



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONNECTICUT YANKEE ATOMIC POWER COMPANY

DOCKET NO. 50-213

HADDAM NECK PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 131  
License No. DPR-61

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Connecticut Yankee Atomic Power Company (the licensee), dated July 26, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

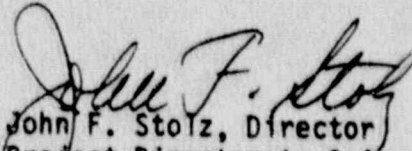
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-61 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 131, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Charges to the Technical  
Specifications

Date of Issuance: September 19, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 131

FACILITY OPERATING LICENSE NO. DPR-61

DOCKET NO. 50-213

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove

3/4 7-3

Insert

3/4 7-3



## AUXILIARY FEEDWATER SYSTEM

### LIMITING CONDITION FOR OPERATION

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3.7.1.2 Two independent steam turbine-driven auxiliary feedwater pumps capable of being powered from an OPERABLE steam supply system and associated flow paths shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

#### ACTION:

- a. With one steam turbine driven auxiliary feedwater pump inoperable, restore the inoperable auxiliary feedwater pump to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With two steam turbine-driven auxiliary feedwater pumps inoperable; restore at least one of the inoperable auxiliary feedwater pumps to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours. Restore both the auxiliary feedwater pumps to OPERABLE status within 72 hours from time of initial loss of the first auxiliary feedwater pump or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the next 6 hours.

### SURVEILLANCE REQUIREMENTS

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4.7.1.2.1 Each auxiliary feedwater pump shall be demonstrated OPERABLE at least once per 31 days on a STAGGERED TEST BASIS by:

- a. Verifying that each steam turbine-driven pump develops a discharge pressure of greater than or equal to 800 psig at a steam supply pressure of greater than or equal to 300 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3;
- b. Verifying that each non-automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position, and

4.7.1.2.2 At least once per 18 months, the Auxiliary Feedwater System shall be demonstrated OPERABLE by:

- a. Verifying the capability of each pump to attain rated flow of 450 gpm at 1050 psig;
- b. Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of an Auxiliary Feedwater Actuation signal; and
- c. Verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of an Auxiliary Feedwater Actuation signal.

(The provisions of Specification 4.0.4 are not applicable for entry into Mode 3 for Specification 4.7.1.2.2.a, b, and c.)