Nuclear

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September 15, 1990

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Brenda DeMerchant, Oyster Creek Licensing Engineer at (609) 971-4642.

Very truly yours,

E. E. Fitzpatrick

E. E. Fitzpatrick
Vice President and Director
Oyster Creek

EEF:BDEM:jc
(MOR)
Enclosures

cc: Mr. Thomas Martin, Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Alexander W. Dromerick, Project Manager
U.S. Nuclear Regulatory Commission
Washington, DC 20555

NRC Resident Inspector
Oyster Creek Nuclear Generating Station

IE24

MONTHLY OPERATING REPORT - AUGUST 1990

At the beginning of August, Oyster Creek was operating at full power. Power reductions were required for maintenance, as specified by the Technical Specifications, to perform condenser backwashing and due to discharge canal temperature limitations.

On August 25-26, 1990, power was reduced to isolate and clear the south half of 'C' main condenser. The plant was returned to full power on August 27, 1990.

MONTHLY OPERATING REPORT AUGUST 1990

The following Licensee Event Reports were submitted during the month of August 1990:

LER 90-010: Electromatic Relief Valve High Pressure Relief Setpoints Exceeded Technical Specification Limit Due to Drift

On July 18, 1990 while performing an Electromatic Relief Valve (EMRV) Pressure Sensor surveillance, the "As Found" trip setpoint for the high pressure relief function on three out of five EMRVs was above that specified in the Technical Specifications. Additionally, a review of records for this surveillance revealed that on April 14, 1988 and June 13, 1990, one EMRV had a high pressure setpoint that was above the limit. The cause of these occurrences is attributed to setpoint repeatability and instrument drift. The design setpoint repeatability can tolerate instrument drift within 2.5 psig of the Technical Specification limit. Previous surveillance records indicate that these instruments frequently undergo additional drift within Technical Specification limits due to changing plant and environmental conditions. This occurrence is considered to have minimal safety significance as the automatic depressurization function of the EMRVs is not affected by these pressure switches, all five EMRVs would have actuated to relieve pressure, and the Isolation Condenser System and turbine bypass valves were fully operable. The pressure switches were adjusted to actuate within the Technical Specification limit. A new pressure sensing system is included in the Oyster Creek Integrated Schedule.

LER 90-011: Unqualified Operators on Shift Due to Inadequacies in Exam Process Results in Violation of Tech Spec Shift Manning Requirements

In June and July of 1990, as a result of an NRC audit of the operator training program, 1989 biennial requalification exams were found to have been incorrectly graded. A regrading effort resulted in the failure of two licensed operators. The operators were immediately removed from licensed duties and entered into an accelerated requalification program. These operators had been performing licensed duties during the period from the requal exam up to the regrading of the exam. Since these two operators were retroactively disqualified, there were 89 shifts during this period with less than two control room operators as required by technical specifications. The cause of this occurrence is attributed to personnel error as a result of programmatic inadequacies in the exam process. An investigation and critique of this incident revealed that these inadequacies led to errors in the preparation, administration and grading of the 1989 written requalification exam. These inadequacies caused the grading anomalies identified. To prevent recurrence, an examination procedure will be developed to provide guidance for the preparation, administration and grading of exams.

LER 90-012: An Error in a Feedwater Flow Calculation Equation May Have Resulted in Operation of the Reactor in Excess of the License Limit

A decrease in plant performance had been noted for the current operating cycle. A leak in the high pressure feedwater reheaters was postulated and investigated. Visual inspections revealed no leaks. A detailed review of plant data was initiated on July 11, 1990 to locate the source of the performance decrease.

On August 1, 1990, it was noted that a procedure revision (approved on February 9, 1987) to the feedwater flow calibration calculation had resulted in approximately a 2% correction in indicated feedwater flow. This change resulted in a decrease in allowed plant power. Therefore, although indicated reactor power has been correct since February 9, 1987, prior to that date the reactor may have been operated slightly in excess of its license limit of 1930 megawatts thermal. The magnitude of the power anomaly and any resultant effects on safety significance are presently being reviewed. A supplemental Licensee Event Report will be submitted when the ongoing review is completed.

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #. 50219
UNIT. . . . . .OYSTER CREEK #1
REPORT DATE . . . . . .SEPTEMBER 7, 1990
COMPILED BY . . . . . .HARI S. SHARMA
TELEPHONE # 609-971-4638

MONTH AUGUST, 1990

DAY	MW	DAY	MW
1.	602	16	549
2.	602	17	[illegible]
3.	594	18.	505
4.	594	19.	518
5.	566	20.	585
6.	520	21.	606
7.	510	22.	609
8.	595	23.	611
9.	585	24.	609
10.	588	25.	268
11.	511	26.	295
12.	559	27.	563
13.	528	28.	586
14.	520	29.	572
15.	553	30.	526
		31.	597

Oyster Creek Station #1
Docket No. 50-219

REFUELING INFORMATION - AUGUST, 1990

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: February 15, 1991 pending necessary state approval.

Scheduled date for restart following refueling: May 31, 1991

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes

Technical Specification Change Request 180 was submitted to the NRC on 5-07-90. This submittal was made in accordance with GL 88-16 to incorporate cycle specific parameters in a core operating limits report.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC.

2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies	(a)	in the core	= 560
	(b)	in the spent fuel storage pool	= 1595
	(c)	in dry storage	= 37

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present licensed capacity: 2600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Reracking of the fuel pool is in progress. Nine (9) out of ten (10) racks have been installed to date. When reracking is completed, discharge capacity to the spent fuel pool will be available until 1994 refueling outage.

OPERATING DATA REPORT

OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: 08/90
3. UTILITY CONTACT: HARI S. SHARMA 609-971-4638
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): 687.5 X 0.8 = 550
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 642
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): None
11. REASON FOR RESTRICTION, IF ANY: None

		MONTH	YEAR	CUMULATIVE
12.	REPORT PERIOD HRS	744.0	5831.0	181367.0
13.	HOURS RX CRITICAL	744.0	4875.6	116136.2
14.	RX RESERVE SHTDWN HRS	0.0	0.0	918.2
15.	HRS GENERATOR ON-LINE	744.0	4752.2	112984.0
16.	UT RESERVE SHTDWN HRS	0.0	0.0	1208.6
17.	GROSS THERM ENER (MWH)	1328933	8375394	189648453
18.	GROSS ELEC ENER (MWH)	423850	2746500	63869240
19.	NET ELEC ENER (MWH)	407070	2634148	61289126
20.	UT SERVICE FACTOR	100.0	81.5	62.3
21.	UT AVAIL FACTOR	100.0	81.5	63.0
22.	UT CAP FACTOR (MDC NET)	88.2	72.9	54.5
23.	UT CAP FACTOR (DER NET)	84.2	69.5	52.0
24.	UT FORCED OUTAGE RATE	0.0	15.3	11.9
25.	FORCED OUTAGE HRS	0.0	859.9	15270.0

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):
Refueling outage 13R commencing February 15, 1991, scheduled to end May 31, 1991.
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP DATE: N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219
UNIT NAME Oyster Creek
DATE September, 1990
COMPLETED BY R. Baron
TELEPHONE 971-4640

REPORT MONTH August, 1990

NO.	DATE	TYPE F: Forced S: Scheduled	DURATION (Hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
110	8/06	F	0	H	1	Load Reduction required to backwash main condensers (416 MWe Gross).
111	8/06	F	0	H	1	Plant shutdown commenced due to high drywell unidentified leak rate following the inadvertent spray down of the drywell during Containment Spray surveillance testing. Shutdown terminated on August 6, 1990 at a plant load of 520 MWe when leak rate returned to normal. On August 7, 1990, load reduced to 203 MWe to facilitate MSIV testing. Returned to full load on August 7, 1990.
112	8/11	F	0	H	1	Load reduction to backwash main condensers (411 MWe Gross)

Summary :

(1) REASON

a. Equipment Failure (Explain)
b. Maintenance or Test
c. Refueling
d. Regulatory Restriction
e. Operator Training & License Exam
f. Administrative
g. Operational Error (Explain)
h. Other (Explain)

METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219
UNIT NAME Oyster Creek
DATE September, 1990
COMPLETED BY R. Baron
TELEPHONE 971-4640

REPORT MONTH August, 1990 (Continued)

NO.	DATE	TYPE F: Forced S: Scheduled	DURATION (Hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
113	8/14	F	0	H	1	Load Reduction required to backwash main condensers (374 MWe Gross).
114	8/19	F	0	H	1	Load reduction required to backwash main condensers (336 MWe Gross).
115	8/25	S	0	H	1	Load Reduction to isolate and clean 'C' South Main Condenser (274 MWe Gross). Returned to full load on August 27, 1990.
116	8/29	F	0	A	1	Commenced plant shutdown due to loss of ADS logic input upon isolation of Core Spray System 2. Isolation required to replace V-20-24. Shutdown terminated when ADS logic input was returned to operability at a plant load of 389 Mwe. Returned to full load on August 30, 1990.

Summary :

(1) REASON

a. Equipment Failure (Explain)
b. Maintenance or Test
c. Refueling
d. Regulatory Restriction
e. Operator Training & License Exam
f. Administrative
g. Operational Error (Explain)
h. Other (Explain)

METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)