NRC FORM 366 (12-81)	LICENCE EVENT REPORT	APPROVED BY GMB 3150-0011 EXPIRES 4-30-82
CONTROL BLOCK:	(PLEASE PRINT OR TYPE AL	L REQUIRED INFORMATION)
0 1 P A S E S 1 2 0 0	0 - 0 0 0 0 0 - 0 0 3 4 1	1 1 1 1 3 4 57 CAT 58
CON'T O 1 REPORT L 6 0 5 0 0 0 3 8 7 7 0 1 0 6 8 3 8 0 2 0 4 8 3 9 TO 1 SOURCE 60 61 DOCKET NUMBER 65 69 EVENT DATE 74 75 REPORT DATE 80		
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (1) O 2 During the Startup Testing Program, the unidentified Reactor Coolant System		
0 3 leakage exceeded 5 gpm. This is reportable per Technical Specification		
0 4 6.9.1.9.b. The action statement was entered, however, the reactor was		
0 5 scrammed as part of the startup test program prior to the action statement		
0 6 time limit becoming a factor.		
0 7		
08		30
SYSTEM CAUSE CODE CODE CODE 11	SUBCODE COMPONENT CODE SUB	PORT VALVE
17 REPORT EVENT YEAR	NEPORT NO. CODE TO 3 24 26 27 26 29 31	L _ 0
ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT NPRO-4 PRIME GCMF. COMPONENT (26) ACTION ON PLANT METHOD HOURS 22 SUBMITTED FORM SUB. SUPPLIER MANUFACTURER 26) A 18 X 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 A 25 A 3 9 1 A 3 A 40 A1 A2 A3 A4 A4 A7		
	It of a valve packing failure. The	
apparently reached its useful service limit. The valve was repacked and		
returned to service. Other similiar valves which have not been repacked		
are scheduled for repa	acking.	
1 4 7 8 9	METHOD OF	80
TACIL TY STATUS SPOWER OTHE	ER STATUS (30) DISCOVERY DISCOV	
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF AC	14 45 46 LOCATIO	ON OF RELEASE 36
7 8 9 10 11 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION	44 45	
17 0 0 0 0 37 Z 38 L	n/a	**
PERSONNEL INJURIES NUMBER DESCRIPTION 41	n/a	
1 8 9 11 12 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 1 9 7 (42)	8302230697_830203	•0
PUBLICITY	PDR ADOCK 0500038	7
2 0 N 44	n/a	68 69 80
NAME OF PREPARER	G. Mitchell PHON	E:_(717) 542-2181 X3524

Attachment

Licensea Event Report 83-005/03L-0

During the Startup Testing Program, unidentified Reactor Coolant System leakage exceeded 5 gpm. This is reportable per 6.9.1.9.b. There were no adverse consequences in that the action statement 3.4.3.2.b was entered. The reactor was scrammed as part of a startup test prior to the action statement time limit expiring.

Upon entry into the drywell, it was determined that the leakage was the result of a packing failure on valve HV-144-F001. This valve is an inboard isolation valve on the Reactor Water Cleanup System. The packing failure was attributed to normal wear. A review of the maintenance history showed this valve had not been repacked since initial system turnover and the packing showed signs of wear.

The valve was repacked and returned to service. The remaining Local Leak Rate Test boundary valves which were not repacked as part of the maintenance valve packing program have had the necessary paperwork initiated to repack them in conjunction with the LLRT package.

This valve will be noted by the mechanical maintenance section program, should the new packing fail in an abnormally short period, and the situation will be evaluated. The valve is a six inch, Anchor-Darling gate valve.