

September 18, 1990

Docket No. 50-333

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Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - ACCIDENT MONITORING
INSTRUMENTATION AMENDMENT - JAMES A. FITZPATRICK NUCLEAR
POWER PLANT (TAC NO. 76938)

Based on our review of your Technical Specification amendment request dated May 30, 1990 (JPN-90-042), for the James A. FitzPatrick Nuclear Power Plant, we have identified the need for additional information in order to complete the review. We, therefore, request that you provide a response to the questions enclosed.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under PL 96-511.

Sincerely,

ORIGINAL SIGNED BY:

Daniel G. McDonald for

David E. LaBarge, Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: See next page

PDI-I:LA

CVogan

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[Signature]
PDI-I:PM

to DLaBarge:av1

9/18/90

[Signature]
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 18, 1990

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Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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Sincerely,

A handwritten signature in cursive script, appearing to read "David E. LaBarge".

for David E. LaBarge, Project Manager
Project Directorate 1-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: See next page

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Power Authority of the State of New York

James A. FitzPatrick Nuclear
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New York, New York 10271

REQUEST FOR ADDITIONAL INFORMATION

ACCIDENT MONITORING INSTRUMENTATION

1. The proposed submittal would add the requirement to Specification 3.2.H that the instrumentation in Table 3.2-8 "must be operable whenever the reactor is critical and reactor coolant temperature is greater than or equal to 212°F." Please address the following concerns related to this proposed change:
 - a. For all of the instruments in Table 3.2-8, the proposed change to Specification 3.2.H seems to imply that both conditions of coolant temperature and reactor criticality must be in effect before the instrumentation is required to be operable. However, this is inconsistent with the requirements of primary containment integrity Specification 3.7.A.2, which states that the existence of either condition requires that primary containment integrity be in effect. Since many of the instruments are associated with monitoring of the primary containment, Specification 3.7.A.2 would seem to form the basis for the operability requirements for the instruments associated with the primary containment in the proposed change.
 - b. The proposed change does not include the primary containment integrity requirement of Specification 3.7.A.2 concerning fuel in the reactor vessel.
 - c. The proposed change to Table 3.2-8 would add certain Residual Heat Removal (RHR) system, RHR Service Water, and Core Spray (CS) system instruments. However, the proposed changes to Specification 3.2.H and to Table 3.2-8 do not recognize the operability requirements of the RHR and the CS systems of Specification 3.5.A or the definition of "Operable" stated in Specification 1.0.J.
2. The present TS Table 3.2-6 indicates that the number of reactor pressure channels provided by design is three and that the minimum number of operable instrument channels is two. The proposal would move this parameter to Table 3.2-8 and would indicate that only two channels are provided by design and that the minimum number required would be one. This change to the number of channels is not discussed in the submittal.
3. There appears to be no justification supplied for the significant differences between the Action Statement in the present TS and that shown in the submittal, for the following parameters:
 - a. Torus Water Level
 - b. Reactor Water Level (wide range)

- c. Reactor Water Level (fuel zone)
- d. Reactor Vessel Pressure
- e. Torus Water Temperature
- f. Drywell Temperature
- g. Drywell/Torus Differential Pressure

Please ensure that for every change that is proposed in the submittal, a corresponding explanation is provided.

- 4. The present TS Table 3.2-6 contains the Suppression Chamber Water Level (Wide Range) instrument. However, the wide range instrument is not indicated in the proposed TS change and there is no explanation for its deletion.
- 5. To be consistent with other applications of the term, a "/" should be placed between "Safety" and "Relief" shown on TS page 77d in the submittal.
- 6. The submittal describes deletion of Table 4.2-6 from the TS, which is on page 84 of the present TS and moves the "NOTES FOR TABLES 4.2-1 THROUGH 4.2-5" onto page 84. The result is that there is no page to show that Table 4.2-6 is intentionally blank, as was done on the new page 76 for deletion of Table 3.2-6. Thus, the numbering sequence of the tables would go from 4.2-5 to 4.2-7.