



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-245

MILLSTONE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company (the licensee), dated November 22, 1993, as supplemented March 4, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

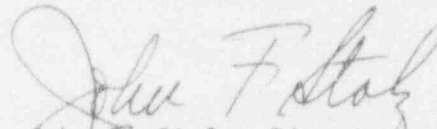
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-21 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 30, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-21

DOCKET NO. 50-245

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 2-1

3/4 2-7

3/4 2-8

Insert

3/4 2-1

3/4 2-7

3/4 2-8

LIMITING CONDITION FOR OPERATION

3.2 PROTECTIVE INSTRUMENTATION

Applicability:

Applies to the plant instrumentation which performs a protective function.

Objective:

To assure the operability of protective instrumentation.

Specification:

A. Primary Containment Isolation Functions

When primary containment integrity is required, the limiting conditions of operation for the instrumentation that initiates primary containment isolation are given in Table 3.2.1.

B. Emergency Core Cooling Subsystems Actuation

The limiting conditions for operation for the instrumentation which initiates the emergency core cooling subsystems are given in Table 3.2.2 except as noted in Specification 3.5.F.6.

C. Control Rod Block Actuation

1. The limiting conditions of operation for the instrumentation that initiates control rod block are given in Table 3.2.3.

SURVEILLANCE REQUIREMENT

4.2 PROTECTIVE INSTRUMENTATION

Applicability:

Applies to the surveillance requirements of the instrumentation that performs a protective function.

Objective:

To specify the type and frequency of surveillance to be applied to protective instrumentation.

Specification:

- A. The instrumentation to be functionally tested and calibrated as indicated in Table 4.2.1.
- B. When the reactor mode selector switch is in the REFUEL or SHUTDOWN position and fuel is in the reactor vessel, no control rod block functions are required to be operable provided that all control rods are fully inserted, and either electrically or hydraulically disarmed. Thereafter, daily surveillance shall be performed to verify that all controls rods remain either electrically or hydraulically disarmed.

TABLE 3.2.3

INSTRUMENTATION THAT INITIATES ROD BLOCK

Minimum Number of Operable Instrument Channels per Trip System ⁽¹⁾	Instrument	Trip Level Setting
1 ⁽⁷⁾	APRM Upscale (Flow Biased)	See Specification 2.1.2B
1 ⁽⁷⁾	APRM Downscale	$\geq 3/125$ Full Scale
1 ⁽⁸⁾	Rod Block Monitor Upscale (Flow Biased)	$\leq .65 W + 42$ ⁽²⁾
1 ⁽⁸⁾	Rod Block Monitor Downscale	$\geq 3/125$ Full Scale
3	IRM Downscale ⁽³⁾	$\geq 3/125$ Full Scale
3	IRM Upscale	$\leq 108/125$ Full Scale
2	SRM Detector not in Startup Position	⁽⁴⁾
2 ⁽⁵⁾	SRM Upscale	$\leq 10^6$ counts/sec.
— ⁽⁸⁾	Scram Discharge Volume - Water Level High	≤ 14 inches above lower cap to SDIV pipe weld
1	Scram Discharge Volume - Scram Trip Bypassed	N/A

TABLE 3.2.3 (Continued)
INSTRUMENTATION THAT INITIATES ROD BLOCK

Notes:

- (1) For the STARTUP/HOT STANDBY and RUN positions of the Reactor Mode Selector Switch, there shall be two operable or tripped trip systems for each function except the scram discharge volume — water level high and the SRM rod blocks; IRM downscale are not operable in the RUN position and APRM downscale need not be operable in the STARTUP/HOT STANDBY mode. If the first column cannot be met for one of the two trip systems, this condition may exist for up to seven days provided that during that time the operable system is functionally tested immediately and daily thereafter; if this condition lasts longer than seven days, the system shall be tripped. If the first column cannot be met for both trip systems, the systems shall be tripped.
- (2) W is the recirculation flow required to achieve rated core flow expressed in percent.
- (3) IRM downscale may be bypassed when it is on its lowest range.
- (4) This function may be bypassed when the count rate is ≥ 100 cps or when all IRM range switches are above Position 2.
- (5) One of these trips may be bypassed. The SRM function may be bypassed, in the higher IRM ranges, when the IRM upscale rod block is operable.
- (6) The trip may be bypassed when the reactor power is $\leq 30\%$ of rated. A RBM channel will be considered inoperable if there are less than half the total number of normal inputs from any LPRM level.
- (7) There must be a total of at least four (4) operable or operating APRM channels.
- (8) For the STARTUP/HOT STANDBY and RUN positions of the Reactor Mode Selector Switch, there shall be one (1) scram discharge volume — water level high instrument per instrument volume tank required to be operable. If either of the instruments become inoperable, a rod block shall be initiated within one hour.