

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JULY, 1982

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by John Ullrich, telephone number (815)357-6761, extension 481.

II. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

July 1 - 20 Unit was in cold shutdown.

July 20 - 21 Reactor startup, sequence B commenced at 23:00 hours on July 20, 1982. Criticality was achieved at 3:04 hours on July 21, 1982. This criticality was terminated after 15 hours 50 minutes.

July 22 - 24 Reactor startup commenced at 3:37 hours on July 22, 1982, achieving criticality at 5:15 hours. This criticality was terminated after 64 hours 57 minutes due to a high vessel pressure of 1043#. The NRC notification was made after 1 hour.

July 25 - 26 The reactor went critical at 9:40 hours on July 25, 1982, for 13 hours 10 minutes when it was scrammed for cooldown.

July 26 - 31 Unit was in cold shutdown.

Startup testing and maintenance on the unit continued throughout the month.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY
RELATED MAINTENANCE

A. The following is a list of amendments to the facility license
and technical specifications:

AMENDMENT 1. In Mr. A. Schwencer's letter to Mr. Louis O. DelGeorge
of June 21, 1982:

- a. Modify Tech Spec 4.6.6.1(b)(2) from $1200 \pm 9^{\circ}\text{F}$
to $1200 \pm 25^{\circ}\text{F}$.
- b. Change the main steam isolation valve scram set-
point from a nominal value of 94% open to a
nominal value of 92% open.
- c. Modify Tech Spec 3/4.10.7 so as to permit perform-
ance of the confirmatory flow induced vibration
test with one low pressure core injection loop
isolated.

AMENDMENT 2. In Mr. A. Schwencer's letter to Mr. L. O. DelGeorge
of July 9, 1982:

- a. To reduce the count rate of the source range
monitors from 3 cps to .7 cps for modes other than
shutdown with a minimum allowable value of .5 cps
instead of 2 cps.

- b. To revise Tech Spec 4.4.3.2.2.B alarm setpoint for the reactor core isolation cooling system from 60 in² to 90 in².

B. Facility or Procedure Changes Requiring NRC Approval

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following table present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L14680	---	SRV 1B21-F013G, L cable & connector	Connectors broken	No effects - were in shutdown	Connector replaced
L15067	---	RHR & LPCS Pump Discharge Pressure switches 1E12-N022A, B, C & 1E12-N005	Reset span too large to operate between Tech Spec limit and normal operating pressure	No effect - plant was shutdown	Replaced and calibrated pressure switch.
L15192	---	1E12-F087A input to ESF status panel	Burned 74 relay	No effect - plant was shutdown	Replaced 74 relay
L15375	---	Neutron monitoring	Overheated resistors	No effect - plant was shutdown	Replaced resistor - transformer with a redesigned resistor in the power supplies to the neutron monitors
L16540	---	"A" Outboard MSIV	Bad valve seating	No effect - plant was shutdown	Valve seats were lapped/ground and reassembled
L17067	---	RCIC drain trap level switch	Float rod stuck	No effect - plant was shutdown	Rewired, replaced, recalibrated new switch
L17146	---	ID RHR service H ₂ O pump	Bad motor	No effect - plant was shutdown	Replaced with new motor
L17264	---	1A RHR SWP	Trip settings	No effect	Tested trip settings
L17353	---	Drywell Personnel airlock	"O" rings leaked during local leak rate test	No effect - plant was shutdown	Installed new "O" rings
L17400	---	"B" RHR	Transmitter for pressure gauges were out of spec.	No effect	Recalibrated transmitter
L17418	---	Div II Post LOCA % Oxygen monitor	Malfunctioning flow-raters	No effect - plant was shutdown	Cleaned the flowraters
L17423	---	1B RHR SP spray valve	Torque switch bad	Resulted in having only one S.P. spray loop. Two required by Tech Specs	Changed & set torque switch

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L17443	LER82-061/03L-0	1B/1C RHR injection line integrity monitor	RHR B/C injection lines monitor out of calibration limits	No effect	Recalibrated monitor - bled air from sensing lines
L17467	---	HPCS D/G petter air compressor	Starter not engaging	Motor driven compressor was still available	Replaced woodruff key & set screw on starter bendix shaft
L17469	LER82-064/03L-0	U SRV	Improper interaction between the magnet & the wired set of reed switches	Valve still operated correctly	Wiring changed to spare reed switches
L17491	LER82-066/03L-0	RCIC Turbine	Oil leaking	The HPCS system was operable	Tightened fittings replaced oil filter gaskets
L17493	LER82-068/03L-0	H safety relief valve	Improper interaction between the magnet & the wired set of reed switches	Valve still operated properly	Cleaned magnet & changed wiring to the spare set of reed switches
L17515	---	RCIC Vacuum pump	Pump not pulling negative vacuum	The HPCS system was operable	Chained lubricator & installed plug in sight glass
L17526	---	Vacuum pump Discharge check valve	Dirty disc & valve	The HPCS system was operable	Disassembled & cleaned valve
L17545	---	1E51-D006	RCIC pump does not achieve rated flow	The HPCS system was operable	Modification to increase orifice size
L17593	LER82-0/7/03L-0	RCIC valves 1E51-F354	Valves opened but failed to reclose	The HPCS system was operable	Adjusted limits switch
L17595	LER82-078/03L-0	Snubber 1RH-04-15 205	Improper weld	No effect	Repaired weld
L17596	---	RCIC Inboard Testable Check	Limit switch out of adjustment	The HPCS system was operable	Adjusted limit switch

WORK REQUEST

LER

COMPONENT

CAUSE OF MALFUNCTION

RESULTS AND EFFECTS
ON SAFE OPERATION

CORRECTIVE ACTION

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L17710	---	SBGT Rad. Mon. OPL058JB	Control not adjusted correctly	No effect - Unit was in cold shutdown	Reset RM-23

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, July 1 to July 31, 1982. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-056/03L-0	July 7	Missed surveillance on Reactor Water pH and Chloride Sampling Analysis
82-057/03L-0	July 8	"A" RHR Service Water Process Radiation Monitor
82-059/03L-0	July 12	"A" RHR Service Water Process Radiation Monitor
82-060/03L-0	July 17	Chlorine and Ammonia Detection System failure
82-061/03L-0	July 20	RHR B/C Integrity Monitor
82-062/03L-0	July 20	Reactor Coolant System Leakage Detection Systems
82-063/03L-0	July 20	APRM D Neutron Flux High
82-064/03L-0	July 21	ADS Valve 1B21-F013 U position Indication
82-065/01T-0	July 22	Vacuum Breaker Analysis to Meet Code Allowables
82-066/03L-0	July 22	Failure of RCIC Governor
82-067/03L-0	July 22	0A VE System
82-068/03L-0	July 22	Safety Relief Valve Position Indicator
82-069/03L-0	July 22	Low Drywell Pressure

IV. LICENSEE EVENT REPORTS (CONTINUED)

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-070/03L-0	July 24	Missed Station Vent Stack Surveillance
82-071/03L-0	July 25	Missed Samples Following Reactor Shutdown
82-072/03L-0	July 24	Group 1 Isolation and Reactor Scram
82-073/03L-0	July 25	Heatup Rate Exceeded
82-074/03L-0	July 19	Post LOCA H ₂ & O ₂
82-075/03L-0	July 26	Failure of ADS Accumulator IN Line
82-076/03L-0	July 27	RCIC Steam Supply Pressure Low Isolation
82-077/03L-0	July 26	Failure of RCIC Cont. Isolation Valve 1E51-F355
82-078/03L-0	July 26	Inoperable Snubber RH04-15205

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT 1
 DATE 8/5/82
 COMPLETED BY John Ullrich
 TELEPHONE 815/357-6761, X-481

OPERATING STATUS

1. REPORTING PERIOD: July, 1982 GROSS HOURS IN REPORTING PERIOD: 744.0
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 5% MAX. DEPEND. CAPACITY (MWe-Net): 0
 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 5%
 4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	93.95	99.05	99.05
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	0	0	0
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	428	428	428
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	0	0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0	0	0
12. REACTOR SERVICE FACTOR	NA	NA	NA
13. REACTOR AVAILABILITY FACTOR	NA	NA	NA
14. UNIT SERVICE FACTOR	NA	NA	NA
15. UNIT AVAILABILITY FACTOR	NA	NA	NA
16. UNIT CAPACITY FACTOR (Using MDC)	NA	NA	NA
17. UNIT CAPACITY FACTOR (Using Design MWe)	NA	NA	NA
18. UNIT FORCED OUTAGE RATE	NA	NA	NA

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 8/2/82

UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY		<u>6/21/82</u>
INITIAL ELECTRICITY	<u>9/1/82</u>	
COMMERCIAL OPERATION	<u>10/1/82</u>	

ATTACHMENT A
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373
UNIT 1
DATE 8/5/82
COMPLETED BY John Ullrich
TELEPHONE 815/357-6761, X-481

MONTH July, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	<u>0</u>
2.	<u>0</u>
3.	<u>0</u>
4.	<u>0</u>
5.	<u>0</u>
6.	<u>0</u>
7.	<u>0</u>
8.	<u>0</u>
9.	<u>0</u>
10.	<u>0</u>
11.	<u>0</u>
12.	<u>0</u>
13.	<u>0</u>
14.	<u>0</u>
15.	<u>0</u>
16.	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17.	<u>0</u>
18.	<u>0</u>
19.	<u>0</u>
20.	<u>0</u>
21.	<u>0</u>
22.	<u>0</u>
23.	<u>0</u>
24.	<u>0</u>
25.	<u>0</u>
26.	<u>0</u>
27.	<u>0</u>
28.	<u>0</u>
29.	<u>0</u>
30.	<u>0</u>
31.	<u>0</u>

LIP-300-1
Revision 2
November 13, 1979
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ATTACHMENT B
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle 1

DATE 8/9/82

COMPLETED BY John Ullrich
ext.
TELEPHONE 815/357-6701, 481

REPORT MONTH July, 1982

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF	
					SHUTTING DOWN THE REACTOR OR	CORRECTIVE ACTIONS/COMMENTS

NONE

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

<u>Date</u>	<u>Valves Actuated</u>	<u>No. & Type Actuations</u>	<u>Plant Conditions</u>	<u>Description of Events</u>
7/21/82	1B21-F013C	1 Manual	150 psig	LOS-MS-R2
7/21/82	1B21-F013D	1 Manual	150 psig	LOS-MS-R2
7/21/82	1B21-F013E	1 Manual	150 psig	LOS-MS-R2
7/21/82	1B21-F013R	1 Manual	150 psig	LOS-MS-R2
7/21/82	1B21-F013S	1 Manual	150 psig	LOS-MS-R2
7/21/82	1B21-F013U	2 Manual	150 psig	LOS-MS-R2
7/22/82	1B21-F013P	2 Manual	230 psig	PT-MS-101C
7/22/82	1B21-F013A	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013F	2 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013H	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013B	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013G	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013J	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013S	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013L	1 Manual	240 psig	PT-MS-101C

VI. UNIQUE REPORTING REQUIREMENTS (CONTINUED)

<u>Date</u>	<u>Valves Actuated</u>	<u>No. & Type Actuations</u>	<u>Plant Conditions</u>	<u>Description of Events</u>
7/22/82	1B21-F013R	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013V	3 Manual	240 psig	FT-MS-101C
7/22/82	1B21-F013C	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013D	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013N	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013M	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013U	3 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013K	1 Manual	240 psig	PT-MS-101C
7/22/82	1B21-F013E	3 Manual	240 psig	PT-MS-101C
7/23/82	1B21-F013U	2 Manual	140 psig	Post Maintenance
7/23/82	1B21-F013H	1 Manual	140 psig	Post Maintenance
7/23/82	1B21-F013K	2 Manual	230 psig	Post Maintenance
7/23/82	1B21-F013P	1 Manual	230 psig	Post Maintenance
7/23/82	1B21-F013H	2 Manual	220 psig	Post Maintenance
7/23/82	1B21-F013H	1 Manual	230 psig	Post Maintenance
7/23/82	1B21-F013C	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013E	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013R	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013U	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013V	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013D	5 Manual	230 psig	PT-MS-101C
7/23/82	1B21-F013S	5 Manual	230 psig	PT-MS-101C
7/26/82	1B21-F013V	1 Manual	940 psig	STP-26

VI. UNIQUE REPORTING REQUIREMENTS (CONTINUED)

B. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period:

<u>Outage No.</u>	<u>Equipment</u>	<u>Purpose of Outage</u>
1-1321-82	SRV 1B21-F013U	Repair 'U' SRV position indication
1-1324-82	SRV 1B21-F013H	Repair 'H' SRV position indication

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.