### MONTHLY OPERATIONS SUMMARY

#### JULY 1982

At the beginning of the report period, the Oyster Creek Nuclear Generating Station was operating at 502 MWe with power limited by core reactivity.

On July 7 and again on July 8, a Reactor shutdown was commenced due to a loss of the Stack Gas Sample System. Each shutdown was terminated, within minutes, upon restoration of the Stack Gas Sample System. The loss of the system was attributed to the pump motor tripping on thermal overload. Consequently, a higher rated thermal overload was installed.

On July 9 and July 27, load was temporarily reduced approximately 25 MWe for brush replacement on "C" and "E" Reactor Recirculation Pump MG Sets (respectively).

Containment Spray System II was unavailable for use for a two (2) day period because of a problem in the auto start feature which was due to faulty time delay relays. In addition, the backup position indicator (thermocouple) on Safety Valve NR28J was declared inoperable.

At the end of the report period, the Station was operating at approximately 437 MWe.

The following events were identified as potential Reportable Occurrences:

On July 1, 1982, during the performance of the Thermocouple Valve Monitoring System monthly channel check, it was determined that secondary lift monitor for Safety Valve NR28J was inoperable.

On July 7, 1982, the input cable for the primary lift monitor (acoustical monitor) from Reactor Safety Valve NR28D was found disconnected.

On July 7 and July 8, 1982, the Stack Gas Sample Pump tripped due to thermal overload.

On July 16, 1982, during normal routine surveillance testing, it was discovered that Containment Spray System II would not auto start due to failure of Time Delay Relay 16K14B (Containment Spray Pump "51C").

On July 17, 1982, it was discovered that Containment Spray System II would not auto start due to failure of Time Delay Relay 16K11B (Emergency Service Water Pump 52C).

On July 19, 1982, a procedural error was identified which removed the capability to automatically isolate an Isolation Condenser during a surveillance test.

## AVERAGE DAILY POWER LEVEL

NET MWe

Docket #			50-219
Unit			O. C. #1
Report Date .			August 10, 1982
Compiled by .			J. B. Sklar
Telephone .		•	(609) 971-4613

MONTH July 1982

DAY	MW	DAY	MW
1	476	17	429
2	474	18	424
3	472	19	424
4	471	20	423
5	466	21	424
6	461	22	425
7	457	23	424
8	454	24	422
9	454	25	421
10	441	26	413
11	442	27	412
12	441	28	409
13	440	29	410
14	440	30	411
15	435	31	409
16	431		

### OPERATING DATA REPORT

OPERATING STATUS

UNIT NAME ... OYSTER CREEK DOCKET NUMBER ... 50-219 UTILITY DATA PREPARED BY ... J. B. SKLAR (609) 971-6013 REPORTING PERIOD ... JULY 1982 LICENSED THERMAL POWER (MWT) ... 1930 NAMEPLATE RATING (GROSS MWE) ... 650 DESIGN ELECTRICAL RATING (NET MWE) ... 650 MAXIMUM DEPENDABLE CAPACITY (GROSS MWE) ... 650 MAXIMUM DEPENDABLE CAPACITY (NET MWE) ... 620 IF CHANGES OCCUR IN CAPACITY RATING SINCE LAST REPORT, GIVE REASON ... NONE POWER LPVEL TO WHICH RESTRICTED, IF ANY (NET MWE) ... 400 REASON FOR RESTRICTION, IF ANY ... FUEL DEPLETION

	MONTH	YEAR	CUMULATIVE
HOURS IN PERIOD	744.0	5087.0	110495.0
HOURS FX CRITICAL	744.0	2487.2	80463.6
RX RESERVE SHUTDOWN HRS.	0.0	0.0	468.2
HRS. GEN ON LINE	744.0	2399.1	78609.1
UT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
GROSS THERMAL ENERGY	1074000.0	3342100.0	131933130.5
GROSS ELEC ENERGY	340570.0	1065870.0	44751845.0
NET ELEC ENERGY	324830.0	1015800.0	43083238.0
UT SERVICE FACTOR	100.0	47.2	71.1
UT AVAILABILITY FACTOR	100.0	47.2	71.1
UT CAPACITY FACTOR MDC	70.4	32.2	64.1
UT CAPACITY FACTOR DER	67.2	30.7	60.0
FORCED OUTAGE FACTOR	0.0	52.8	11.5

THE NEXT SCHEDULED OUTAGE IS TO BEGIN ON JANUARY 15, 1983

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219 UNIT NAME Oyster Creek DATE B-2-82 COMPLETED BY R. Baran TELEPHONE 971-4640

## REPORT MONTH July 1982

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason 2	Method of Shutting Down Reactor3	Licensee Event Report #	System Code <sup>4</sup>	Component Cude <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
25	6/4/82 There v reducti	F ere n lons c	38 o unit uring t	H shutdo he mor	3 wns or th of J	NA reportable po uly.	ZZ	ZZZZZ	Turbine trip greater than 40% power. Turbine trip caused by high Reactor water level when feedwater pumps went to runout while filling the Reactor Cleanup System.

# July SUMMARY OF QASL Mechanical MAINTENANCE

EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
Fire Protection Valves V-9-299 and V-9-301	Packing leaks	Adjusted packing.
1-2 Condensate Transfer Pump	Broken nipple on discharge line	Welded new nipple in line.
CRD System Valves NC-30 A and B	Leaking through	Installed new ball and cage in Valves NC-30 A and NC-30 B
Condensate Transfer Valve V-11-17 Make-Up to Fuel Pool	Leaking through	Installed new stem, bushing, and diaphram in valve.
Old Radwaste Temporary Demineralizer Post Filters	Plugged	Installed new filters
1-2 Condensate Transfer Pump	Pump bearing worn	Rebuilt pump

July SUMMARY OF OASL Instrument MAINTENANCE

EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
Drywell Humidity Recorder	Printing pens jammed	Repaired printing pens and checked for proper operation
Shutdown cooling room temperature indicator (1B06-H)	Erroneous temperature reading	Replaced resistance temperature detector and checked for satisfactory operation
Reactor Building Area Radiation Monitor (C7)	Spiking, causing spurious alarms	Replaced detector and satisfactorily performed calibration procedure. Returned to service.
Stack Gas Monitoring System	Built-in check source inopera- tive	Replaced blown fuse. Tested satis- factorily.
Fuel Pool Flow Controller	Flow controller inoperable	Repaired defective slide wire. Calibrated and returned to service.
Traversing In-core Probe Control Units 1, 2, 4	No "ready"lights	Replaced lamps - tested satisfactorily
Intermediate Range Monitor No. 16	Failed front panel surveillance test	Repaired cold solder joint on am- plifier module - front panel surveillance tested satisfactorily.
Containment Spray System II	Pumps failed to start during auto start surveillance test	Replaced defective relay in auto start logic and performed satisfactory surveil: ance test.
Emergency Service Water Pump (52C) Logic Control Relay	Relay premature; time out of synchronization.	Replaced time delay section of associated relay and performed satis- factory surveillance test.

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# July SUMMARY OF QASL Electrical MAINTENANCE

EQUIPMENT	MALFUNCTION	COSRECTIVE ACTION
Stack Tritium Sampler	Defective furnace	Installed new furnace. Tested satis- factorily.
Reactor Protection System	Fuse 2F16 Open	Fuse internal link pulled out of soldered connection. Fuse failure, not component failure.
Augmented Off Gas System	#2 water removal train blown fuse	Found shorted strip heater. Replaced heater and returned to service.
Augmented Off Gas System	#3 water removal train blown fuse	"ound grounded crank case heater. Replaced heater and returned to service.
DC Invertastat for Public Audress System	DC feed breaker found tripped	Breaker tested satisfactorily. Re- installed breaker and returned to service
Reactor Building Air Lock Door to CRD Room	Faulty door interlock switch	Replaced inside door switch. Door operating satisfactorily.
Reactor Building Northwest Air Lock Door	Door interlock fails inter- mittently	Adjusted air lock solenoids. Doors working satisfactorily.
Drywell Oxygen Sample Valve V-38-10	Valve would not open in auto- matic	Relay 6K37 not working. Replaced relay. Valve working satisfactorily.

Oyster Creek Station Docket No. 50-219

### REFUELING INFORMATION -

Name of Facility: Oyster Creek Station #1 Scheduled date for next refueling shutdown: January 15, 1983 Scheduled date for restart following refueling: late - 1983

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

A Tech Spec Change Request to incorporate G.E. fuel assemblies will be submitted by September 1, 1982

Scheduled date(s) for submitting proposed licensing action and supporting information:

March 9, 1981 - Complete NEDO document #24195 (G.E. Reload Fuel Application for Oyster Creek) was submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

- General Electric fuel assemblies fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
- Exxon Fuel Assemblies No major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core - 560 (b) in the spent fuel storage pool - 781

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Spring 1987 Outage.\*

\*NOTE: This is for a normal refueling. Full core off-load, however can only be accommodated through about 1983 or 1984 with 1800 licensed locations.