

March 29, 1994

Docket Nos. 50-277
and 50-278

Mr. George A. Hunger, Jr.
Director-Licensing, MC 52A-5
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, Pennsylvania 19087-0195

Dear Mr. Hunger:

SUBJECT: SECOND REQUEST FOR ADDITIONAL INFORMATION REGARDING POWER RERATE
REQUEST, PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 (TAC NOS.
M86826 AND M86827)

This letter forwards a request for additional information (RAI) regarding your
June 23, 1993, license amendment request. Your requested amendment would
allow a power rerate which would increase the authorized maximum reactor core
power level by five percent to 3458 megawatts thermal.

You are requested to respond to this RAI within 30 days of receipt of this
letter. The information requested is needed to allow us to continue our
review of your submittal. This is the second of several RAIs that we expect
to issue on your submittal.

This requirement affects less than ten respondents and, therefore, is not
subject to Office of Management and Budget review under P.L. 96-511.

If you have any questions regarding this RAI, please call me at (301) 504-
1422.

Sincerely,

/s/

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Request for Additional Information

cc w/enclosure:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Stephen Dembek".

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Request for Additional Information

cc w/enclosure:
See next page

REQUEST FOR ADDITIONAL INFORMATION
PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3
POWER UPRATE
DOCKET NOS. 50-277 AND 50-278

Your submittal for power uprate does not discuss the instrument setpoint methodology. The staff is unable to determine whether the General Electric Co. (GE) setpoint methodology discussed in GE Topical Report NEDC-31336 was used, or a plant-specific setpoint methodology was used for this application.

If a plant-specific setpoint methodology was used, then provide the following information:

1. Discuss your setpoint methodology. Include a discussion on how it differs from NEDC-31336.
2. The calculations and related documents that were used to derive the new trip setpoint and allowable values for the following parameters:
 - a. reactor vessel steam dome pressure, high
 - b. flow biased simulated thermal power, high
 - c. main steam line flow, high.

If the GE setpoint methodology is used, then provide the following information:

1. Provide a statement indicating that you used the GE topical report.
2. The staff's Safety Evaluation Report (SER) on GE Topical Report NEDC-31336 identified certain plant-specific information needed to justify the application of the report. Provide a discussion regarding the applicability of the topical report.
3. Confirm that the calculation for the instrument setpoint is identical to that used by the plant on which the topical report is based. If not, then justify the differences.