VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT NO. 82-07

JULY, 1982

APPROVED BY: J. W. Wilson
Station Manager

OPERATING DATA REPORT

DOCKET NO. 50-280
DATE 06 AUG 82
COMPLETED BY SUE D. DUNN
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1	
2. REPORTING PERIOD	70182 TO 73182	
3. LICENSED THERMAL POWER (MWT)	2441	NOTES
4. NAMEPLATE RATING (GROSS MWE)	847.5	
5. DESIGN ELECTRICAL RATING (NET MWE)	788	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775	
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A	
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A	
10. REASONS FOR RESTRICTIONS, IF ANY	N/A	

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	5087.0	84215.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	736.6	4637.1	50671.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3731.5
14. HOURS GENERATOR ON-LINE	725.7	4569.3	49644.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	3736.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1751564.0	10857907.0	115191173.4
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	543530.0	3419760.0	37238973.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	514302.0	3242167.0	35319403.0
19. UNIT SERVICE FACTOR	97.5 %	89.8 %	58.9 %
20. UNIT AVAILABILITY FACTOR	97.5 %	89.8 %	63.4 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	89.2 %	82.2 %	54.1 %
22. UNIT CAPACITY FACTOR (USING DER NET)	87.7 %	80.9 %	53.2 %
23. UNIT FORCED OUTAGE RATE	2.5 %	4.2 %	23.9 %

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE,DATE,AND DURATION OF EACH): FALL MAINTENANCE - 10-01-82- 14 Days

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

DOCKET NO. 50-281
DATE 06 AUG 82
COMPLETED BY C.J. COSTELLO
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 2	
2. REPORTING PERIOD	70182 TO 73182	
3. LICENSED THERMAL POWER (MWT)	2441	NOTES
4. NAMEPLATE RATING (GROSS MWE)	847.5	
5. DESIGN ELECTRICAL RATING (NET MWE)	788	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775	
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A	
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A	
10. REASONS FOR RESTRICTIONS, IF ANY	N/A	

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	5087.0	81095.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	4532.1	49392.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	744.0	4461.2	48572.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1805997.5	10042177.5	113339463.4
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	580480.0	3223905.0	36901084.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	550489.0	3043453.0	34972168.0
19. UNIT SERVICE FACTOR	100.0 %	87.7 %	59.9 %
20. UNIT AVAILABILITY FACTOR	100.0 %	87.7 %	59.9 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	95.5 %	77.2 %	55.6 %
22. UNIT CAPACITY FACTOR (USING DER NET)	93.9 %	75.9 %	54.7 %
23. UNIT FORCED OUTAGE RATE	0.0	3.7 %	16.2 %

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): FALL MAINTENANCE - 11-05-82--10 Days

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-280
UNIT NAME Surry 1
DATE August 9, 1982
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

REPORT MONTH July 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-19	07-13-82	F	18.3	G	3	82-02/03L-0			"B" Reactor trip bypass breaker was not racked in properly prior to Instrument Dept. testing of Train "B" Reactor Trip signals. This caused a Reactor Trip. Operators have been reinstructed on proper procedures for racking in reactor trip bypass breakers and verifying proper insertion of breaker in cubicle.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-281
UNIT NAME Surry 2
DATE August 9, 1982
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

REPORT MONTH July 82

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-33	7-6-82	S	0.0	F					Reduced power to load follow on orders of the System Operator.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 1 MONTH: July, 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON

NONE DURING THIS REPORTING PERIOD.

MONTHLY TOTAL	

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2 MONTH: July, 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON
			NONE DURING THIS REPORTING PERIOD.			
				MONTHLY TOTAL		

DOCKET NO 50-280
UNIT SURRY I
DATE 8-1-82
COMPLETED BY G J COSTELLO

AVERAGE DAILY UNIT POWER LEVEL

MONTH: JULY 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	730.1	17	718.8
2	718.8	18	720.5
3	723.6	19	730.1
4	725.2	20	717.6
5	729.6	21	716.0
6	730.0	22	717.4
7	729.0	23	717.9
8	718.9	24	713.1
9	707.7	25	711.4
10	723.7	26	709.6
11	726.5	27	705.8
12	725.3	28	706.4
13	420.5	29	710.6
14	228.4	30	705.8
15	670.9	31	713.5
16	718.2		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

DOCKET NO 50-281
UNIT SURRY II
DATE 8-1-82
COMPLETED BY G J COSTELLO

AVERAGE DAILY UNIT POWER LEVEL

MONTH: JULY 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	746.3	17	741.1
2	745.5	18	742.3
3	745.4	19	746.3
4	745.6	20	734.6
5	748.1	21	740.2
6	708.6	22	739.4
7	744.8	23	738.5
8	743.0	24	735.8
9	743.1	25	726.0
10	747.2	26	735.0
11	746.6	27	738.4
12	742.6	28	738.7
13	741.7	29	732.3
14	740.0	30	740.5
15	740.3	31	744.6
16	740.1		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

SUMMARY OF OPERATING EXPERIENCE

July, 1982

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

July 1 — This reporting period begins with the unit at 100% power.

July 2 — 1154 - Turbine runback to 92% power on Overtemperature ΔT signal. The runback signal was a result of OTΔT setpoint reduction due to reduced primary pressure. The primary pressure reduction was a result of a problem encountered with a pressurizer spray valve.

1220 - Started power increase at 3% per hour.

1715 - The unit reached 100% power.

July 13 — 1400 - The reactor tripped as a result of instrument department testing of the "B" Train Reactor Trip signals with the "B" Reactor Trip Bypass Breaker improperly racked into the breaker cubicle.

1612 - The reactor is critical.

1828 - Chemist reported dose equivalent I-131 was greater than 10 microcuries per cc. Started six hours clock for having unit shutdown and cooled down to less 500°F.

2253 - The reactor was manually shutdown.

2330 - The reactor coolant system was less than 500°F.

July 14 — 0145 - Chemist reported dose equivalent. I-131 was less than 10 microcuries per cc.

0405 - The reactor was critical.

0819 - The generator was placed on the line.

0842 - Stopped power increase at 35% power to verify secondary chemistry was in specification.

0900 - Secondary chemistry is in specification. Started increasing power at 3% per hour.

July 15 — 1335 - The unit reached 100% power.

July 21 2200 - The supply breaker for the 2G Screenwell Transformer tripped de-energizing four (4) of the seven (7) operating circulating water pumps. Power was reduced to 95% to allow throttling the circulating water flow through the main condenser.

2210 - Cross-tied the 1G and 2G buses and restarted the circulating water pumps fed from the 2G bus.

2220 - Started increasing power at 3% per hour.

July 22 0100 - The unit reached 100% power.

July 31 This reporting period ends with the unit at 100% power.

UNIT TWO

July 1 This reporting period begins with the unit at 100% power.

July 6 0100 - Start power reduction at 150 MWe per hour to load follow on orders of the system operator.

0236 - Stopped load decrease at 69% power on orders of the system operator.

0438 - Started load increase on orders of the system operator.

0700 - The unit reached 100% power.

July 20 1745 - Both RWST temperature indicators indicate 46°F. Started 6 hour clock for hot shutdown based on exceeding maximum RWST temperature of 45°F.

1932 - Started power decrease at 150 MWe per hour.

2033 - Both RWST temperature indicators indicate 45°F. Stopped power decrease at 89% power.

2108 - Started increasing power.

2215 - The unit reached 100% power.

July 21 2210 - Decreased load 25 MWe due to loss of four (4) of seven (7) operating circulating water pumps.

2239 - Increased power to 100%.

July 25 2115 - Decreased power 15 MWe due to vacuum problems encountered while cleaning condenser waterboxes.

July 26 0518 - Increased power to 100%.

July 31 This reporting period ends with the Unit at 100% power.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

July, 1982

The Nuclear Regulatory Commission, on June 17, 1982, issued Amendment Nos. 1 and 2 respectively. The changes have been designated as Technical Specification change No. 88.

These amendments revise the Technical Specifications to limit control rod misalignment to no more than ± 12 steps indicated positions. This change is consistent with the Standard Technical Specifications.

Also, on page 3.16-2, a typographical error was corrected from 2 hours to 24 hours.

Accordingly, the paragraph 3.B of the Operating License for Unit 1 and 2, respectively, is amended as follows:

(Unit 1) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 78, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

(Unit 2) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

FACILITY CHANGES REQUIRING NRC APPROVAL

July, 1982

None during this reporting period.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

July, 1982

		Unit
D/C 80-76	Process Vent	1 & 2

This design change improves the performance and reliability of the process vent system. This was accomplished by the installation of a moisture trap on the inlet to the process vent to prevent water ingress to the filter media.

Summary of Safety Analysis

The modification will not create any safety problems. It will remove the water entering the system through the system input and give an indication in the control room if water enters the filter banks.

D/C 81-19	Machine Shop Replacement Facility	1 & 2

D/C 81-19E Machine Shop Replacement Facility - Electrical Systems and
D/C 81-19F Machine Shop Replacement Facility - Electrical Power Supply were implemented.

Summary of Safety Analysis

The addition of the Machine Shop Replacement Facility does not minimize the safety of operating untis or effect the operation of safety related equipment.

D/C 81-30	Surry Main Switchyard and Gas Turbine Area Fire Protection	1 & 2

This design change provides a water supply and fire fighting equipment to the gas turbine area, material storage area and switchyard area.

Summary of Safety Analysis

The modification will not reduce the safety of operating units, and fire protection capabilities to safety-related equipment will be maintained.

D/C 81-109 Steam Line to Auxiliary Boiler Drum Heater 1 & 2

This design change provides a tie in from the Building Heating System to supply the new Auxiliary Boiler Drum Heating Units. A steam heating element was installed in the lower drum of each of two Auxiliary Boilers during their recent retubing. The elements provide heat to the boiler during periods when the boilers are idle in order to reduce boiler startup time and reduce boiler corrosion.

Summary of Safety Analysis

The addition of the steam line from the heating steam system to the Auxiliary Boiler drum heating element does not affect the operation of any safety-related equipment.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

July, 1982

None during this reporting period.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

July, 1982

Special Test No.	Unit	Title	Completed
ST-52	1	RCS Flow Measurement Data	07-01-82
ST-89	2	Flow Coastdown Measurement	07-22-82
ST-90	2	S/G Water Level, Stability and Control Demonstration	07-22-82
ST-105	2	S/G Water Level, Stability and Control Demonstration	07-22-82
ST-106	1	Containment Sump Penetrations No. 66 and No. 69	07-22-82
ST-109	1	Engineered Safety Features Functional Test	07-22-82
ST-115	1	Chemical and Volume Control System Hot Functional Testing	07-22-82
ST-120	1	S/G Water Level, Stability and Control Demonstration	07-22-82
ST-134	1	S/G Moisture Carryover Measurement	07-22-82

OTHER CHANGES, TESTS AND EXPERIMENTS

July, 1982

None during this reporting period.

SURRY POWER STATION

CHEMISTRY REPORT

July 19 82

T.S. 6.6.3.d

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., µCi/ml	(A) Eo 6.64	(A) Eo 2.02	(A) Eo 3.35	E-1 4.86	E-1 1.64	E-1 2.62
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, µCi/ml	E-1 1.31	E-1 1.19	E-1 1.25	E-1 2.22	E-1 2.08	E-1 2.15
Iodine-131, µCi/ml	(A) Eo 6.72	(A) E-2 9.22	(A) E-1 4.77	E-3 4.21	E-3 1.23	E-3 2.06
I-131/I-133	1.5026	.8830	1.1499	1.8776	.4451	.9087
Hydrogen, cc/kg	(C) 54.8	25.7	32.6	(C) 58.0	20.1	33.2
Lithium, ppm	1.45	(B) .54	.85	1.35	1.05	1.24
Boron-10, ppm +	101.53	71.15	79.32	138.96	127.40	133.17
Oxygen-16, ppm	.000	.000	.000	.000	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	6.96	6.47	6.77	6.69	6.56	6.62

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL (D)
RELEASES, POUNDS
T.S. 4.13.A.6

Phosphate	-	Boron	890
Sulfate	-	Chromate	0.0
50% NaOH	-	Chlorine	-

REMARKS: (A) Activity levels indicate possible failed fuel (B) Two separate $LiOH$ additions made (7-8-82) &(7-22-82) (C) Hydrogen levels too high-recommended reduce pressure on the VCT - Unit #1 Rx trip 7-13-82 - unit startup 7-14-82 (D) The levels of these chemicals should create no adverse environmental impact.

DESCRIPTION OF ALL INSTANCES WHERE THERMAL DISCHARGE LIMITS WERE EXCEEDED

July, 1982

Due to the impairment of the circulating water system on the following days, the thermal discharge limits were exceeded as noted.

July 1, 1982	Exceeded 15°F	ΔT across station*
July 2, 1982	Exceeded 15°F	ΔT across station
July 3, 1982	Exceeded 15°F	ΔT across station*
July 5, 1982	Exceeded 15°F	ΔT across station*
July 6, 1982	Exceeded 15°F	ΔT across station*
July 7, 1982	Exceeded 15°F	ΔT across station
July 8, 1982	Exceeded 15°F	ΔT across station
July 9, 1982	Exceeded 17.5°F	ΔT across station*
July10, 1982	Exceeded 15°F	ΔT across station*
July12, 1982	Exceeded 15°F	ΔT across station
July13, 1982	Exceeded 15°F	ΔT across station*
July14, 1982	Exceeded 15°F	ΔT across station*
July15, 1982	Exceeded 17.5°F	ΔT across station*
July16, 1982	Exceeded 15°F	ΔT across station*
July17, 1982	Exceeded 17.5°F	ΔT across station
July18, 1982	Exceeded 17.5°F	ΔT across station*
July19, 1982	Exceeded 15°F	ΔT across station*
July20, 1982	Exceeded 15°F	ΔT across station
July21, 1982	Exceeded 15°F	ΔT across station
July22, 1982	Exceeded 15°F	ΔT across station
July23, 1982	Exceeded 17.5°F	ΔT across station
July24, 1982	Exceeded 17.5°F	ΔT across station*
July25, 1982	Exceeded 17.5°F	ΔT across station
July26, 1982	Exceeded 15°F	ΔT across station*

July 27, 1982	Exceeded 17.5°F	ΔT across station
July 28, 1982	Exceeded 17.5°F	ΔT across station
July 29, 1982	Exceeded 15°F	ΔT across station*
July 30, 1982	Exceeded 15°F	ΔT across station*

*Indicates dates where station ΔT was less than or equal to 15.0°F across station for some time during the day.

The ΔT excursions were allowable under Technical Specification 4.14.B.2. There were no reported instances of adverse environmental impact.

The temperature change at the station discharge exceeded 3°F per hour on July 13, 1982, due to a Unit 1 reactor trip. This event was allowable in accordance with Technical Specification 4.14.B.1. There were no reported instances of adverse environmental impact.

The temperature change at the station discharge exceeded 3°F per hour on July 21, 1982, due to a loss of four (4) of seven (7) operating circulating water pumps. The four (4) circulating water pumps were temporarily out of service due to an electrical malfunction. This event was reported in accordance with Technical Specification 4.14.C.1.

FUEL HANDLING

July, 1982

Units One and Two

None during this reporting period.

PROCEDURE REVISIONS THAT CHANGED THE OPERATING MODE DESCRIBED IN THE FSAR

July, 1982

None during this reporting period.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

July, 1982

None during this reporting period.

INSERVICE INSPECTION

July, 1982

No Inservice Inspections were performed on Units One or Two.

REPORTABLE OCCURRENCES PERTAINING TO
ANY OUTAGE OR POWER REDUCTIONS

July, 1982

None during this reporting period.

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Mechanical Maintenance

DEPT=MECH

UNIT1

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTLMTN
07/15/82	CH	PUMP	1-CH-P-1B	EXCESSIVE BEARING VIBRATION	REPLACED INBOARD BEARING	1	203130700	986
07/15/82	FW	VALVE	FCV-FW-1498	VALVE PERFORMS ERRATICALLY	REPLACED DIAPHRAM	1	203151932	41
07/15/82	CH	PUMP	1-CH-P-1B	REMOVE OIL COOLER	PLUGGED TUBE	1	207052250	120
								1147

DEPT TOTAL

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Mechanical Maintenance

DEPT=MECH

UNIT2
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RPTSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWNTM
07/06/82	IA	COMPRESS	2-IA-C-4A	COMPRESSOR WILL NOT LOAD UP	CHECK AT SYSTEM	2	206121410	645

DEPT TOTAL

Maintenance of Safety Related Systems During Outage or Reduced Power Periods

UNIT #1

Electrical Maintenance

DEPT=ELEC

UNIT1

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKFERF	U	NR	TOTIWKTM
07/13/82	EE	BREAKER	1-A1-221	TYRAP PLUG IN TERMINAL STRIPS	TYRAPPED STRIPS	1	207091430	5
07/13/82	EE	BREAKER	1-J1-2143	TYRAP PLUG IN TERMINAL STRIPS	TYRAPPED STRIPS	1	207091431	5
07/13/82	EE	BREAKER	1-A1-231	TYRAP PLUG IN TERMINAL STRIPS	TYRAPPED STRIPS	1	207091432	5
07/13/82	EE	BREAKER	1-J1-211	TYRAP PLUG IN TERMINAL STRIPS	TYRAPPED STRIPS	1	207091433	5
07/14/82	KP	B RX TRI	B	INSPECT B RX TRIP BYPASS	TESTED SAT	1	207131540	17
								37

DEPT TOTAL

Maintenance of Safety Related Systems During Outage or Reduced Power Periods

UNIT #2

Electrical Maintenance

Maintenance of Safety Related Systems During Outage or Reduced Power Periods

July, 1982

None during this reporting period.

Maintenance of Safety Related Systems During Outage or Reduced Power Periods

UNIT #1

Instrument Maintenance

DEPT=INST

Unit 1

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RPTSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTMANIN
07/15/82	CV	INSTR	FI-CV-150	DOES NOT INDICATE CORRECTLY INDICATE	REPLACED NOZZLE AND TRANSMITTER	1	205052345	469
07/15/82	CC	INSTR	FT-CC-110B	CALIBRATE TRANSMITTER	RECALIBRATE TRANSX	1	207050640	162
								631

DEPT TOTAL

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Instrument Maintenance

Maintenance of Safety Related Systems During Outage or Reduced Power Periods

July, 1982

None during this reporting period.

HEALTH PHYSICS

July, 1982

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CFR20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

July, 1982

Procedure No.	Unit	Title	Date Deviated	Date SNSOC Reviewed
PT-18.2	2	Safety Injection System Tests	12-22-81	07-22-82
PT-17.2	1	Containment Inside Recirculation Spray Pumps Tests	07-09-82	07-30-82