

Patrick M Donnelly Plant Manager

Big Rock Point Nuclear Plant, 10269 US-31 North, Charle-oix, MI 49720

April 4, 1994

Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of March 1, 1994 through March 31, 1994.

Sincerely,

P M Donnelly Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission DRHahn, Department of Public Heath JRPadgett, Michigan Public Service Commission RAben, Michigan Department of Labor MPCass, American Nuclear Insures FYost, Research Services Utility Data Institute INPO Records Center DWJoos, P24-117E GCWithrow, Big Rock Point NRC Resident Inspector, Big Rock Point Document Control, Big Rock Point, 740/22*35*10 File



NUCLEAR OPERATIONS DEPARTMENT Unit Shutdowns and Power Reductions

Report Month	Docket Number	Unit	Date	Completed by	Telephone
March, 1994	55-150	Big Bock Point Plant	April 4, 1994	JR JOPNSTON	(616) 547-8223

03/02/94	S 469.5 Hz	s H	1	-		
						On 03/02/94 at 14:21; The unit was removed from service to repair the following: minor steam leak: ECCS outlet valve MO-7053, #1 Reactor Recirculating pump seal replacement, and balance the turbine. After repairs were completed the unit was returned to service on 03/22/94 at 03:55 hrc.

¹F = Forced

S = Scheduled

²Reason:

A = Equipment Failure (Explain)

B = Maintenance of Test

C = Retueling

D = Regulatory Restriction

E = Operator Training and License Examination

F = Administrative

G = Gpantional Error (Explain)

H = Other (Explain)

3_{Method}:

1 = Manual

2 = Manual Scram

3 = Automatic Scram

4 = Other (Explain)

⁴Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155 DATE: 4 / 4 / 94

BY: JR JOHNSTON PHONE: 616-547-6537, EXT 223

OPERATING STATUS

1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT

NOTES:

2. REPORTING PERIOD: 3 / 94

3. LICENSED THERMAL POWER (MWT): 240

4. NAMEPLATE RATING (GROSS MWE): 75

5. DESIGN ELECTRICAL RATING (NET MWE): 72

6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0

7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0

C. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
- 10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	2160.0	271819.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	284.8	1700.8	196819.2
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	274.5	1690.5	193769.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	52416.0	372955.0	37275946.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	16936.0	119181.0	11841296.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	15987.8	113031.0	11202291.0
19. UNIT SERVICE FACTOR	36.9%	78.3%	71.3%
20. UNIT AVAILABILITY FACTOR	36.9%	78.3%	71.3%
21. UNIT CAPACITY FACTOR (USING MDC NET)	32.0%	78.1%	61.2%
22. UNIT CAPACITY FACTOR (USING DER NET)	29.8%	72.7%	57.2%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	11.1%

- 24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH):
- 25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

DAY	AVERAGE DAILY POWER(MWT)	(MWEN)
1	228.21	69.08
2	123.00	36.32
3	0.00	0.00
4	0.00	0.00
5	0.00	0.00
6	0.00	0.00
7	0.00	0.00
8	0.00	0.00
9	0.00	0.00
10	0.00	0.00
11	0.00	0.00
12	0.00	0.00
13	0.00	0.00
14	0.00	0.00
15	0.00	0.00
16	0.00	0.00
17	0.00	0.00
18	0.00	0.00
19	0.00	0.00
20	0.00	0.00
21	1.88	0.00
22	61.17	16.70
23	150.92	47.00
24	177.29	54.85
25	205.54	63.02
26	208.21	63.20
27	203.88	62.92
28	205.33	62.79
29	204.92	62.83
30	208.08	63.44
31	205.58	63.16

(3/94) - CYCLE 27

Refueling Information Request

- 1. Facility Name: Big Rock Point Plant
- 2. Scheduled date for next refueling shutdown: September, 1994.
- 3. Scheduled date for restart following shutdown: November, 1994.
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? Yes

If no review has taken place, when is it scheduled?

- 5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
- 6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
- 7. Number of fuel assemblies in: core 84; spent fuel pool storage 316; new fuel storage .
- 8. Present licensed spent fuel pool storage capacity: 441

 Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
- Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: Last total core off load 1996; Last refueling 1999.

REFUELING.DO6 March 1, 1994