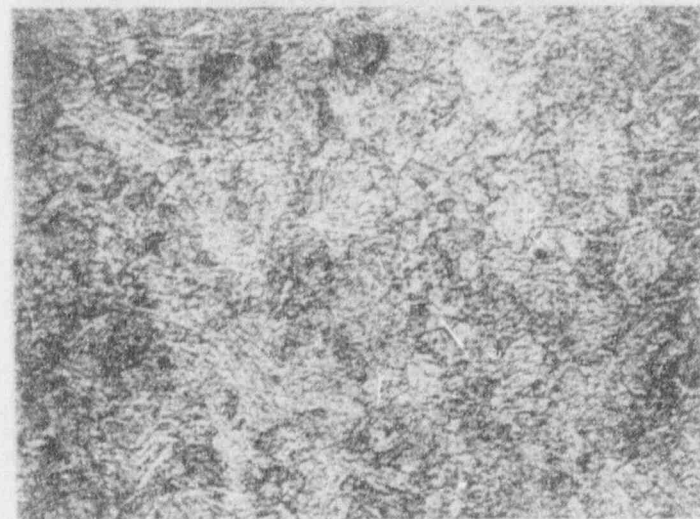
U91 0229



SS



Interface



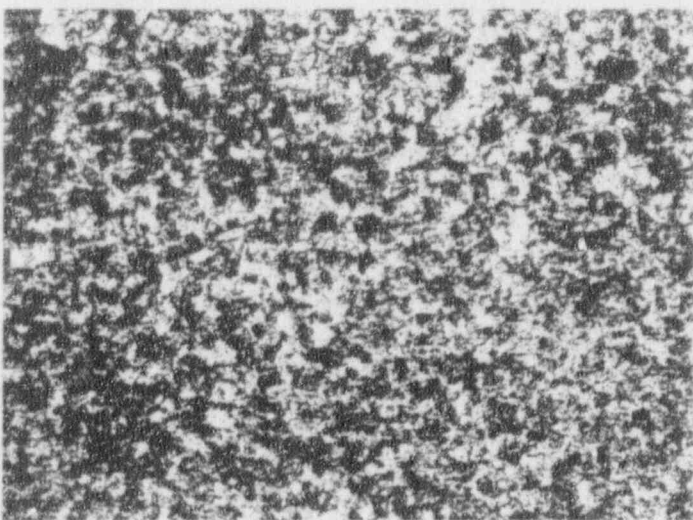
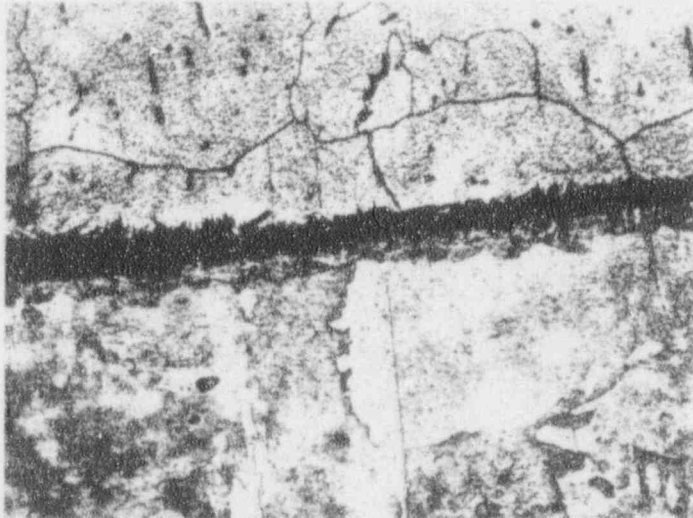
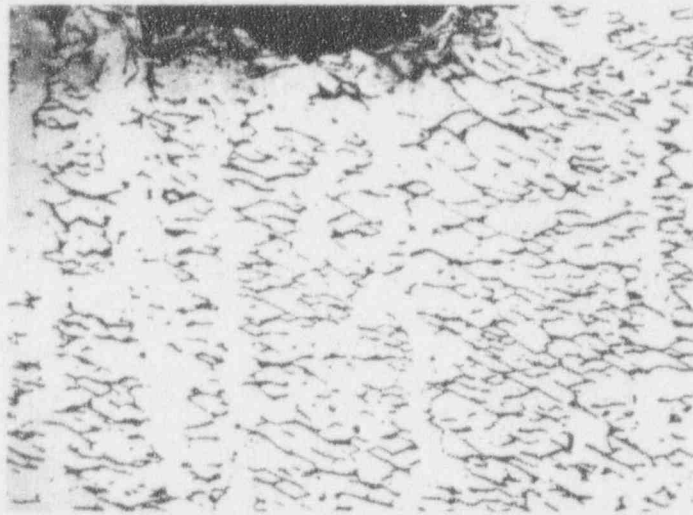
A533B steel

Midland archive material (as fabricated)

U91 0230

50µm

B-4

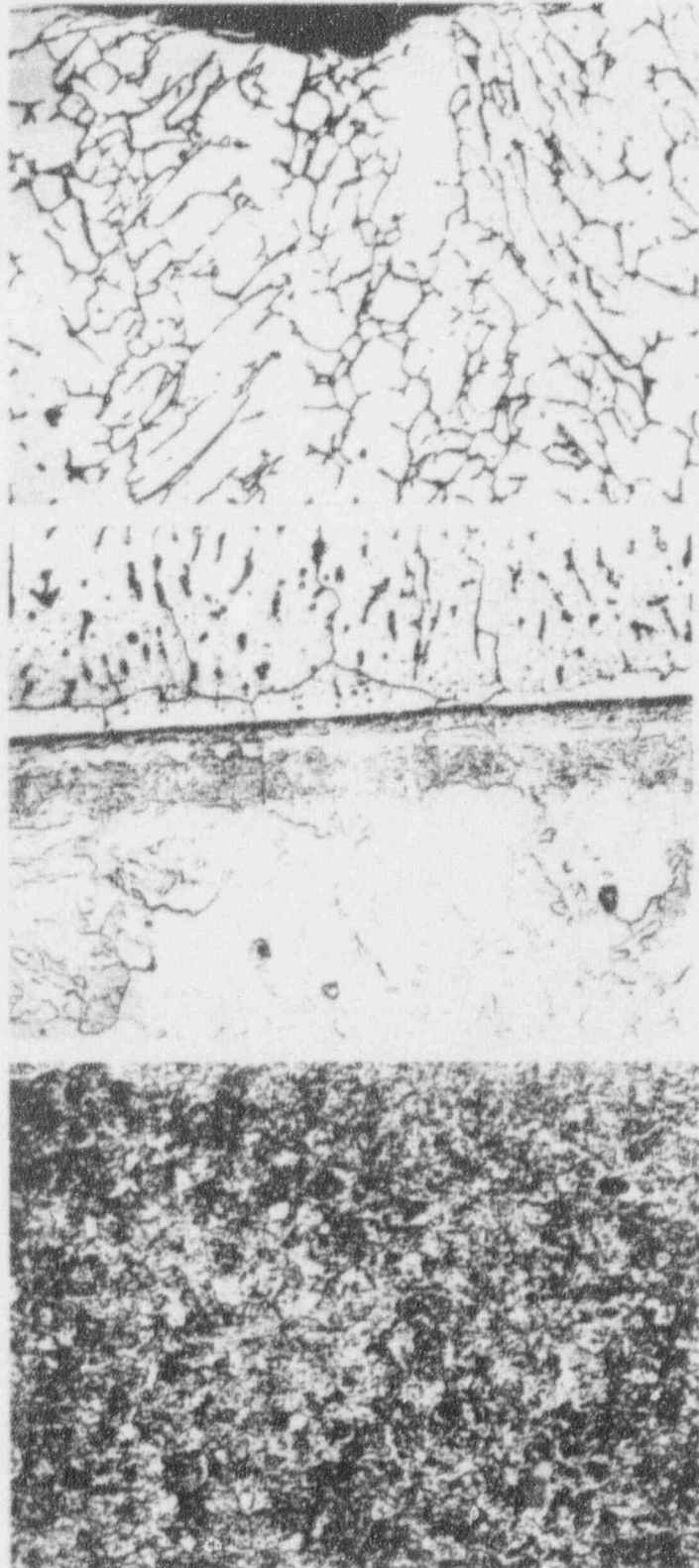


TMIG-15 (800°C/1 min.)

U91 0231

—|  
50µm  
B-5

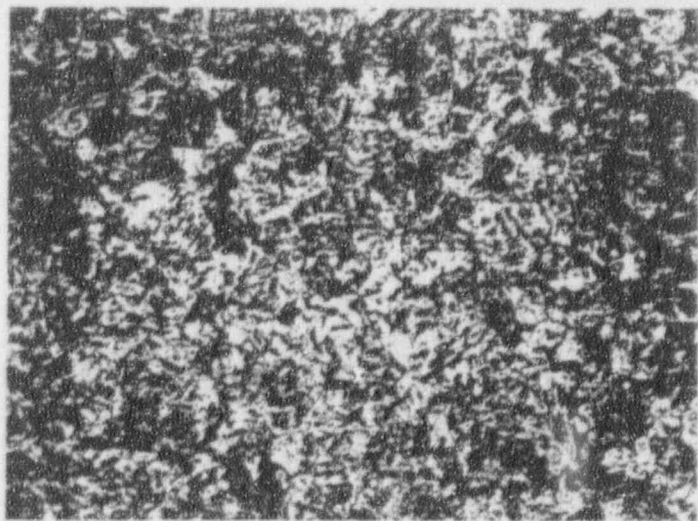
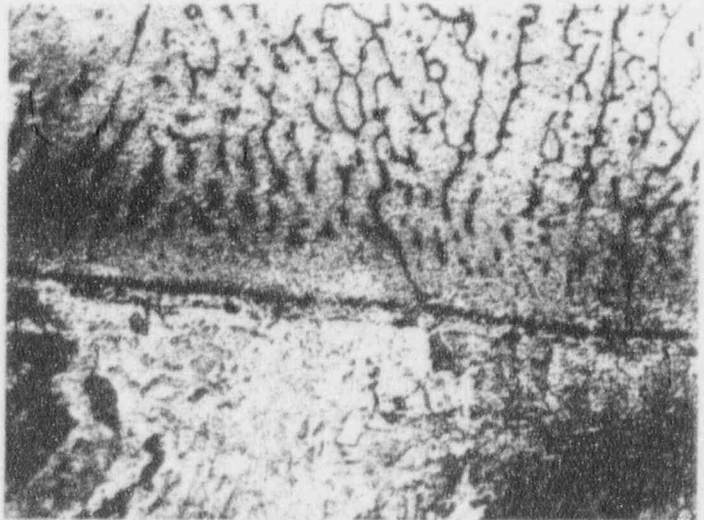
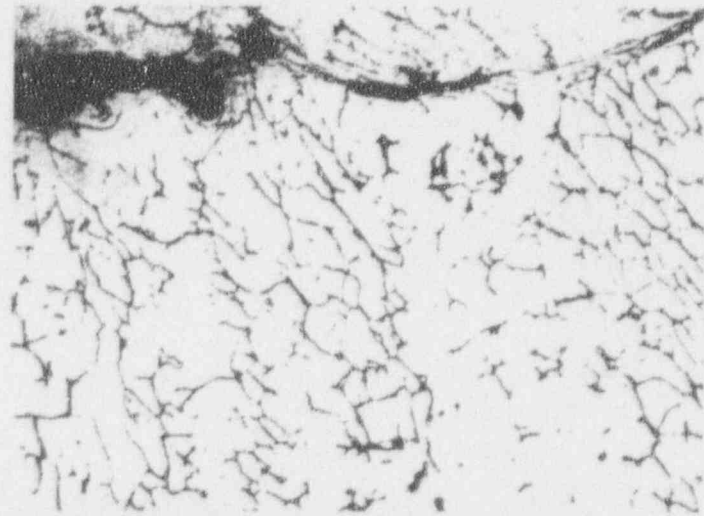




TMIG-19 (800°C/10 min.)

U91 0235

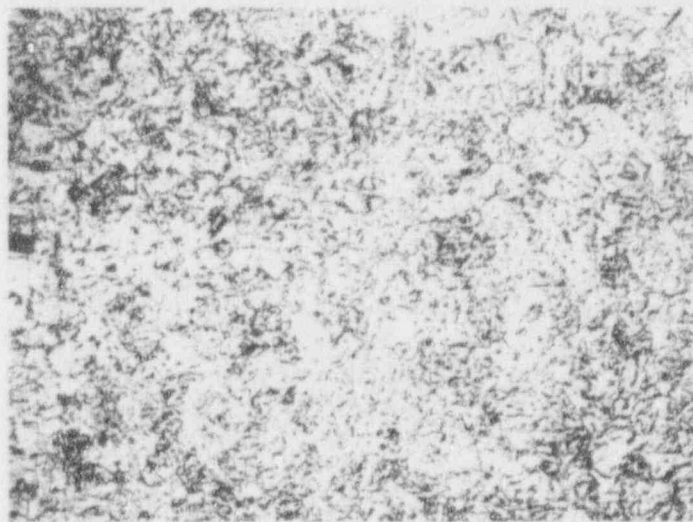
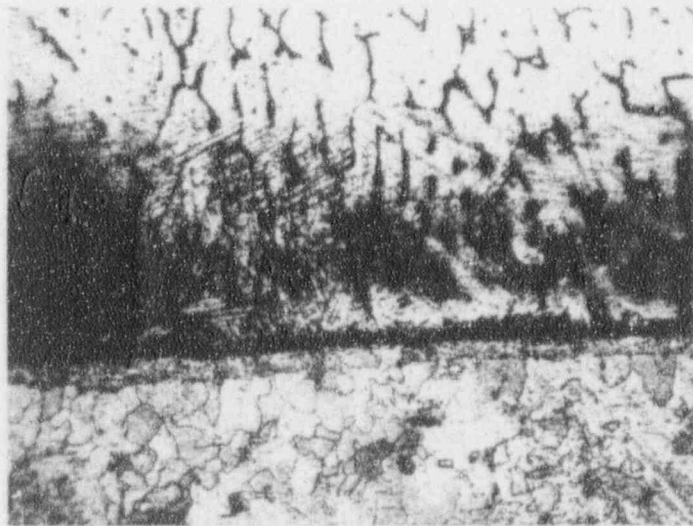
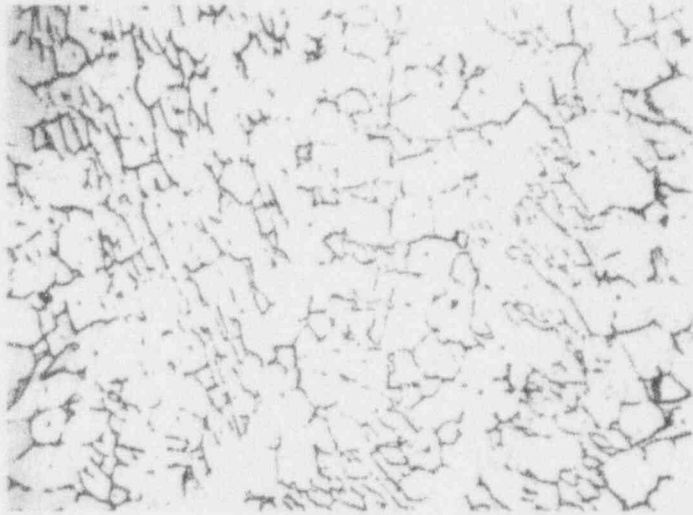
—  
50μm  
B-6



TMIG-23 (800°C/100 min.)

U91 0239

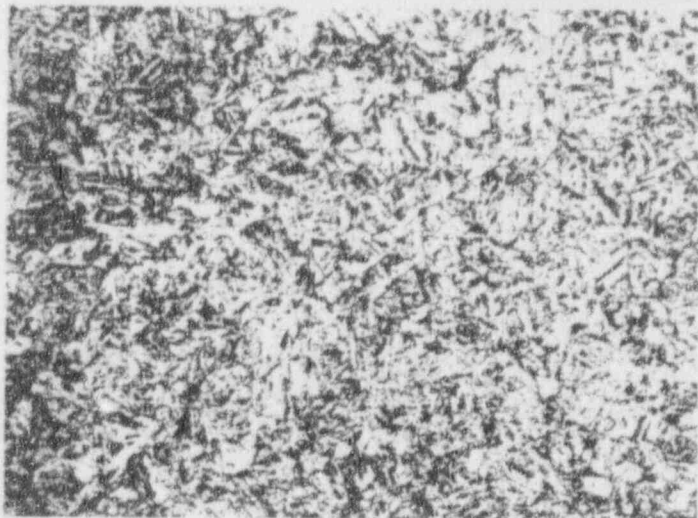
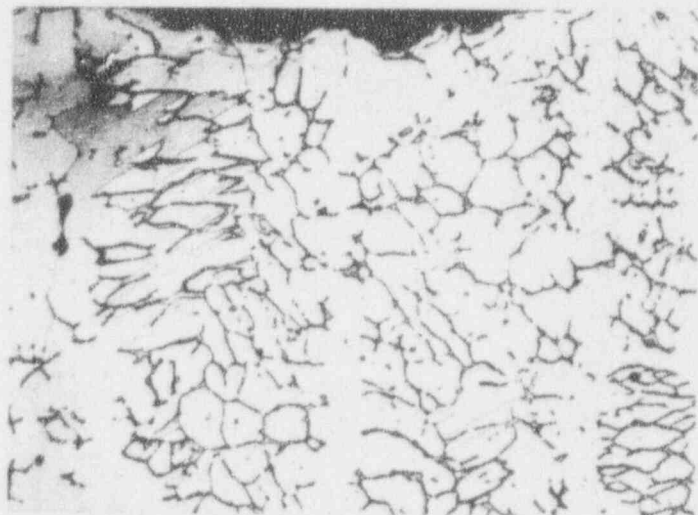
—  
50µm  
B-7



TMIG-16 (900°C/1 min.)

U9: 0232

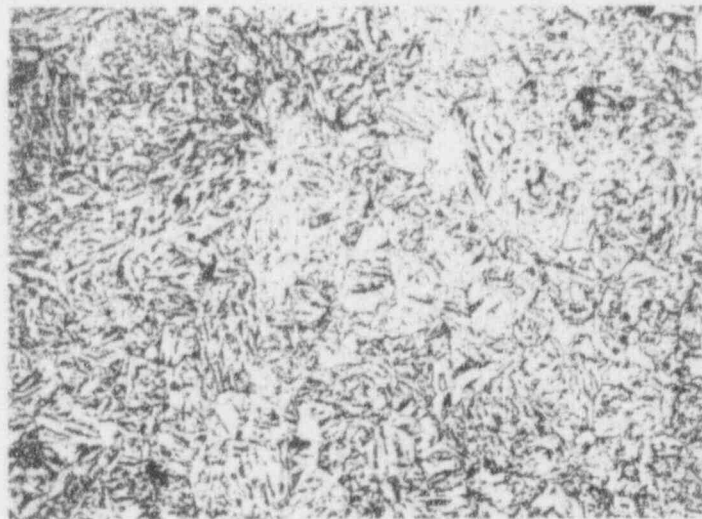
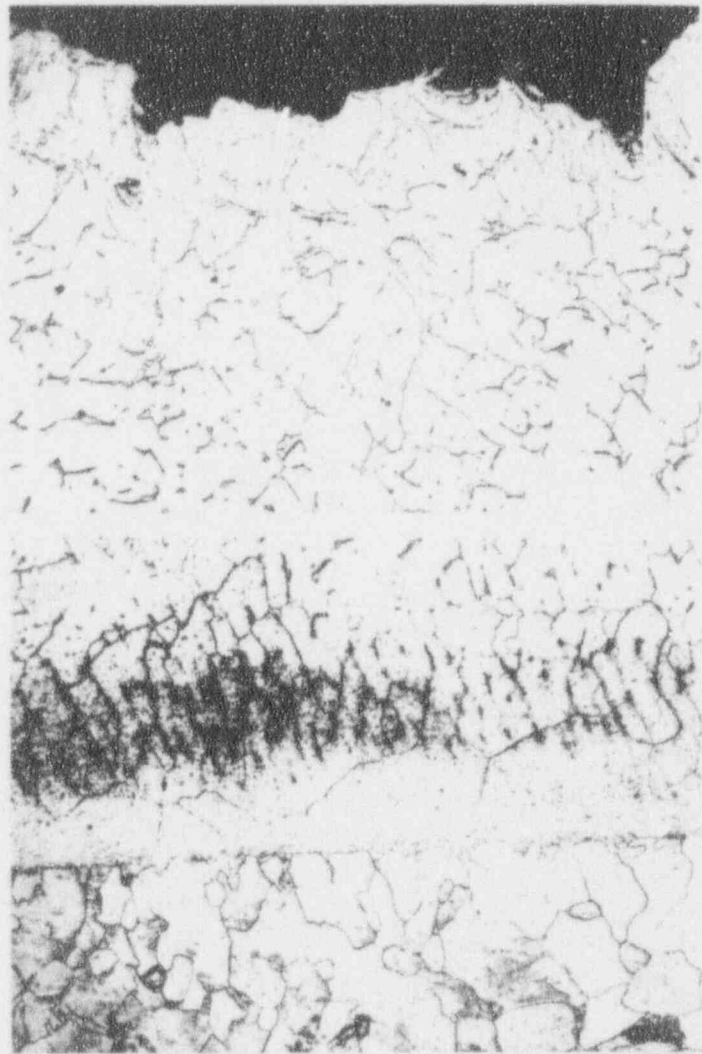
—  
50µm  
B-8



TMIG-20 (900°C/10 min.)

U91 0236

50µm  
B-9

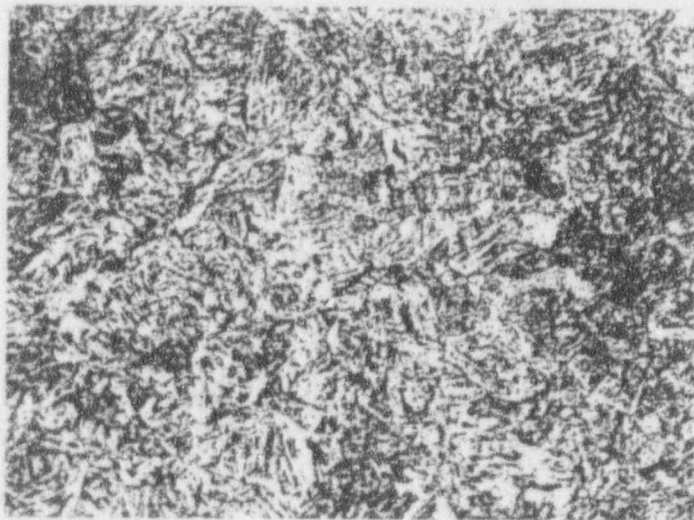
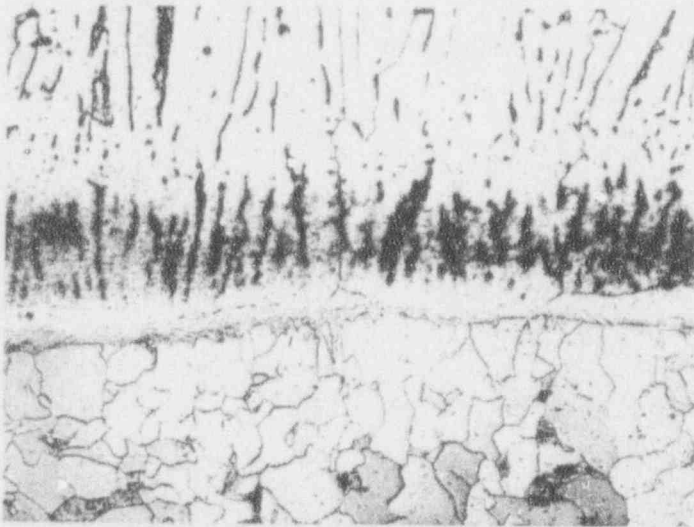
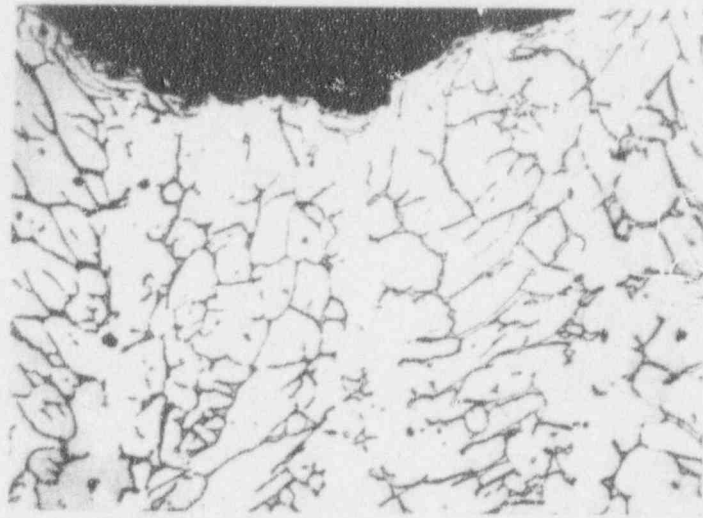


TMIG-24 (900°C/100 min.)

U91 0240

50µm

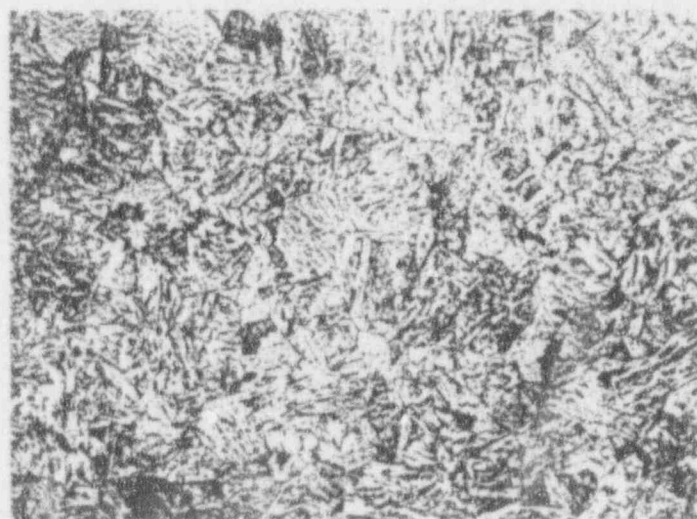
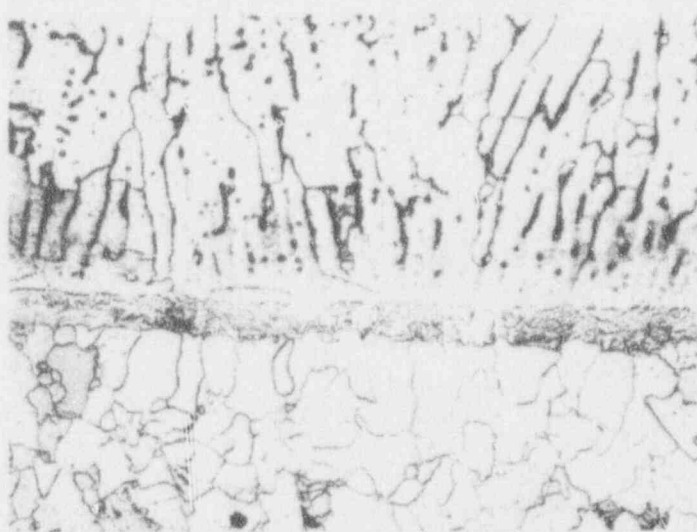
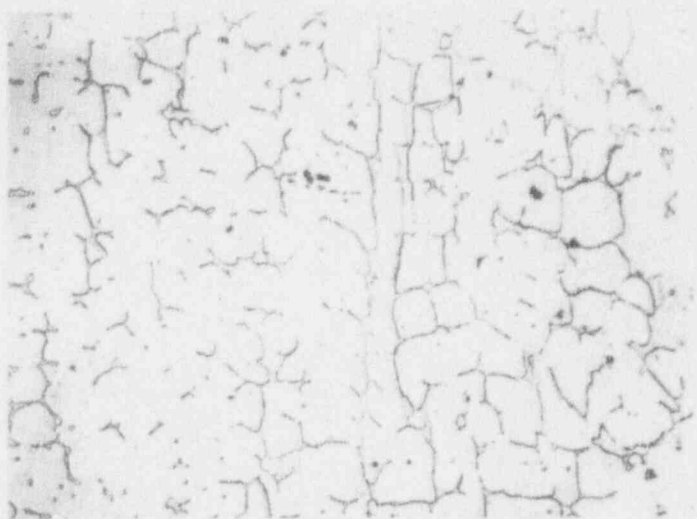
B-10



TMIG-17 (1000°C/1 min.)

U91 0233

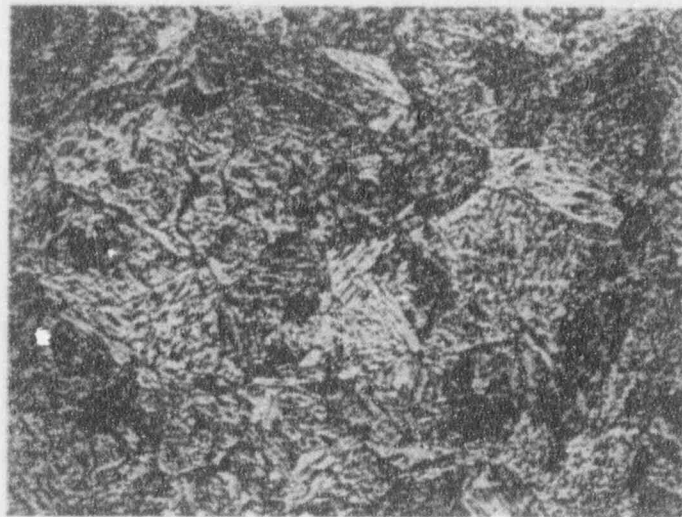
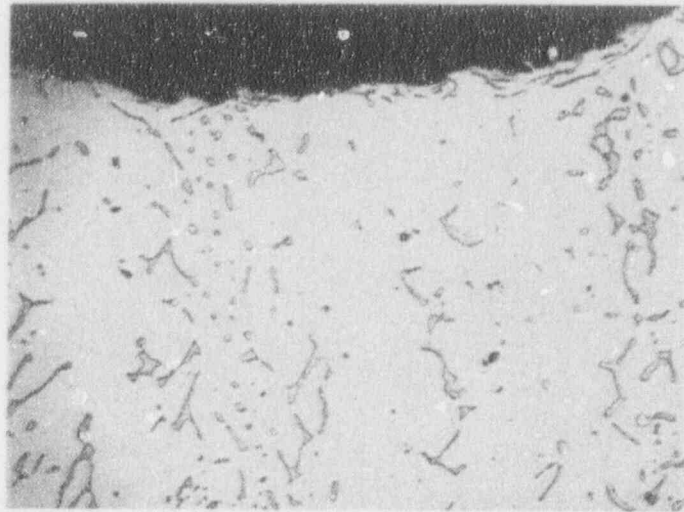
50µm  
B-11



TMIG-21 (1000°C/10 min.)

U91 0237

50µm  
B-12

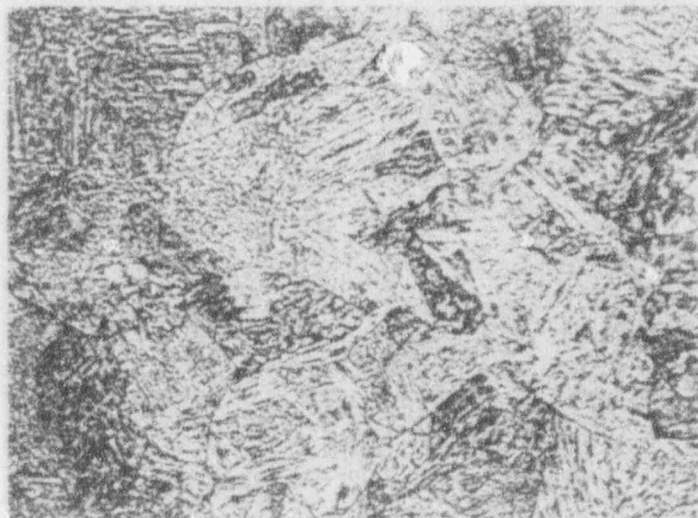
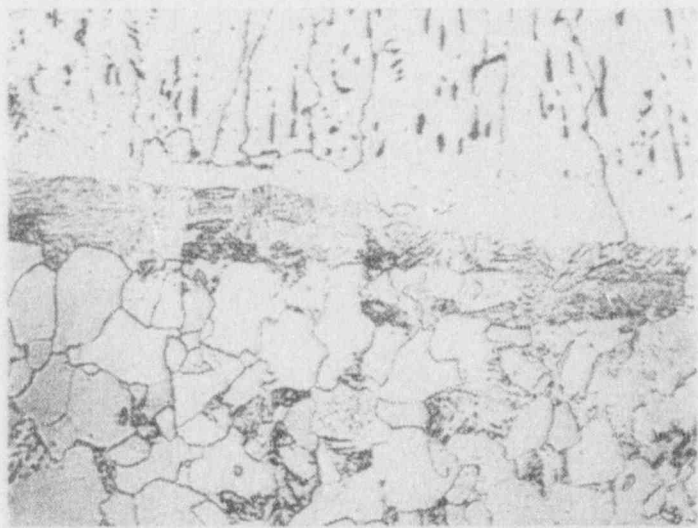
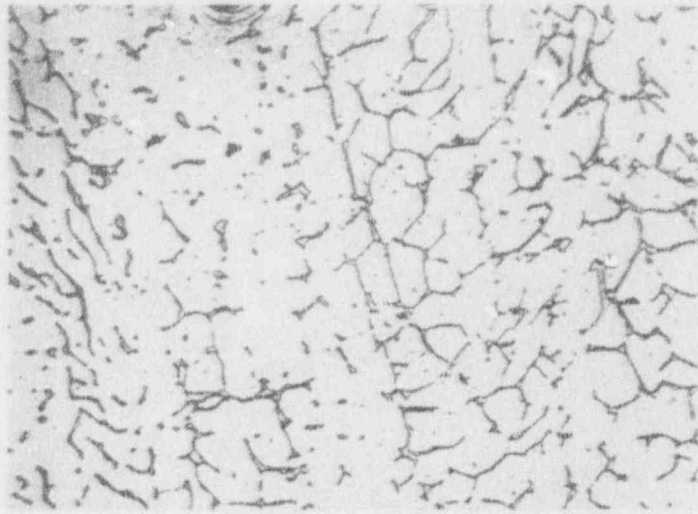


TMIG-25 (1000°C/100 min.)

U91 0241

50µm

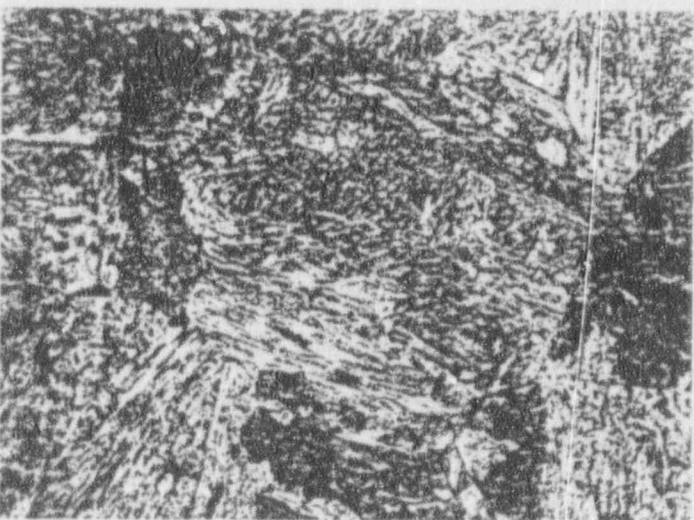
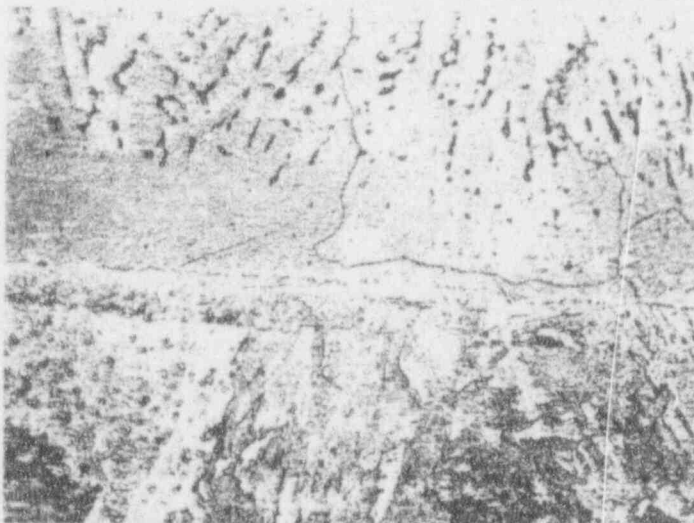
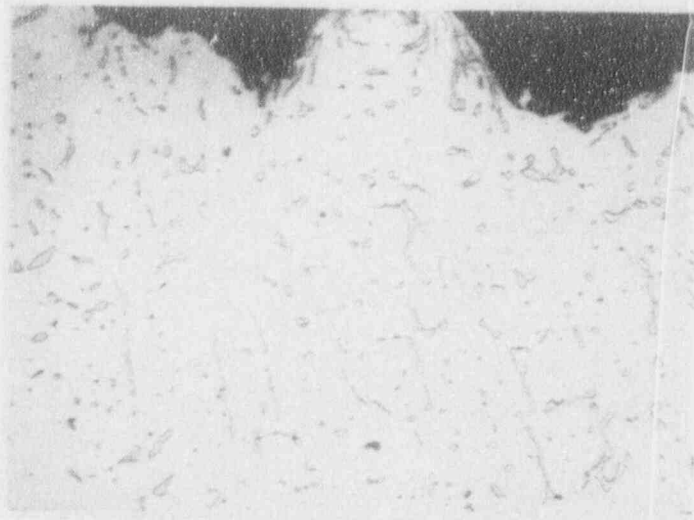




TMIG-18 (1100°C/1 min.)

U91 0234

50µm

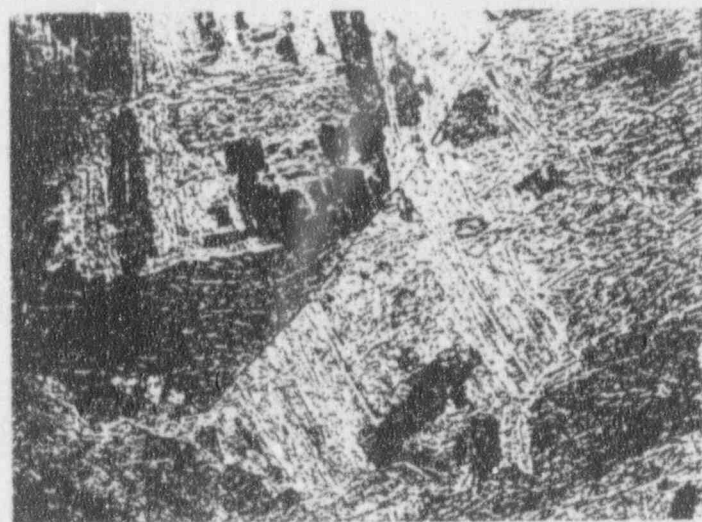
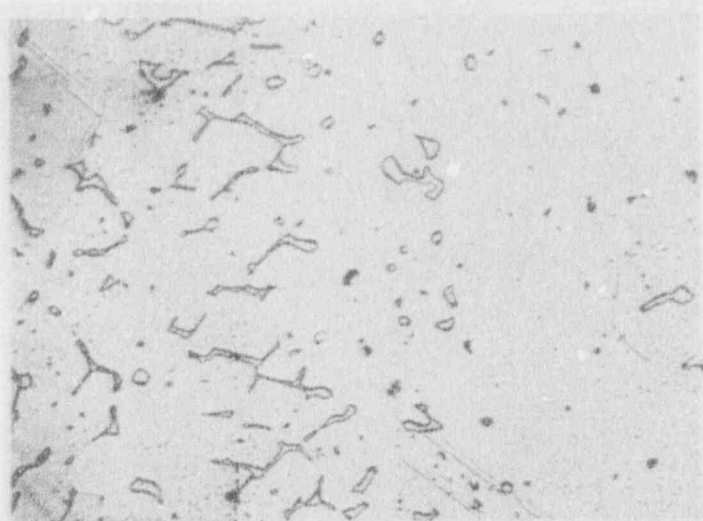


TMIG-22 (1100°C/10 min.)

U91 0238

50µm

B-15



TMIG-26 (1100°C/100 min.)

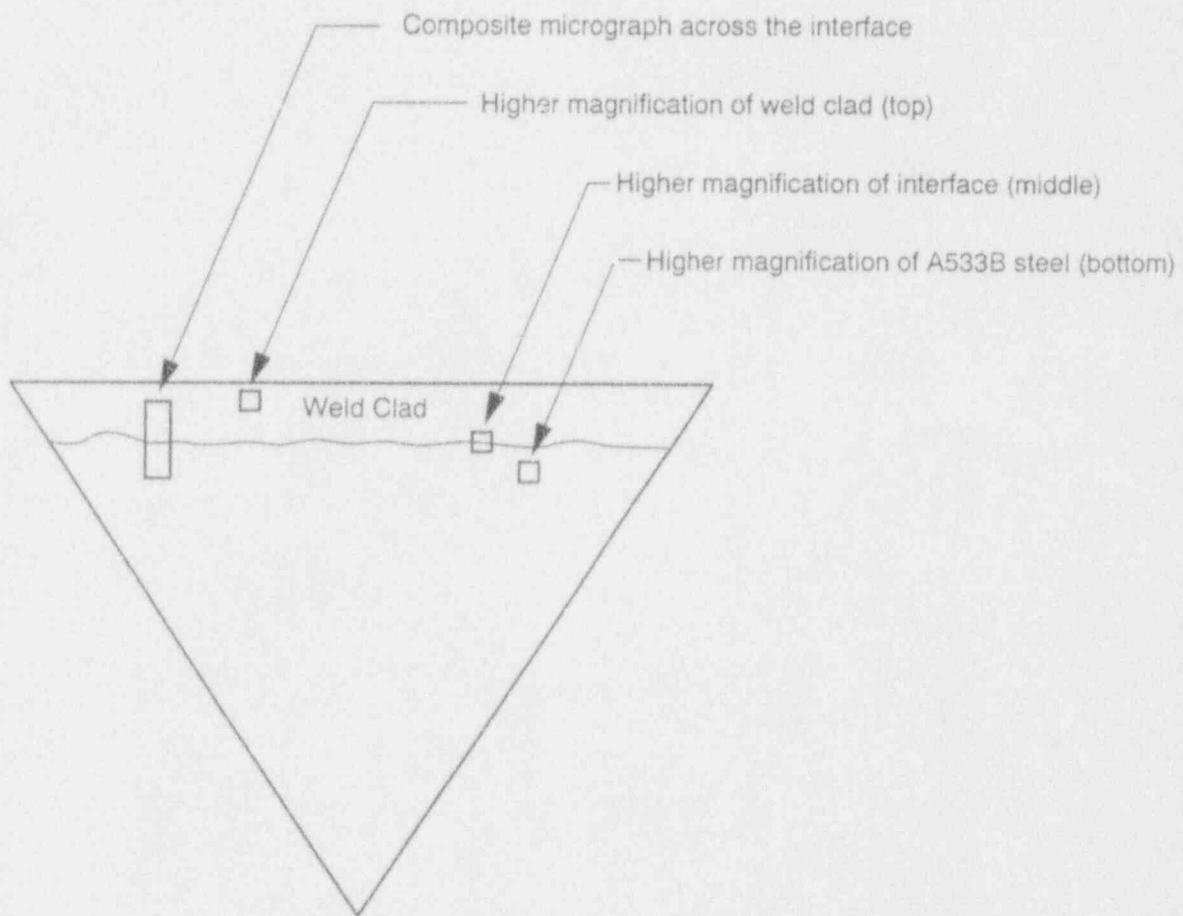
U91 0242

50µm

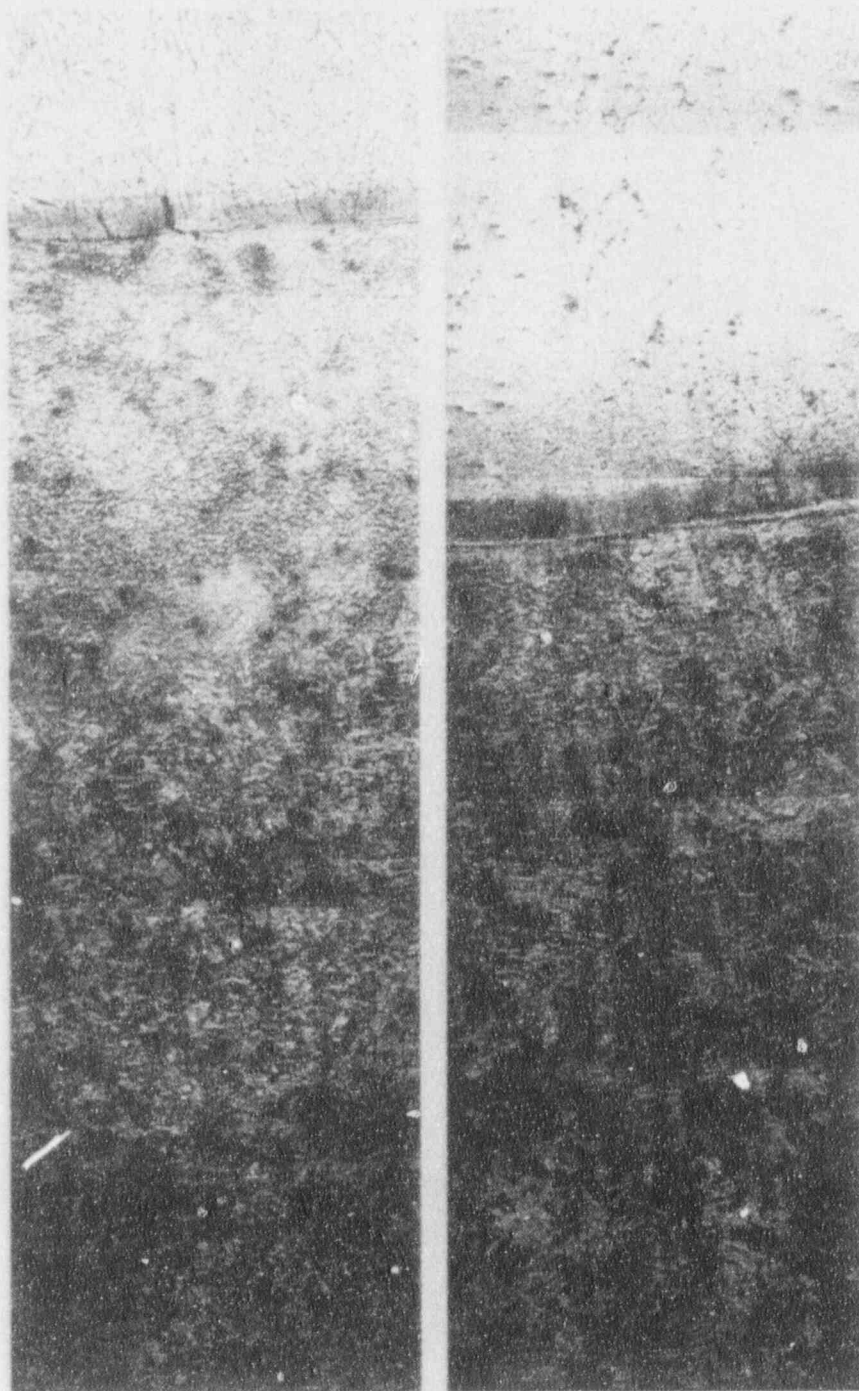
B-16

## Appendix C

**Microstructure of TMI-2 Samples F-10(M-3),  
E-8(M-3), G-8 (408P-3), E-6 (402A-1),  
and Accident-Simulated Heat Treated Slices  
of Samples H-4(M-3) and M-11(M-3)**



U91 0266



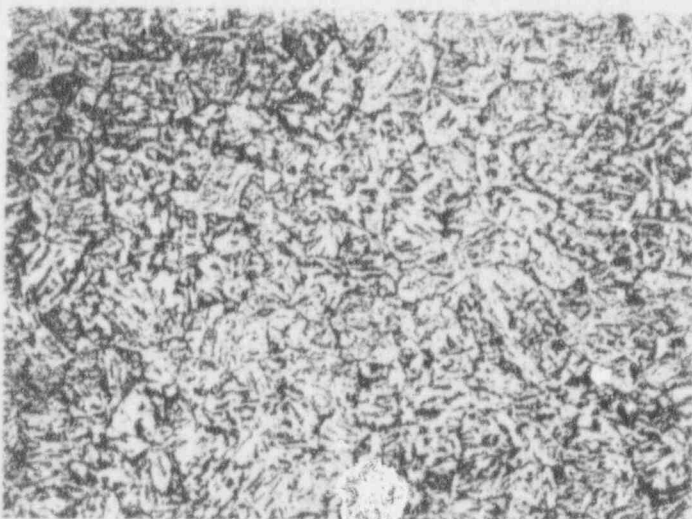
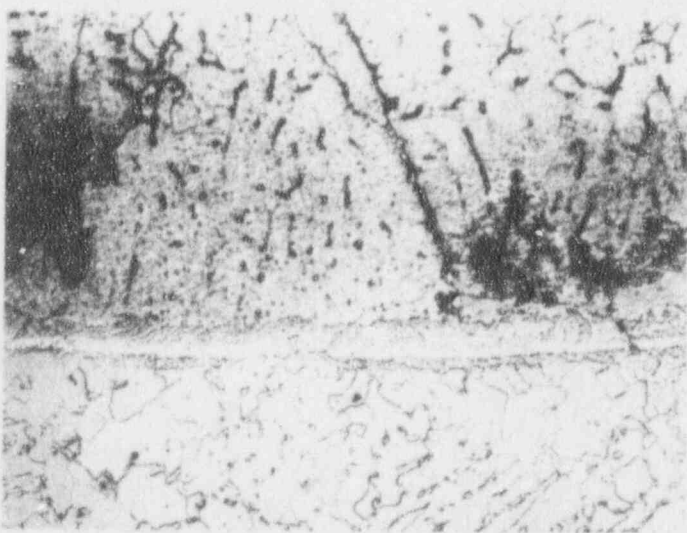
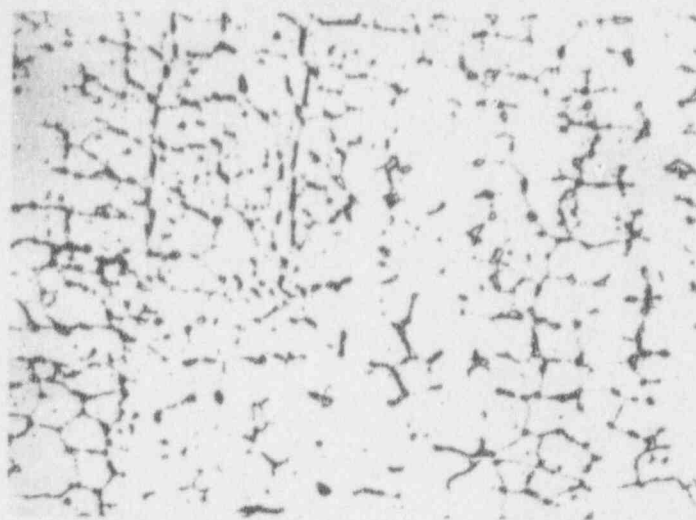
F-10

E-8

U91 0243

200µm

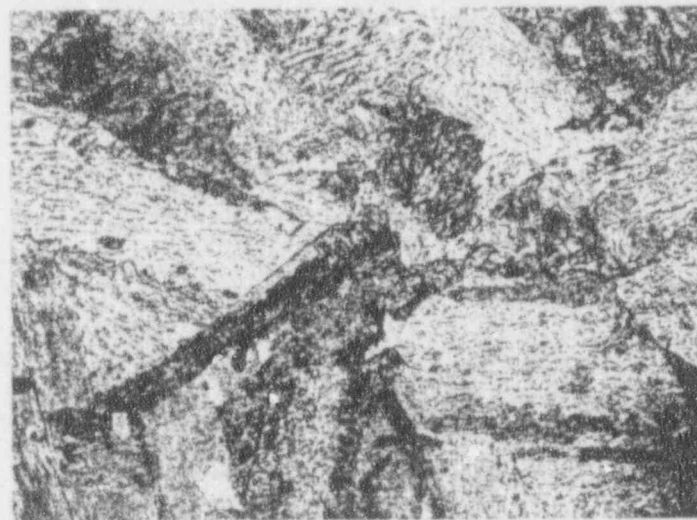
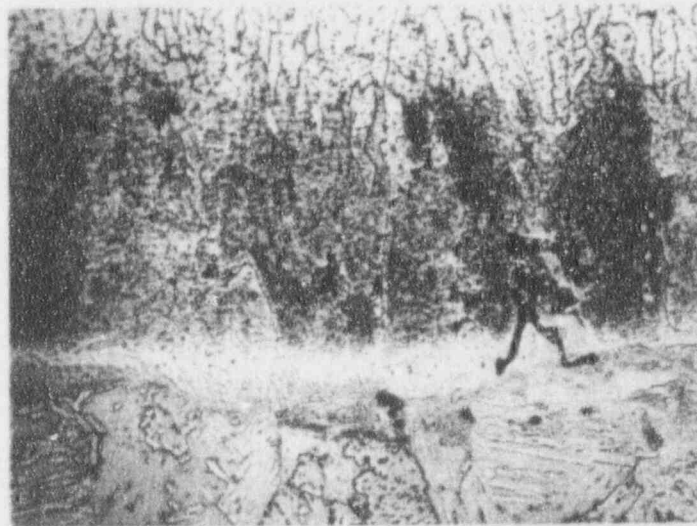
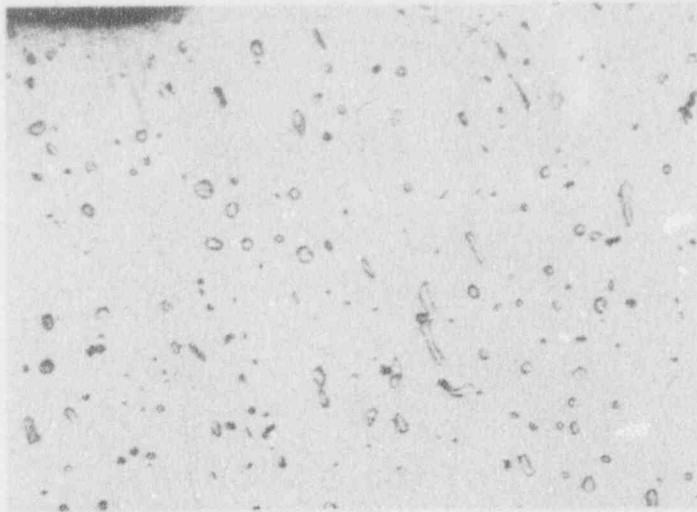
C-4



F-10

U91 0244

50 $\mu$ m

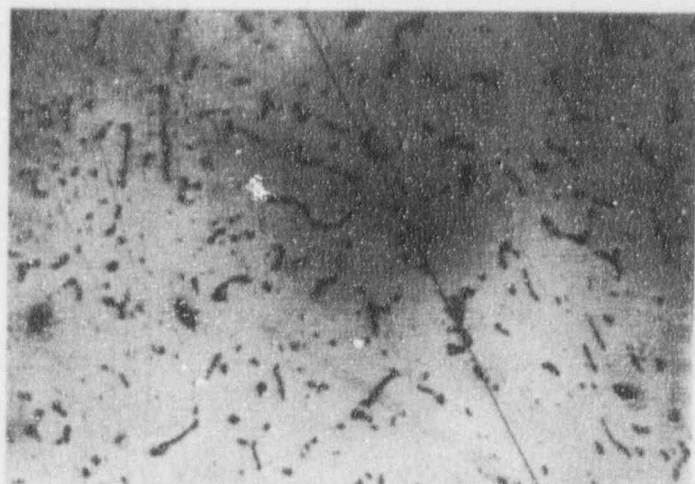


E-8

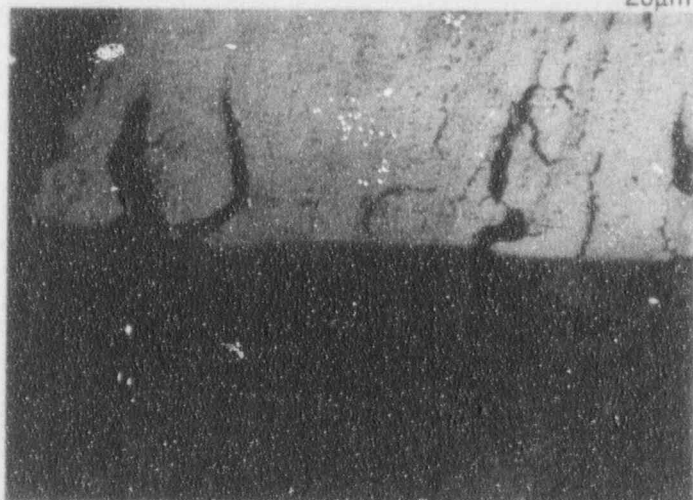
U91 0245

50μm





20 $\mu$ m



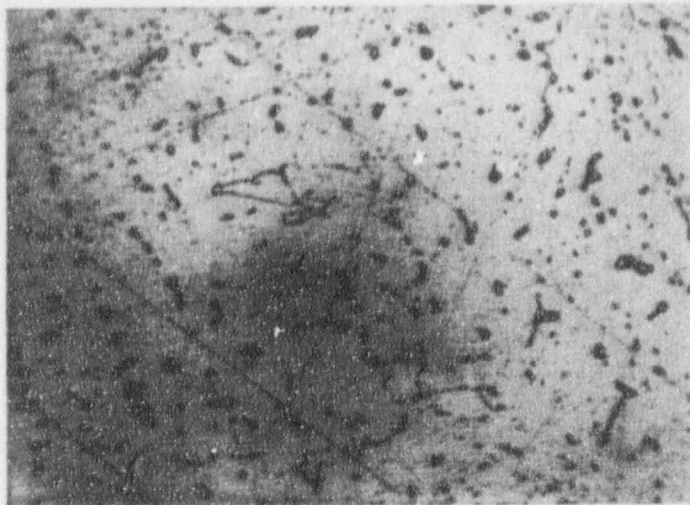
100 $\mu$ m



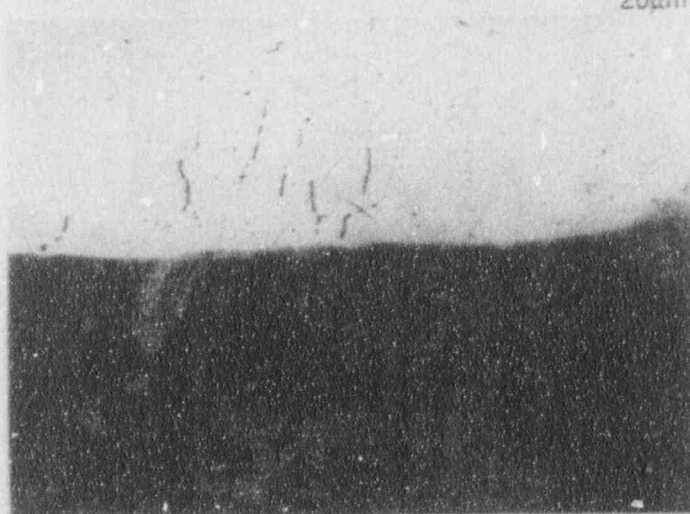
100 $\mu$ m

G-8 (408P-3) (Micrographs courtesy of D.R. Diercks, Argonne National Laboratory)

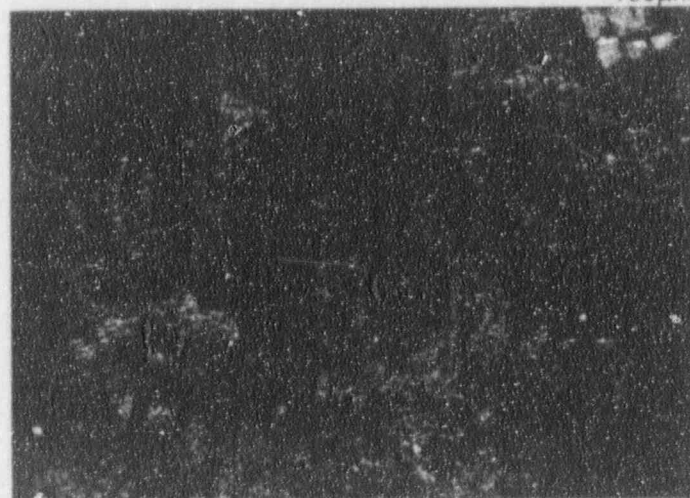
U91 0268



20µm



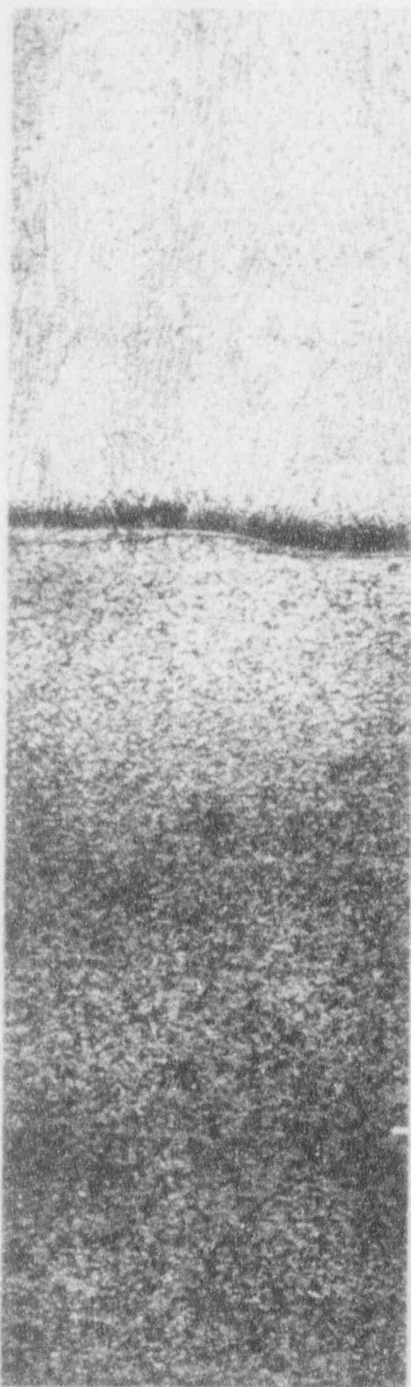
100µm



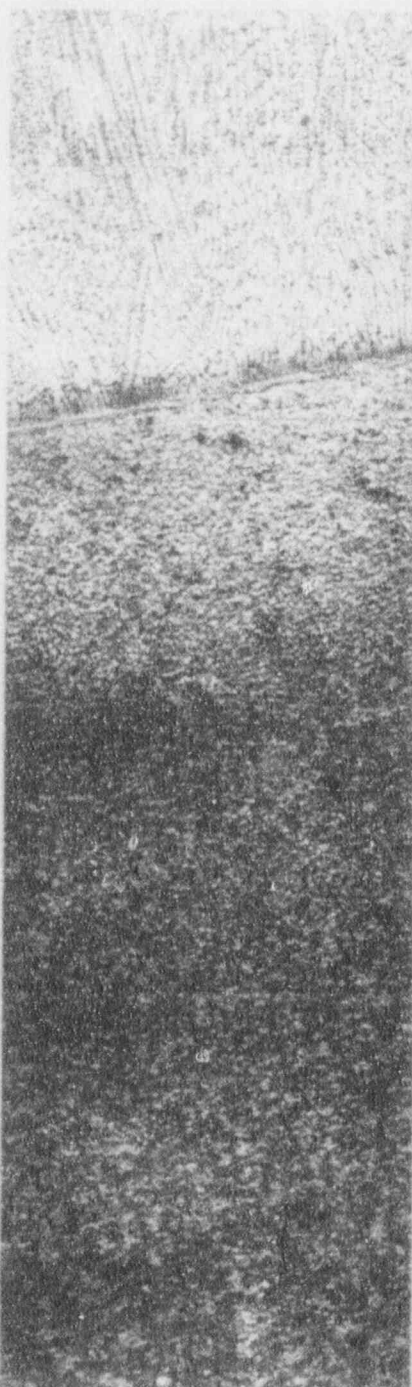
100µm

E-6 (402A-1) (Micrographs courtesy of  
D.R. Diercks, Argonne National Laboratory)

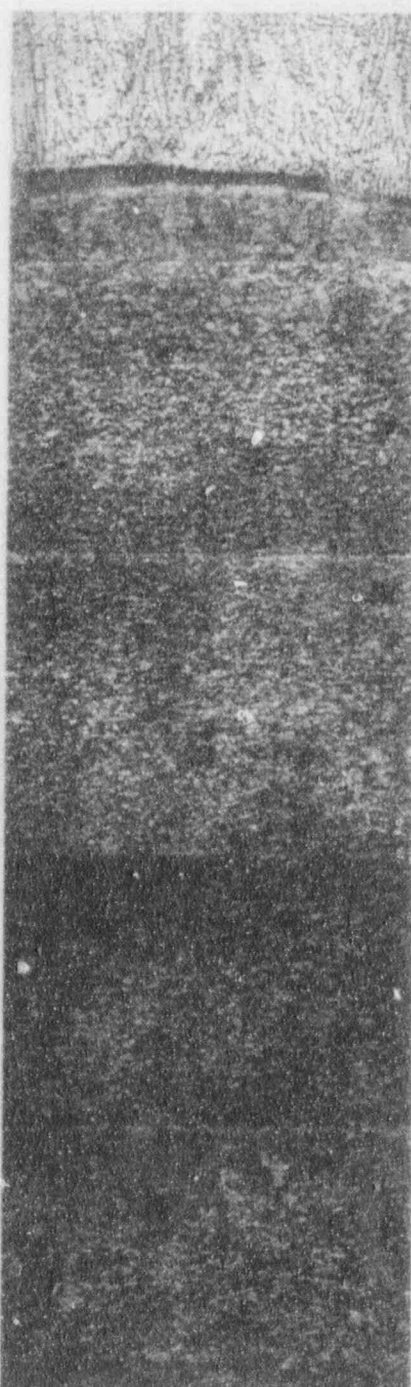
U91 0267



H4-1  
(950°C/10 min)



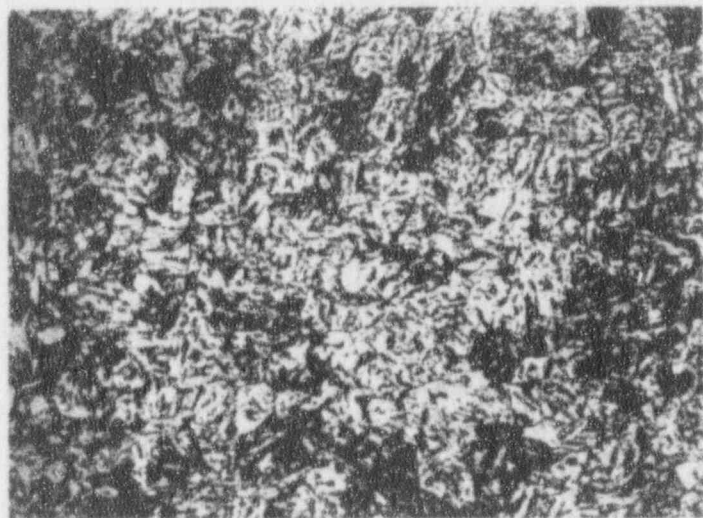
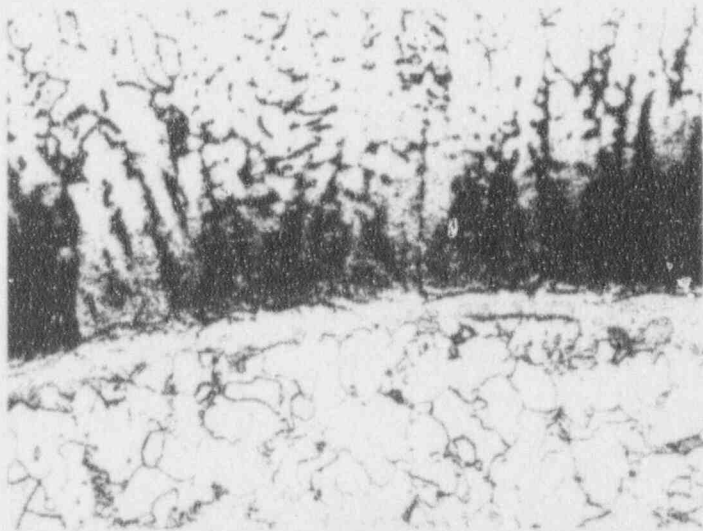
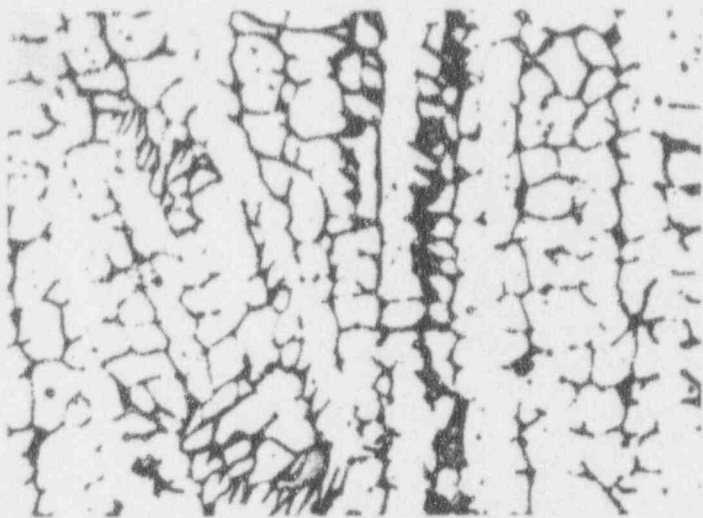
H4-5  
(950°C/30 min)



M11-3  
(950°C/100 min)

U91 0246

200μm

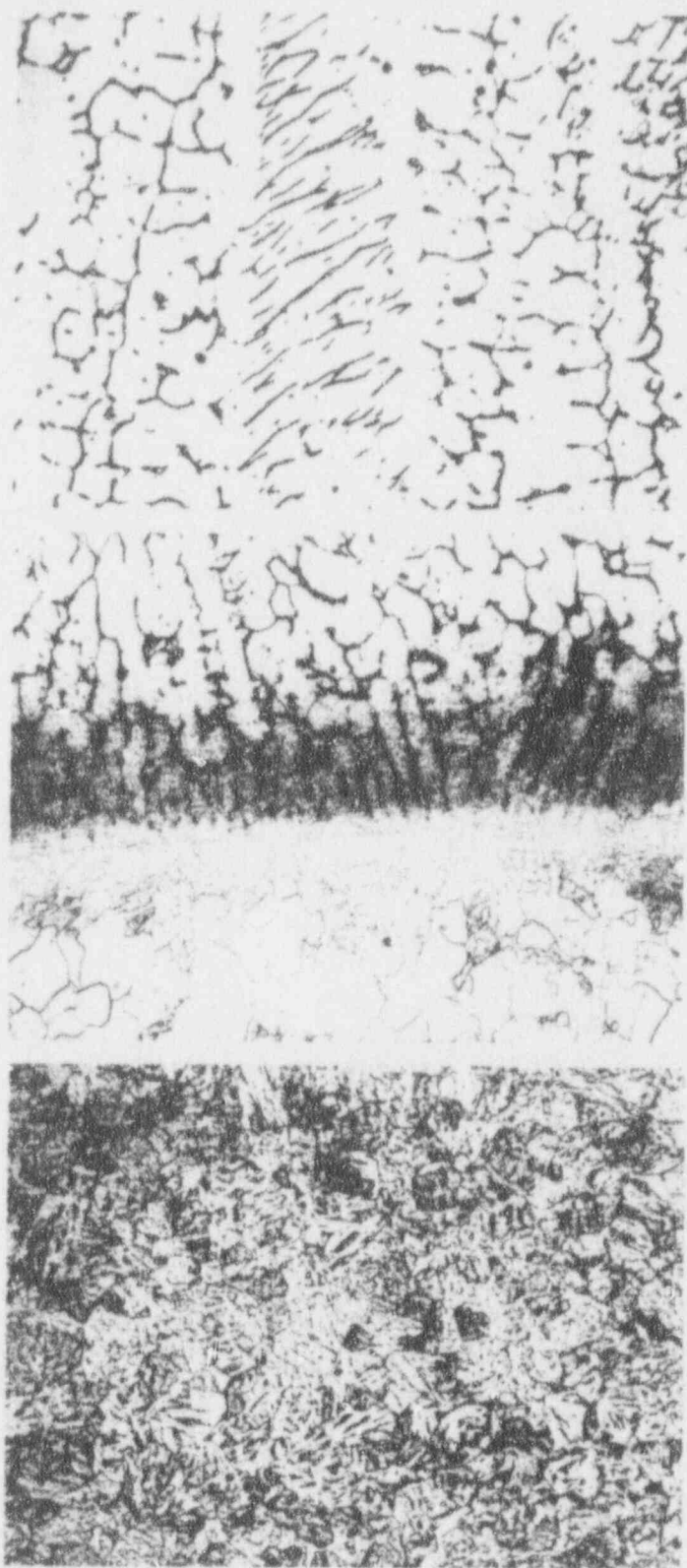


H4-1 (950°C/10 min)

U91 0247

50µm

C-10

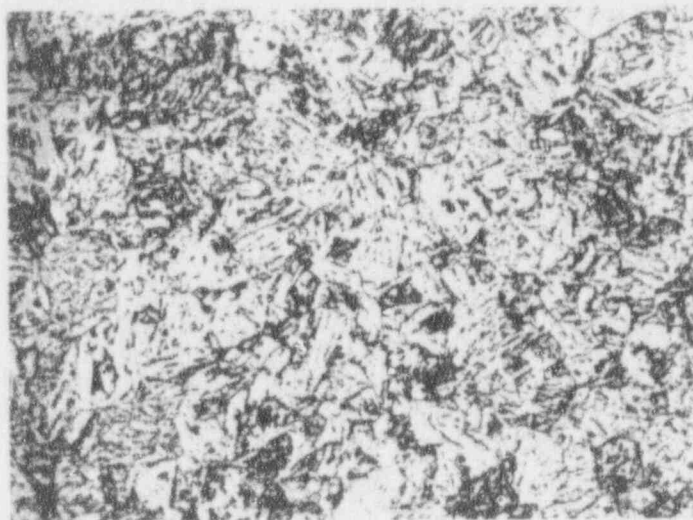
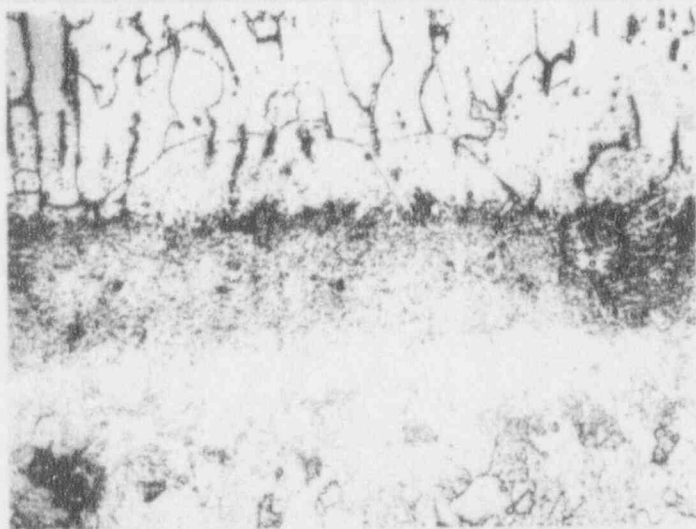
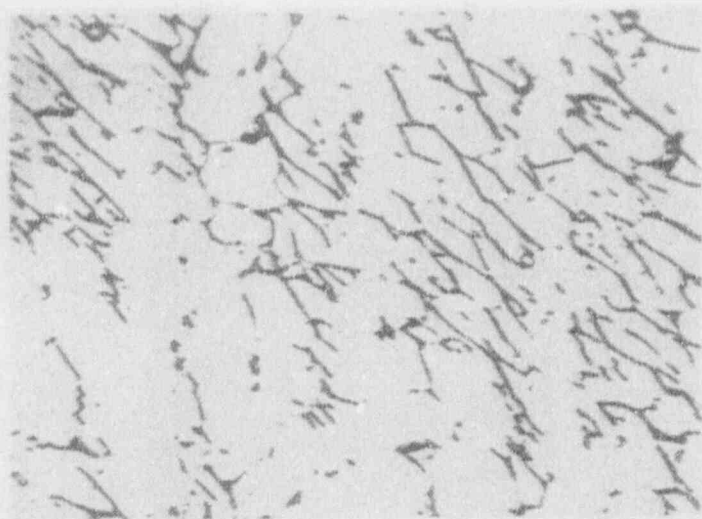


H4-5 (950°C/30 min)

U91 0248

50µm

C-11

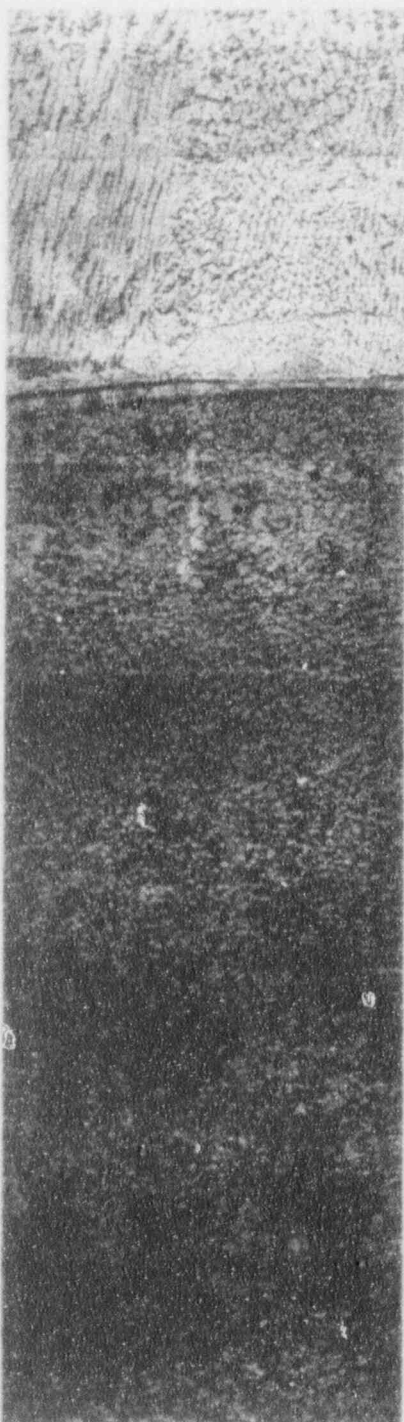


M11-3 (950°C/100 min)

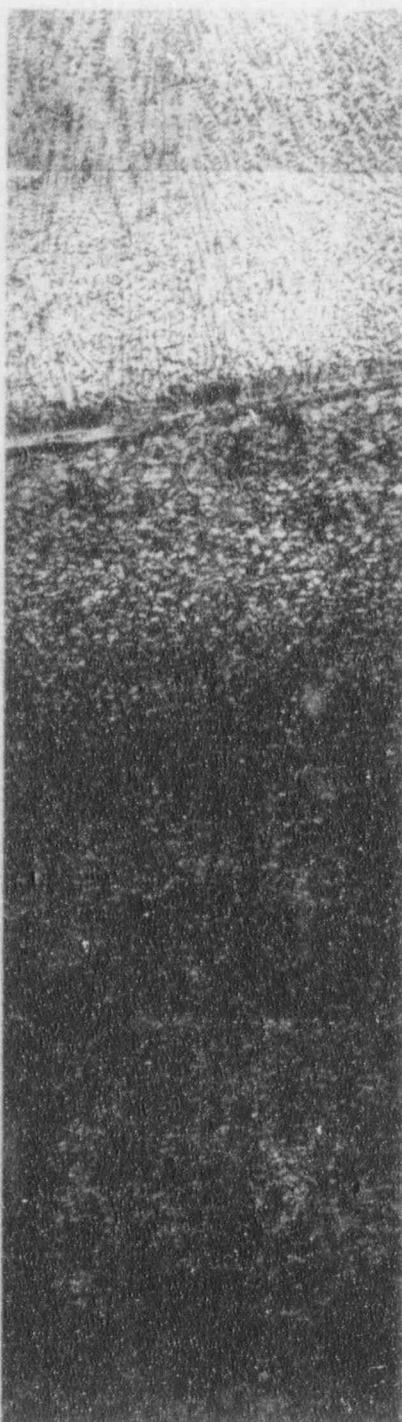
U91 0249

50µm

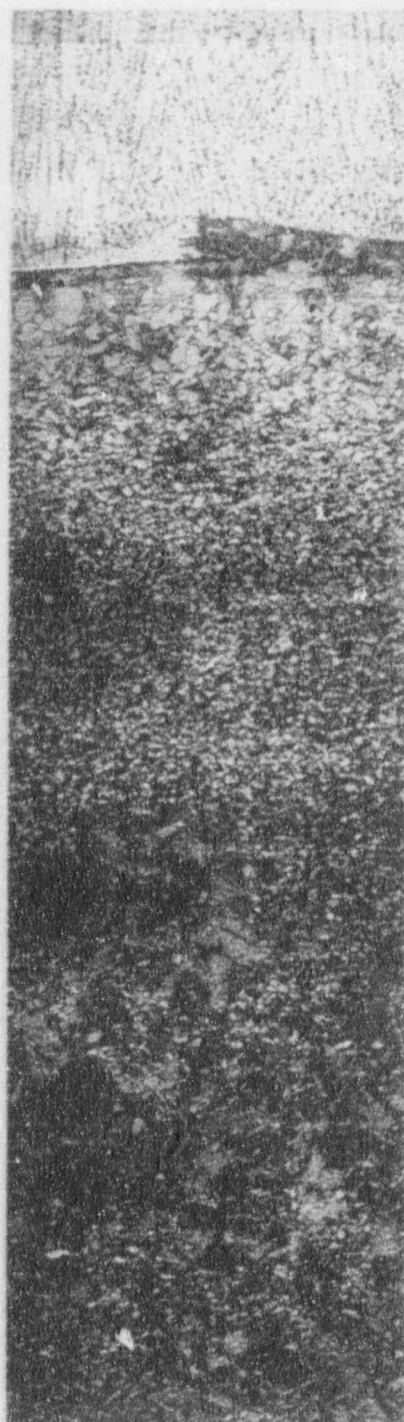
C-12



H4-2  
(1000°C/10 min)



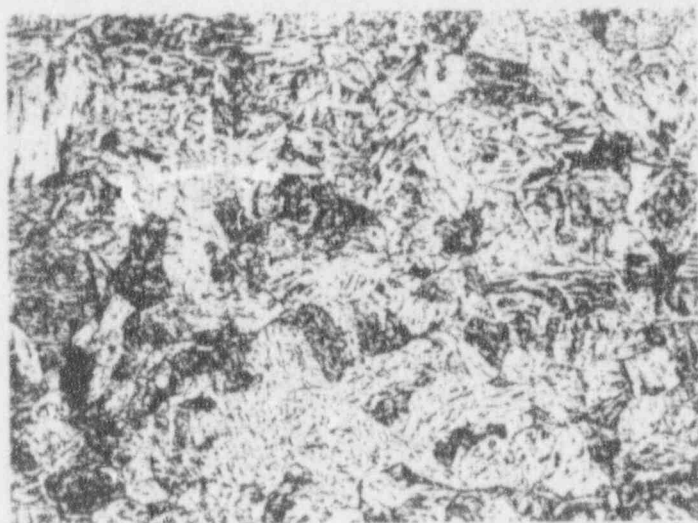
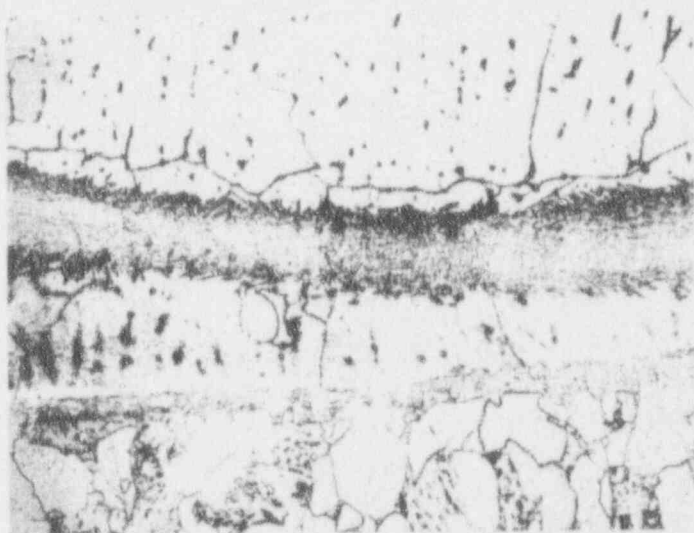
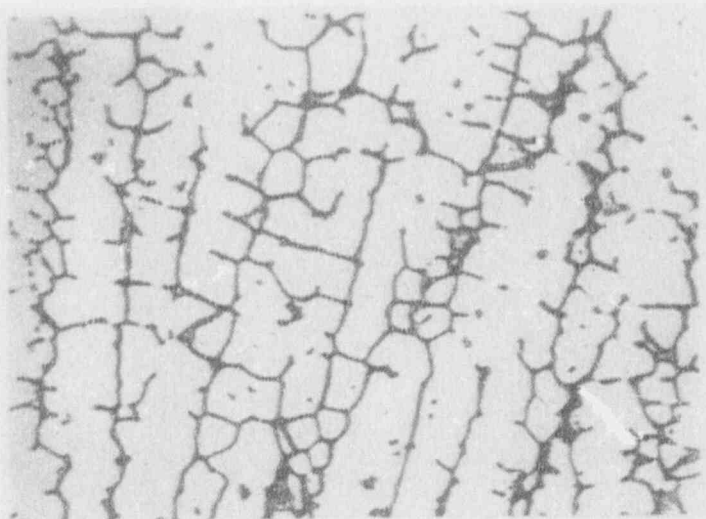
H4-6  
(1000°C/30 min)



M11-4  
(1000°C/100 min)

U91 0250

200µm

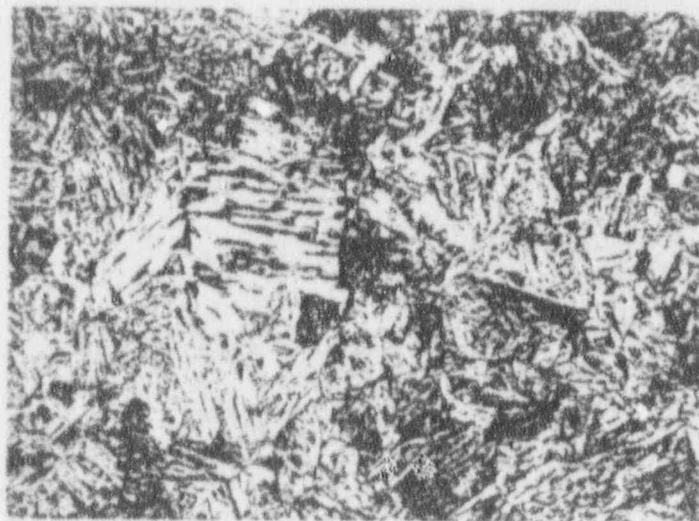
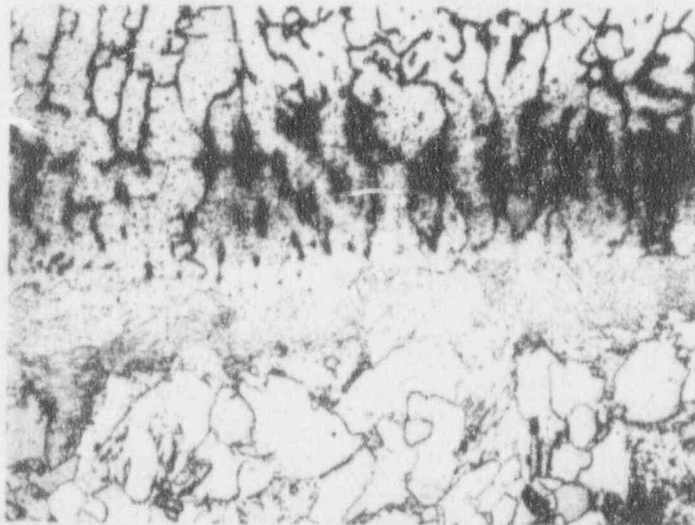
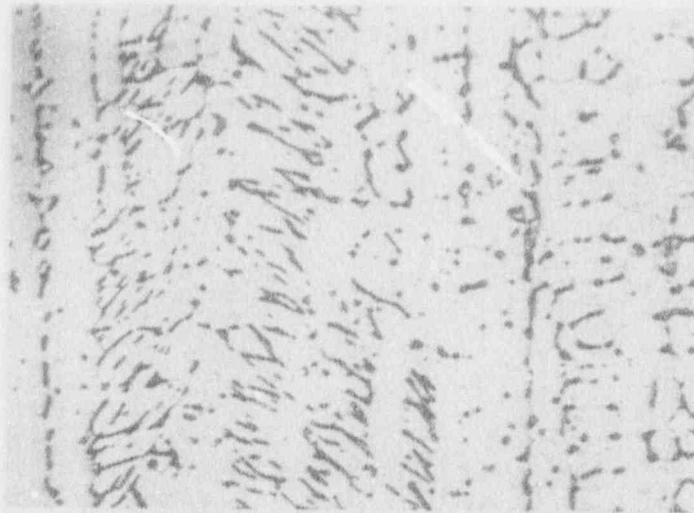


H4-2 (1000°C/10 min)

U91 0269

50μm





H4-6 (1000°C/30 min)

U91 0270

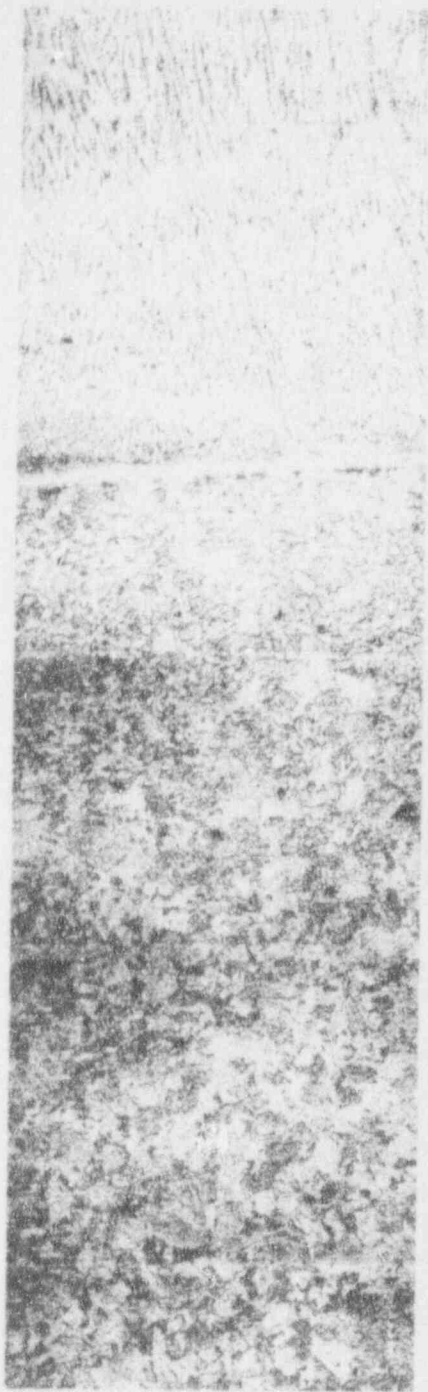
50µm



M11-4 (1000°C/100 min)

U91 0253

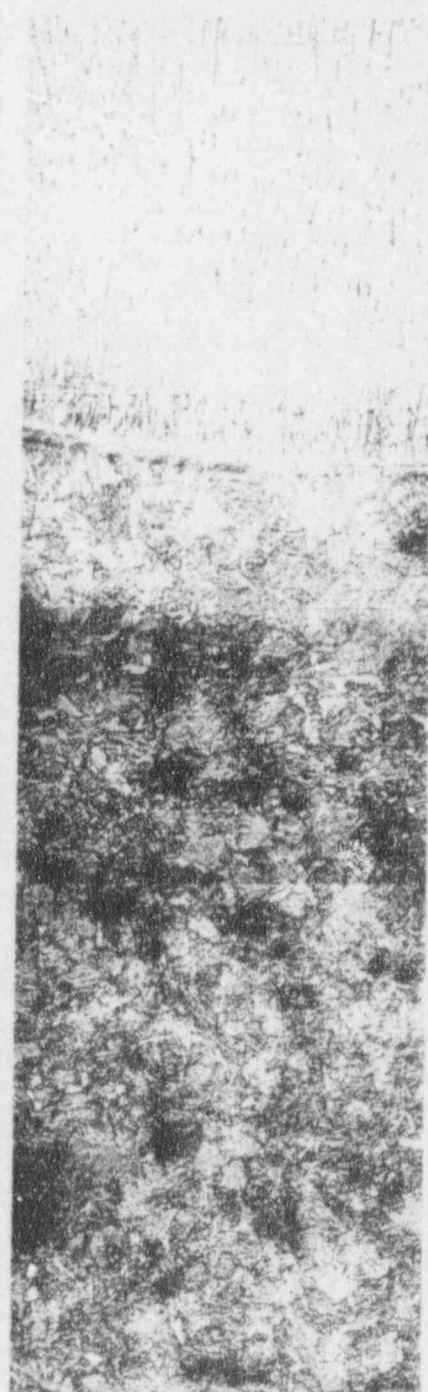
50µm



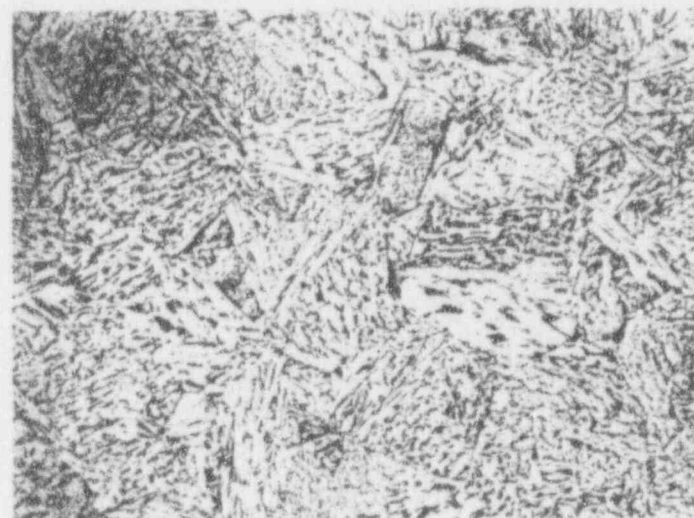
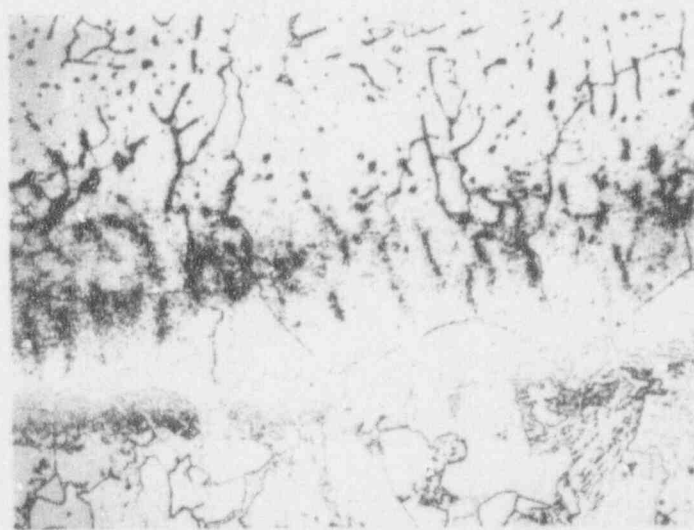
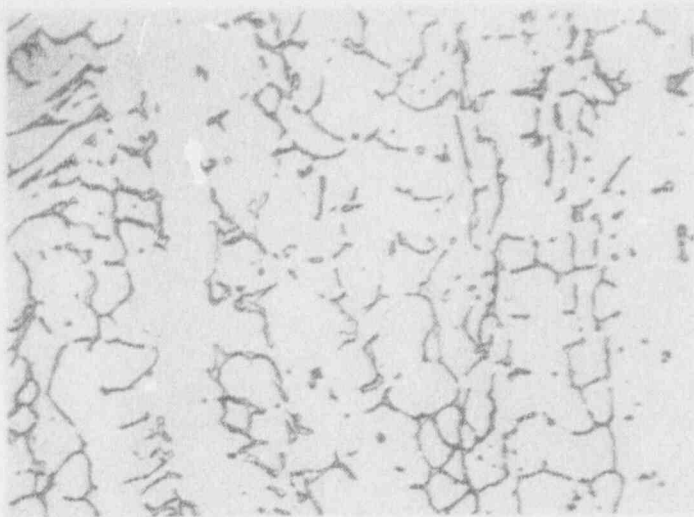
H4-3  
(1050°C/10 min)



M11-1  
(1050°C/30 min)  
200μm



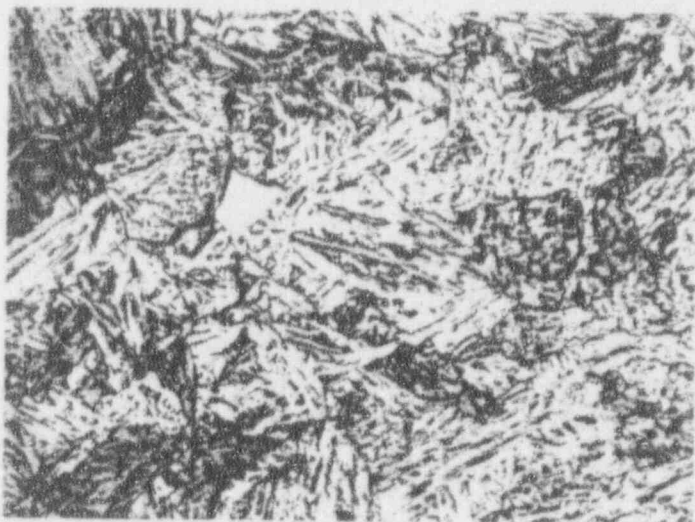
M11-5 U91 0254  
(1050°C/100 min)



H4-3 (1050°C/10 min)

U91 0255

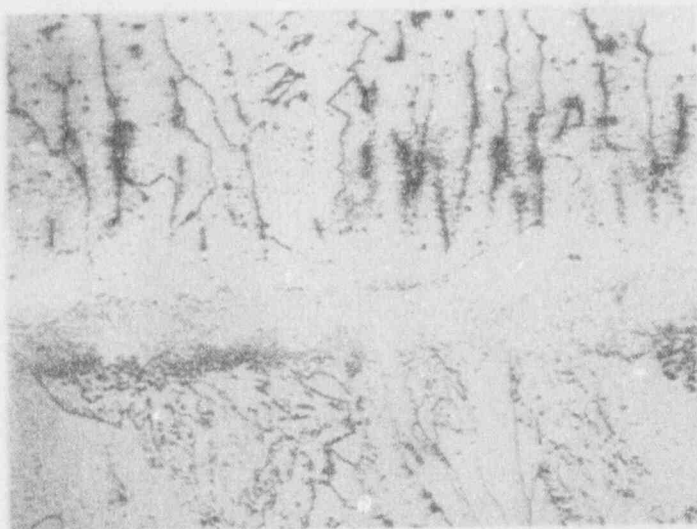
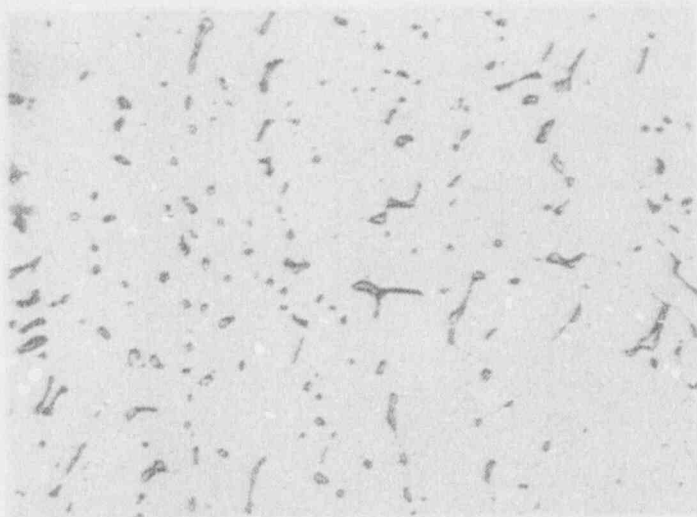
50μm



M11-1 (1050°C/30 min)

U91 0256

50µm

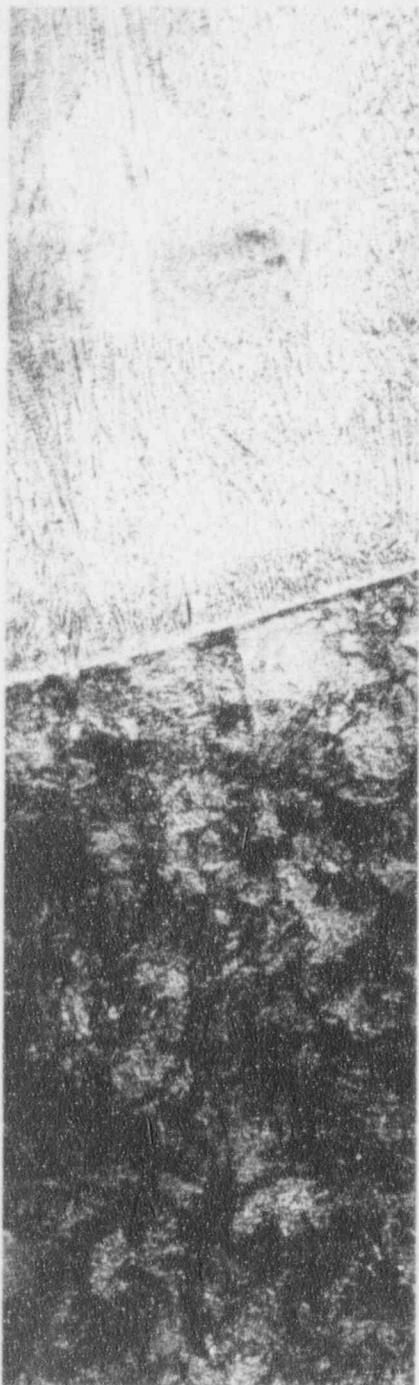


M11-5 (1050°C/100 min)

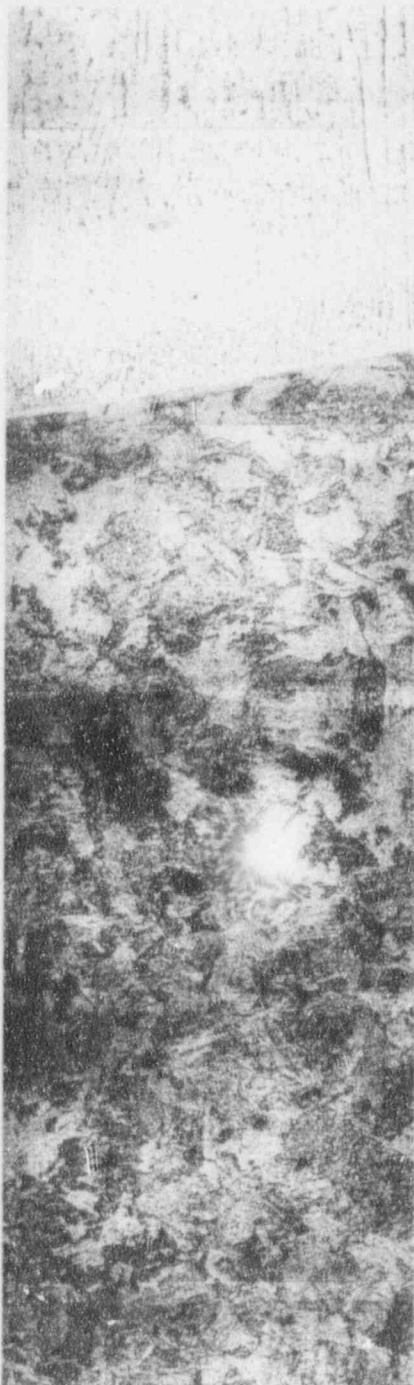
U91 0257

50μm

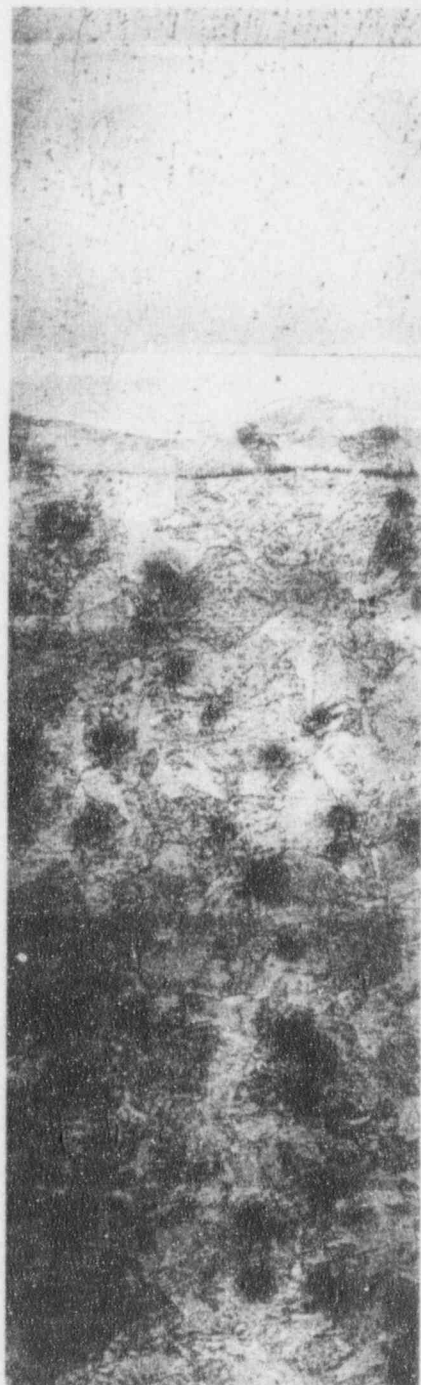
C-20



H4-4  
(1100°C/10 min)



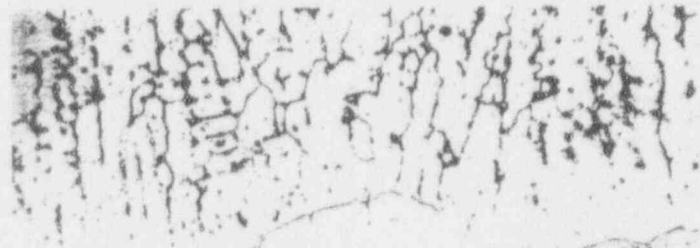
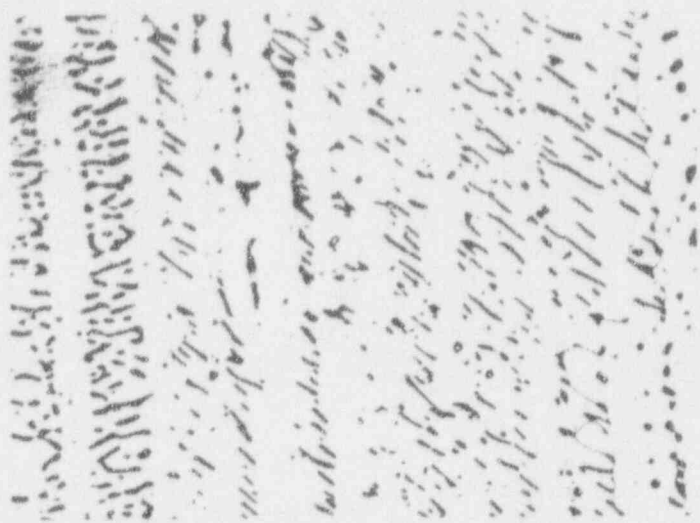
M11-2  
(1100°C/30 min)



M11-6  
(1100°C/100 min)

U91 0259

200μm

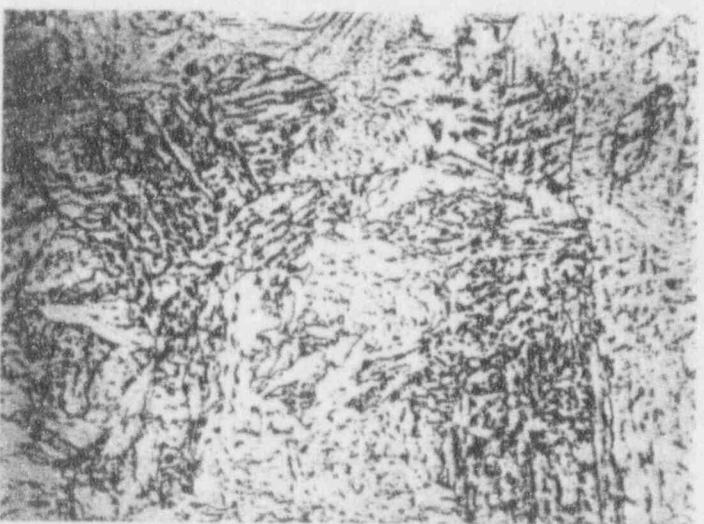
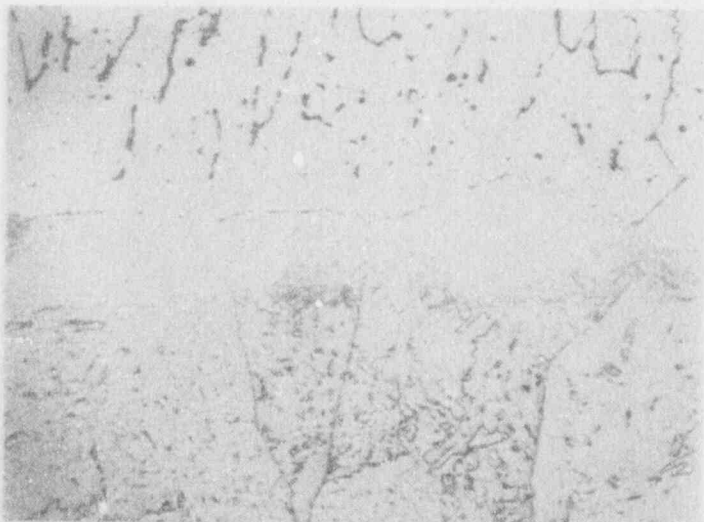
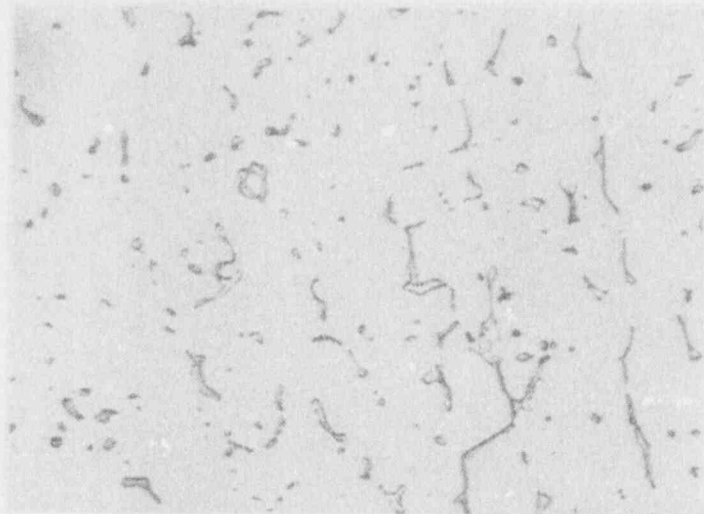


H4-4 (1100°C/10 min)

U91 0260

50µm



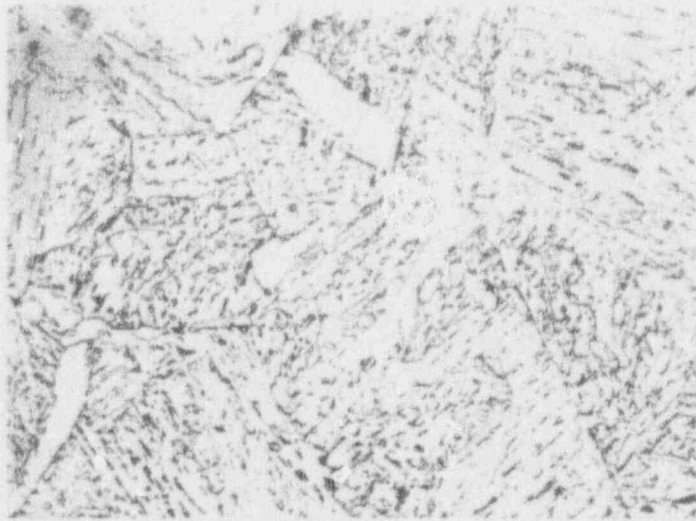
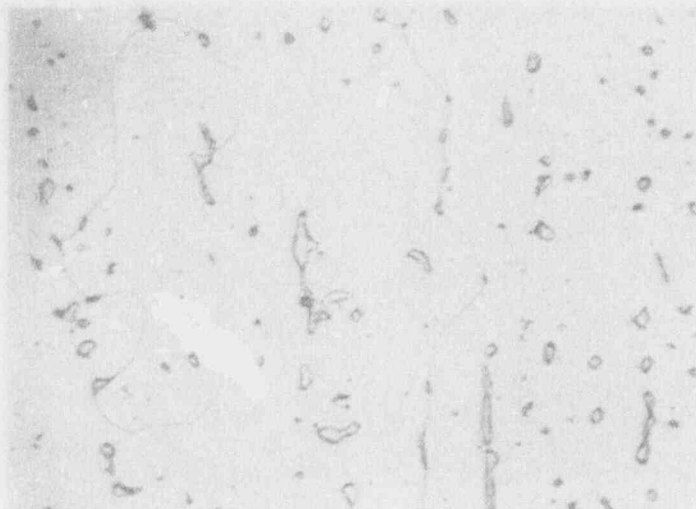


M11-2 (1100°C/30 min)

U91 0262

50μm

C-23



M11-6 (1100°C/100 min)

U91 0263

50µm

**BIBLIOGRAPHIC DATA SHEET**

(See instructions on the reverse.)

1. REPORT NUMBER  
(Assigned by NRC. Add Vol., Supp., Rev.,  
and Addendum Numbers, if any.)

NUREG/CR-6194  
TMI V(92)EG01  
EGG-2731

2. TITLE AND SUBTITLE

Metallographic and Hardness Examinations of TMI-2  
Lower Pressure Vessel Head Samples

3. DATE REPORT PUBLISHED

MONTH	YEAR
March	1994

4. FIN OR GRANT NUMBER

L1004

5. AUTHOR(S)

G. E. Korth

6. TYPE OF REPORT

Technical

7. PERIOD COVERED (Inclusive Dates)

8. PERFORMING ORGANIZATION - NAME AND ADDRESS (If NRC, provide Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address; if contractor, provide name and mailing address.)

EG&G Idaho, Inc.  
Idaho Falls, ID 83415

9. SPONSORING ORGANIZATION - NAME AND ADDRESS (If NRC, (use "Same as above"; if contractor, provide NRC Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address.)

Division of Systems Research  
Office of Nuclear Regulatory Research  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

10. SUPPLEMENTARY NOTES

11. ABSTRACT (200 words or less)

Fifteen steel samples were removed from the lower pressure vessel head of the damaged TMI-2 nuclear reactor to assess the thermal threat to the head posed by 15 to 20 metric tons of molten core debris relocating there during the accident. Full sections of thirteen of the samples and partial sections of the other two samples underwent hardness and metallographic examinations at the Idaho National Engineering Laboratory. These examinations have shown that eleven of the fifteen samples did not exceed the ferrite-austenite transformation temperature of 727°C during the accident. The remaining four samples did show evidence of having a much more severe thermal history. The samples from core grid positions F-10 and G-8 are believed to have experienced temperatures of 1,040 to 1,060°C for about 30 minutes. Samples from positions E-8 and E-6 appear to have been subjected to 1,075 to 1,100°C for approximately 30 minutes.

12. KEY WORDS/DESCRIPTORS (List words or phrases that will assist researchers in locating the report.)

TMI-2, Metallography, Hardness, Thermal Damage, Accident Temperatures

13. AVAILABILITY STATEMENT

Unlimited

14. SECURITY CLASSIFICATION

(This Page)

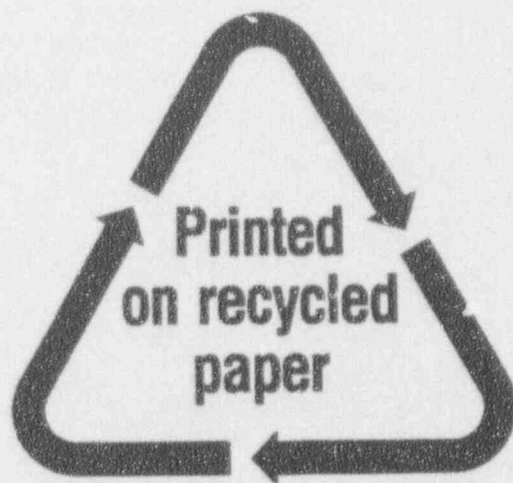
Unclassified

(This Report)

Unclassified

15. NUMBER OF PAGES

16. PRICE



Federal Recycling Program

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

OFFICIAL BUSINESS  
PENALTY FOR PRIVATE USE, \$300

SPECIAL FOURTH-CLASS RATE  
POSTAGE AND FEES PAID  
USNRC  
PERMIT NO. G-87