TENNESSEE VALLEY AUTHORITY

CHATTANOOGA. TENNESSEE 37401 400 Chestnut Street Tower II

September 20, 1982

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BLRD-50-438/81-77 BLRD-50-439/81-76

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U.S. Nuclear Regulatory Commission Region II Attn: Mr. James P. O'Reilly, Regional Administrator 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303

Dear Mr. O'Reilly:

BELLEFONTE NUCLEAR PLANT UNITE 1 AND 2 - ALLOWABLE STRESSES FOR PIPE SUPPORT DESIGN - BLRD-50-438/81-77, BLRD-50-439/81-76 - THIRD INTERIM REPORT

The subject deficiency was initially reported to NRC-OIE Inspector R. V. Crlenjak on December 7, 1981 in accordance with 10 CFR 50.55(e) as NCR BLN CEB 8110. This was followed by our interim reports dated December 30, 1981 and May 11, 1982. Enclosed is our third interim report. We expect to submit our next report by December 17, 1982.

If you have any questions concerning this matter, please get in touch with R. H. Shell at FTS 858-2688.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills, Manager Nuclear Licensing

Enclosure cc: Mr. Richard C. DeYoung, Director (Enclosure) Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, D.C. 20555

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ENCLOSURE

BELLEFONTE NUCLEAR PLANT UNITS 1 AND 2 ALLOWABLE STRESSES FOR PIPE SUPPORT DESIGN BLRD-50-438/81-77, BLRD-50-439/81-76 10 CFR 50.55(e) THIRD INTERIM REPORT

Description of Deficiency

For the upset condition for primary load sources, the Bellefonte FSAR indicates that allowable stresses for support designs is 1.0 times the normal condition AISC allowable stresses. The procedures governing Bellefonte piping analysis indicate that a factor of 1.33 can be used. This deficiency was caused by the EP and handbook being issued with the 1.33 factor in anticipation that the ASME Code would adopt the 1.33 factor. The code adopted a factor of 1.0 instead of 1.33. The factor of 1.33 was used in the design of pipe supports.

Interim Progress

TVA is still in the process of evaluating the subject deficiency. Further details will be provided in our next report.