

September 14, 1990

Docket No. 50-219

Mr. E. E. Fitzpatrick
Vice President and Director
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - SEP TOPIC III-7.B RELATED
TO DRYWELL TEMPERATURES, OYSTER CREEK NUCLEAR GENERATING STATION
(TAC NO. 76879)

By letter dated May 25, 1990 you provided information concerning SEP Topic III-7.B related to drywell temperature conditions at the Oyster Creek Nuclear Generating Station. We have reviewed the information and have determined that additional information is required in order for the staff to complete its review. The specific information requested is presented in the enclosure.

We request that the information be provided within 30 days of receipt of this letter. If you have any questions regarding this request, please contact me.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than 10 respondents; therefore, OMB clearance is not required under P.L. 97-511.

Sincerely,

original signed by Alexander Dromerick

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "Alexander W. Dromerick".

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. E. E. Fitzpatrick
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
Generating Station

cc:

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Licensing Manager
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Forked River, New Jersey 08731

Request for Additional InformationSEP Topic III.7B Related to Drywell TemperaturesOyster Creek Nuclear Generating Station

1. From the OCNCS drywell sketch, it appears that the shield wall extends up to the elevation 84' -0". In light of this, clarify the first sentence of your response. "Above elevation 94 ft.-0" in the Oyster Creek Drywell, the shield wall concrete is not insulated from the operating temperatures of the Reactor."
2. Provide a copy of Reference 5 or if submitted to NRC previously, provide date of the letter.
3. The response indicates that based on the thermocouple data, the average temperature in the shield wall varies between 150°F and 225°F. This is equivalent to the concrete temperatures near RPV shell to be in the order of 250°F to 300°F. The cooler side could be at drywell temperatures. These are sustained operating temperatures. Since you are following ACI 349-80, the Section A.4.3 is applicable here. A thorough evaluation of the shield wall concrete is necessary, and it should include the following:
 - a. Estimating the "as is" strengths and modules of elasticities of the concrete and reinforcing bars in a conservative manner.
 - b. Establishing the revised acceptance criteria.
 - c. Analysing the wall to withstand the postulated loads including the loads from the drywell truss and the RPV stabilizers.
 - d. Supporting the estimates with the inspection of accessible areas during an outage and core sampling or other NDE methods.

A summary of results is needed to address the above issues.

4. The shield wall is also subjected to sustained neutron and gamma radiation. Provide an assessment of the effects of the radiation together with that of temperatures, and factor it into response to 3 above.

DATED:

DISTRIBUTION

Docket File

NRC & Local PDRs

Plant File

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B. Boger (14A2)

S. Norris

A. Dromerick

OGC (for information only)

E. Jordan (MNBB 3302)

ACRS (10)

E. Wenzinger (Region 1)