



UNITED STATES  
 NUCLEAR REGULATORY COMMISSION  
 REGION II  
 101 MARIETTA ST., N.W., SUITE 3100  
 ATLANTA, GEORGIA 30303

Report Nos. 50-348/82-21, 50-364/82-20

Licensee: Alabama Power Company  
 600 North 18th Street  
 Birmingham, AL 35291

Facility Name: Farley

Docket Nos. 50-348, 50-364

License Nos. NPF-2, NPF-8

Inspection at Farley site near Ashford, Alabama

Inspector: E. Brooks  
 E. Brooks

8-25-82  
 Date Signed

Approved by: Frank Jape  
 F. Jape, Section Chief  
 Engineering Inspection Branch  
 Division of Engineering and Technical Programs

8/25-82  
 Date Signed

SUMMARY

Inspection on August 2-6, 1982

Areas Inspected

This routine, announced inspection involved 30 inspector-hours on site in the areas of pipe supports and restraints and type B and C containment leak rate testing.

Results

Of the areas inspected, no violations or deviations were identified.

## REPORT DETAILS

### 1. Persons Contacted

#### Licensee Employees

- \*G. Hairston, Plant Manager
- \*R. Berryhill, Systems Performance Superintendent
- \*J. Woodard, Assistant Plant Manager
- \*W. Shipman, Maintenance Superintendent

#### Other Organizations

- \*D. Urciola, Daniel Construction Company

#### NRC Resident Inspector

- \*W. Bradford

\*Attended exit interview

### 2. Exit Interview

The inspection scope and findings were summarized on August 6, 1982, with the persons indicated in paragraph 1 above. The licensee was informed of the inspection findings listed below. The licensee acknowledged the inspection findings and stated that Region II would be advised of their position by August 11, 1982.

Inspector Followup Item, 348/82-21-01, Report of Test Results (paragraph 6 of this report).

### 3. Licensee Action on Previous Enforcement Matters

Not inspected.

### 4. Unresolved Items

Unresolved items were not identified during this inspection.

### 5. Surveillance Routine Pipe Supports and Restraints (61729) - Units 1 and 2

The inspector reviewed the licensee's Surveillance Test Procedures -

- STP - 610.1      Hydraulic Snubber Functional Test
- STP - 610.2      Hydraulic Snubber Visual Test

Following are the results of snubber inspection and testing as documented in the licensee's data sheets:

|        | <u>SNUBBER<br/>LOCATION</u> | <u>INSPECTION<br/>DATE</u> | <u>FAILURE<br/>NUMBER</u> | <u>INSPECTION<br/>FREQUENCY</u> |
|--------|-----------------------------|----------------------------|---------------------------|---------------------------------|
| Unit 1 | Inaccessible                | 2/15/82                    | 1                         | 12 months                       |
|        | Accessible                  | 2/15/82                    | 0                         | 12 months                       |
| Unit 2 | Inaccessible                | 2/10/82                    | 2                         | 6 months                        |
|        | Accessible                  | 2/10/82                    | 1                         | 12 months                       |

Functional testing of snubbers to verify correct piston movement, lock-up and bleed has been conducted in accordance with technical specifications and meets the acceptance criteria as specified in procedure STP-610.1.

6. Surveillance - Containment Leak Rate Testing - Type B & C Tests (61720) - Units 1 and 2

The inspector reviewed the licensee's leakage testing documentation for both units 1 and 2 from the time of initial plant operation. Following are the chronologically listed dates during which containment integrated leakage rate tests (ILRT) and/or Type B and C (penetrations and isolation valves) were conducted:

Unit 1 -

|                           |              |
|---------------------------|--------------|
| Preoperational ILRT       | 2/77         |
| Commercial Operation      | 12/77        |
| First Refueling B&C Tests | 3/79 - 10/79 |
| Second Refuel B&C Tests   | 11/80 - 2/81 |
| First Periodic ILRT       | 1/81         |
| Third Refueling B&C Tests | 10/81 - 1/82 |

Unit 2 -

|                           |              |
|---------------------------|--------------|
| Preoperational B&C Tests  | 11/79 - 6/80 |
| Preoperational ILRT       | 6/80         |
| Commercial Operation      | 7/81         |
| Partial B&C Tests         | 2/82         |
| First Scheduled Refueling | 10/82        |

Note: Appendix J to 10 CFR requires type B&C testing to be performed during each reactor shutdown for refueling but in no case at intervals greater than two years. The licensee has received a one time exemption to extend the two year interval during the first fuel cycle to allow individual penetrations to be tested as plant conditions permit, but not to extend beyond the first refueling outage. (Refer to Technical Specifications Amendment 10)

A summary analysis of the type B & C tests conducted during the first refueling for Unit 1 were not included in the licensee's integrated leakage rate test report submitted to the NRC. Also, the report did not include leakage test results that failed to meet the acceptance criteria (0.6La), or an analysis of structural conditions of the components which contributed to failure in meeting the acceptance criteria. "Before and after" leakage test results for isolation valves were not included in the report, and certain isolation valves which could not be pressurized during testing were not identified as having exceeded acceptance criteria (0.6La). Much of the above detail is available in the licensee's test data sheets. The inspector requested this information be included in future test reports in order to evaluate apparent potential containment degradation.

During the exit interview, the licensee acknowledged the inspector's findings and agreed to advise Region II on August 12, 1982, of their proposed actions to document test results accordingly.

Following are the licensee's proposed actions as received by telephone conversation on August 12, 1982. "Farley Nuclear Plant will report as-found leakages to the extent we are capable of measuring them. The measurable as-found leakages will be reported as part of the Type A test report. Technical Specifications acceptance criterion shall be applied against final test results for determining operability for applicable modes. Measurements off-scale of our test equipment shall be reported as off-scale, or greater than the range of the equipment. Local leak rate test results will be reported as a failure when acceptance criteria cannot be achieved in an applicable Technical Specification mode."

Followup on future reports to ensure sufficient details are included for containment integrity determination is identified as an inspector followup item (348/82-21-01).

Within the areas examined no violations or deviations were observed.