

DCS MS-016

AUG 27 1982

Docket No. 50-309

MEMORANDUM FOR: Robert A. Clark, Chief  
 Operating Reactors Branch #3  
 Division of Licensing

FROM: Chris C. Nelson, Project Manager  
 Operating Reactors Branch #3  
 Division of Licensing

SUBJECT: SUMMARY OF MEETING WITH YANKEE ATOMIC ELECTRIC  
 COMPANY

Attendees at this meeting are listed in Enclosure 1.

Enclosure 2 summarizes presentations made by the Yankee Atomic Electric Company to the NRC staff.

Specific comments made by the NRC staff in reply to the YAEC presentation are as follows:

1. Better documentation should be made on the existing and approved models from which RELAP5YA was developed. Charts summarizing these changes from earlier models would help the NRC reviewers.
2. Indicate how RELAP5YA addresses the specific staff concerns summarized in the Bulletins and Orders Task Force Reports.
3. Emphasize comparisons with experimental data.
4. Develop a summary indicating in detail how RELAP5YA will be used. This should include a tabulation of what ancillary codes are used as part of these calculations. Indicate which ancillary codes have received NRC review and approval.
5. Relevant sample calculations should be submitted as soon as practical indicating how the codes would be used in the specific licensing actions discussed in item 4.

B209230003 B20827  
 PDR ADOCK 05000309  
 P PDR

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

- 6. All submittals should be extremely clear about the specific licensing actions and related schedules for which NRC review and approval is requested. Careful consideration should be given to critical path scheduling for reviews affecting multiple licensing actions for several plants.

Original signed by  
*Robert A. Clark* for  
Chris C. Nelson, Project Manager  
Operating Reactors Branch #3  
Division of Licensing

cc: See next page

OFFICE	ORB #3	ORB #3 <i>RA</i>	ORB #3	ORB #3		
SURNAME	Kreutzer	Heitner/tcm	Nelson	RAClark		
DATE	8/26/82	8/20/82	8/21/82	8/21/82		

MEETING SUMMARY DISTRIBUTION

Licensee: Maine Yankee Atomic Power Company

\*Copies also sent to those people on service (cc) list for subject plant(s).

✓ Docket File  
NRC PDR  
L PDR  
NSIC  
TERA  
ORB#3 Rdg  
JHeltemes  
BGrimes  
RAClark  
Project Manager  
PMKreutzer  
OELD  
I&E  
ACRS-10  
ORB#3 Summary File  
NRC Participants

Maine Yankee Atomic Power Company

cc: E. W. Thurlow, President  
Maine Yankee Atomic Power Company  
Edison Drive  
Augusta, Maine 04336

Mr. Donald E. Vandenburg  
Vice President - Engineering  
Yankee Atomic Electric Company  
20 Turnpike Road  
Westboro, Massachusetts 01581

John A. Ritsher, Esq.  
Ropes & Gray  
225 Franklin Street  
Boston, Massachusetts 02110

Wiscasset Public Library  
Association  
High Street  
Wiscasset, Maine 04578

Mr. E. C. Wood, Plant Manager  
Maine Yankee Atomic Power Company  
P. O. Box 3270  
Wiscasset, Maine 04578

Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

First Selectman of Wiscasset  
Municipal Building  
U. S. Route 1  
Wiscasset, Maine 04578

Mr. Paul Swetland  
Resident Inspector  
c/o U. S. Nuclear Regulatory  
Commission  
P. O. Box E  
Wiscasset, Maine 04578

Mr. Charles B. Brinkman  
Manager - Washington Nuclear Operations  
Combustion Engineering, Inc.  
4853 Cordell Avenue, Suite A-1  
Bethesda, Maryland 20014

Mr. Robert H. Groce  
Senior Engineer - Licensing  
Maine Yankee Atomic Power Company  
1671 Worcester Road  
Framingham, Massachusetts 01701

U. S. Environmental Protection Agency  
Region I Office  
ATTN: Reg. Radiation Representative  
JFK Federal Building  
Boston, Massachusetts 02203

State Planning Officer  
Executive Department  
189 State Street  
Augusta, Maine 04330



ATTENDANCE MEETING FOR  
MAINE YANKEE MEETING WITH NRC

August 19, 1982

<u>NAME</u>	<u>ORGANIZATION</u>
C. Nelson	NRC-PM
W. Lyon	NRR/RSB
Kenneth L. Heitner	NRC-PM
Edward D. Throm	NRC/RSB
Jack Guttman	NRC/RSB
Gordon Willcutt	LANL
Tom Fernandez	YAEC
Bob Groce	YAEC
Ramu Sundaram	YAEC
Jamal Ghaus	YAEC
Bob Harvey	YAEC
Jim Loomis	YAEC
Ming-Shig Lu	BNL
Melvin Levine	BNL
N. Lauben	NRR/RSB
A. Husain	YAEC

# YANKEE ATOMIC ELECTRIC COMPANY



YANKEE ATOMIC ELECTRIC COMPANY  
RELAP5YA  
PRESENTATION TO NRC  
AUGUST 19, 1982

YNSD Participants:

A. Husain  
R. T. Fernandez  
R. K. Sundaram  
R. C. Harvey  
J. M. Ghaus  
J. N. Loomis

MEETING OBJECTIVES

1. To Inform NRC of the TECHNICAL CONTENTS of RELAP5YA  
Methods Development & Assessment Project.

Submittal slated for Oct. 15, 1982.

2. To Elicit NRC comments on the tech. contents.
3. To obtain NRC Review schedule and a preliminary  
estimate on the date of SER issuance.

## LOCA METHODS DEVELOPMENT & ASSESSMENT PROGRAM

### GOALS

- o DEVELOP BWR LOCA ANALYSIS CAPABILITY AT YAEC  
TO SUPPORT VERMONT YANKEE
  - . COMPLETE BREAK SIZE AND LOCATION SPECTRA
  - . BEST ESTIMATE (BE) AND EVALUATION MODEL  
(EM) CAPABILITY
  
- o UPGRADE PWR Small Break Methods to Support  
Yankee, Maine Yankee and Seabrook
  - . Meet NUREG-0737 Requirement II.K.3.30

## MAJOR TASKS

- METHODS DEVELOPMENT
  - Enhancement
  - Evaluation Model
  
- METHODS QUALIFICATION
  - Separate Effects
  - Integral Tests
  
- DOCUMENTATION & Quality Assurance
  
- NRC Review & Approval
  
- Sample Problems (BWR & PWR)
  
- Plant Applications

## AGENDA

9:00- 9:10

### INTRODUCTION

Ausaf Husain

9:10-11:30

### MODEL DEVELOPMENT AND ASSESSMENT

9:10-10:30

#### HYDRODYNAMIC MODELS

9:10- 9:40

Interfacial Shear

Ramu K. Sundaram

9:40- 9:55

Two Phase Critical Flow

R. Thomas Fernandez

9:55-10:00

Accumulator

R. Thomas Fernandez

10:00-10:30

Jet Pump

Jamal Ghaus

10:30-11:15

#### HEAT TRANSFER MODELS

10:30-10:35

Forced Convective Boiling

Robert C. Harvey

10:35-10:45

Critical Heat Flux

James N. Loomis

10:45-10:55

Radiation Heat Transfer

R. Thomas Fernandez

10:55-11:05

Rewet and Quench

R. Thomas Fernandez

11:05-11:15

Heat Transfer Lockouts

R. Thomas Fernandez

11:15-11:35

#### FUEL BEHAVIOR MODELS

Assumptions

James N. Loomis

Model Features

James N. Loomis

Benchmarks

James N. Loomis

AGENDA  
-2-

CODE ASSESSMENT AGAINST SYSTEM TESTS

11:35-12:05	<u>THTF</u>	R. Thomas Fernandez
	Steady State Film Boiling	
	Transient Film Boiling	
	Quasi Steady Boiloff	
	Reflood	
12:05-1:00	<u>LUNCH</u>	
1:00-2:30	<u>TLTA</u>	Ramu K. Sundaram
1:00-1:15	Transient Boiloff	
1:15-1:30	Small Break Large Break w/ ECC	
1:30-2:00	Best Estimate	
2:00-2:15	Evaluation Model	
2:15-2:30	Large Break w/o ECC	
2:30-2:45	<u>LOFT</u>	Robert C. Harvey
	Small Break Test L3-6	
2:45-2:50	<u>SUMMARY</u>	Ausaf Husain/ R. Thomas Fernandez
3:00-3:30	<u>DISCUSSION</u>	All
3:30-3:45	<u>PRELIMINARY NRC REVIEW SCHEDULE</u>	NRR



## INTERPHASE DRAG

### BACKGROUND

. RELAP5 USES 2 MOMENTUM EQUATIONS

. INTERFACIAL DRAG FORCE PER UNIT VOLUME

$$F_{I_{gf}} = -A_{gf} B_{gf} (V_g - V_f) = -F_I (V_g - V_f)$$

.  $F_I = A_{gf} B_{gf}$  -- CONSTITUTIVE RELATIONSHIP

. AT LOW FLOWS,

$$V_r = (V_g - V_f) \sim \frac{1}{F_I} \alpha_g (1 - \alpha_g) (\rho_f - \rho_g) \frac{g}{g_c}$$

.  $A_{gf} = A_{gf}(FR, \alpha, V_r)$

$B_{gf} = B_{gf}(FR, \alpha, V_r)$

- $F_I$  CALCULATION NOT EXPLICIT

$$F_I = A_{gf} B_{gf} \sim (V_Y)^m$$

IF  $V_Y = V_Y^n$ , NUMERICAL OSCILLATIONS  
POSSIBLE AT LOW FLOWS

RELAP5YA MODEL FOR  $F_I$

- SIMPLE FLOW REGIME MAP BASED  
ON  $G$  AND  $\alpha$

- AT HIGH FLOWS,  $V_Y = V_Y^n$

- AT LOW FLOWS,  $V_Y \rightarrow$  INDEPENDENT MODEL

3. SLUG FLOW

$$F_I = 0.5 C_D \rho_f |V_r| (\alpha / L_s)$$

$$V_r = V_r(\alpha, D_h) ; G \leq 100 \text{ kg/m}^2\text{-s}$$

$$V_r = V_r^n ; G \geq 150 \text{ kg/m}^2\text{-s}$$

$$L_s = L_s(\alpha, F_{IB}, F_{IA})$$

4. ANNULAR-MIST REGION

$$F_I = (1 - \epsilon) F_{IA} + \epsilon F_{IM}$$

$$\epsilon = \epsilon(\alpha) = \text{ENTRAINMENT}$$

NOTE:  $F_I(\alpha)$  IS A CONTINUOUS FUNCTION

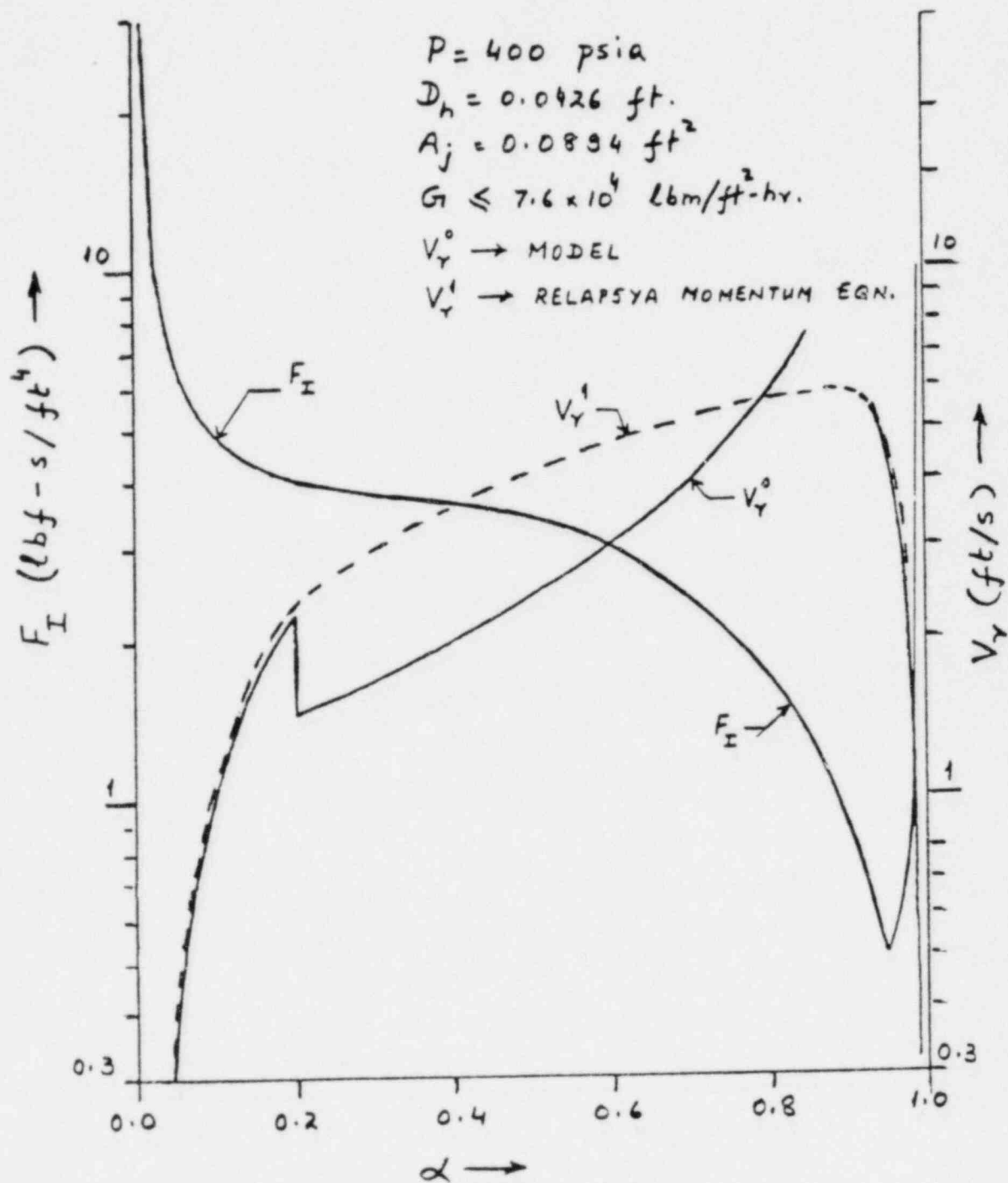


FIG. SAMPLE CALCULATION OF  $V_Y, F_I$

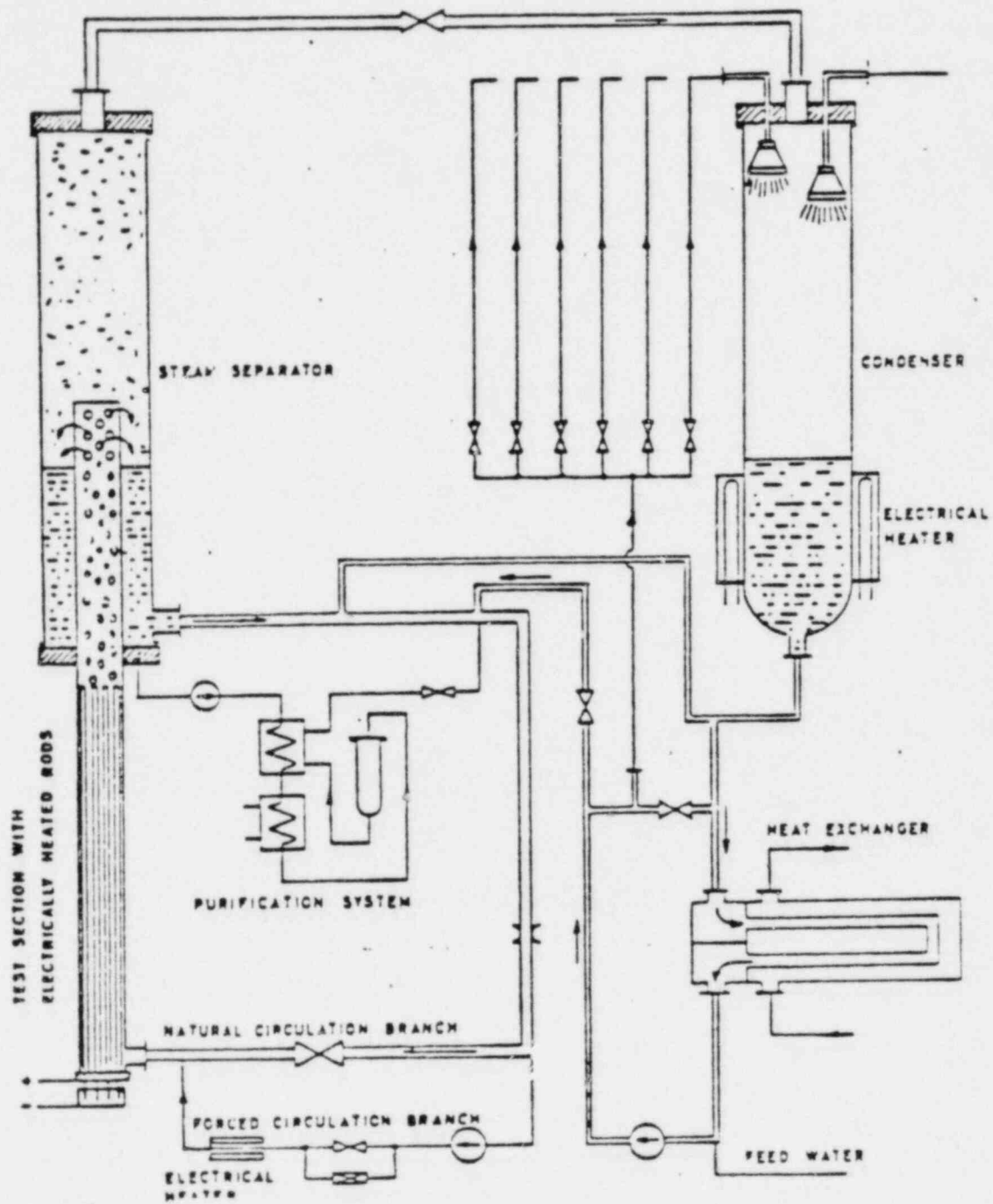
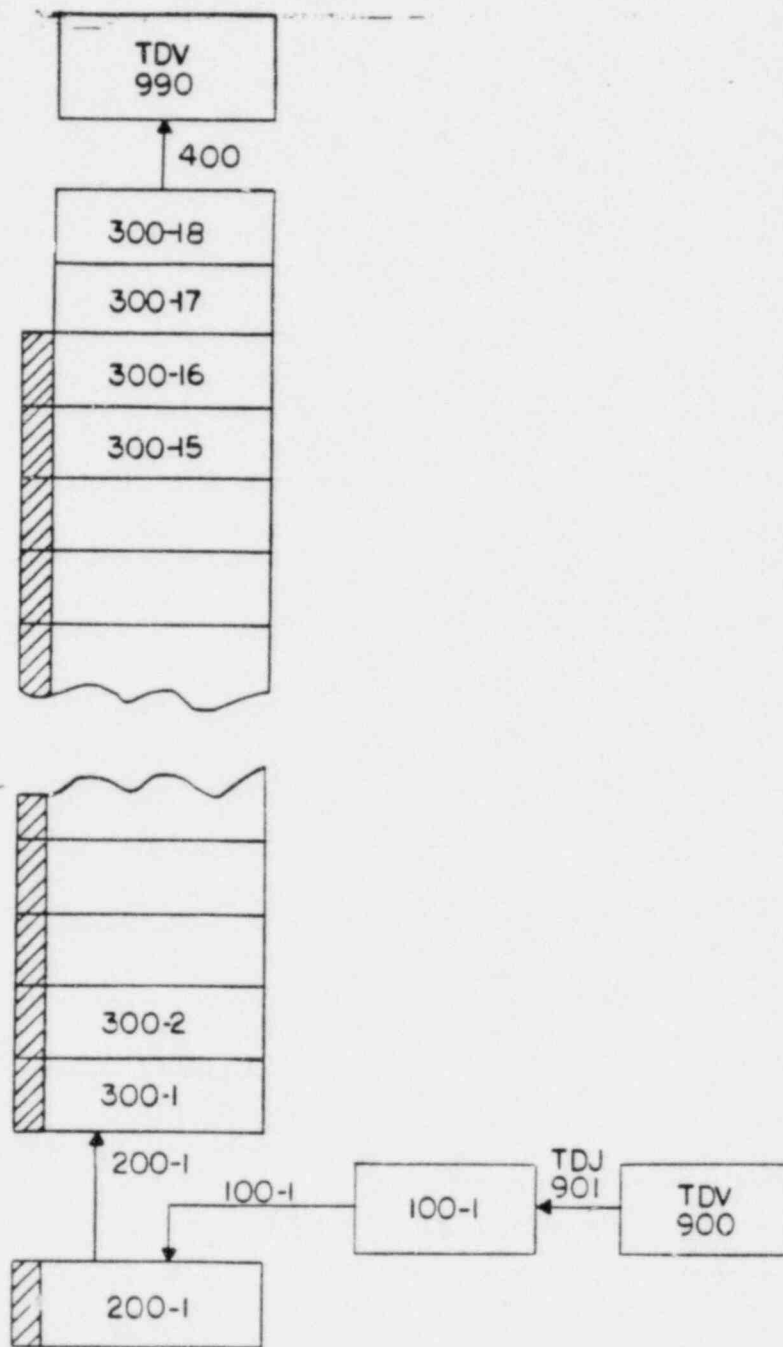
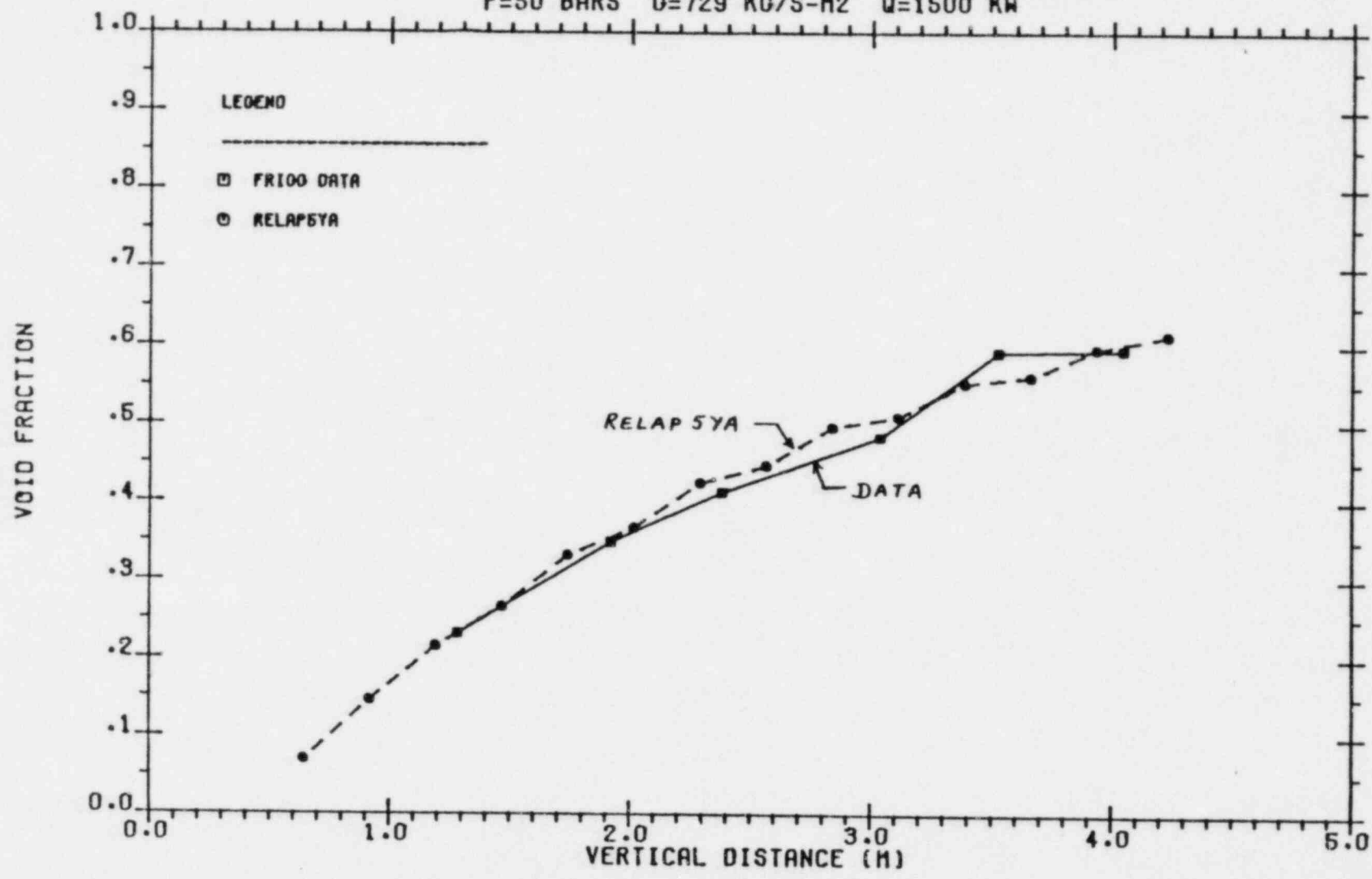


FIG. 1 - SIMPLIFIED FLOW DIAGRAM FOR THE FRIGG LOOP

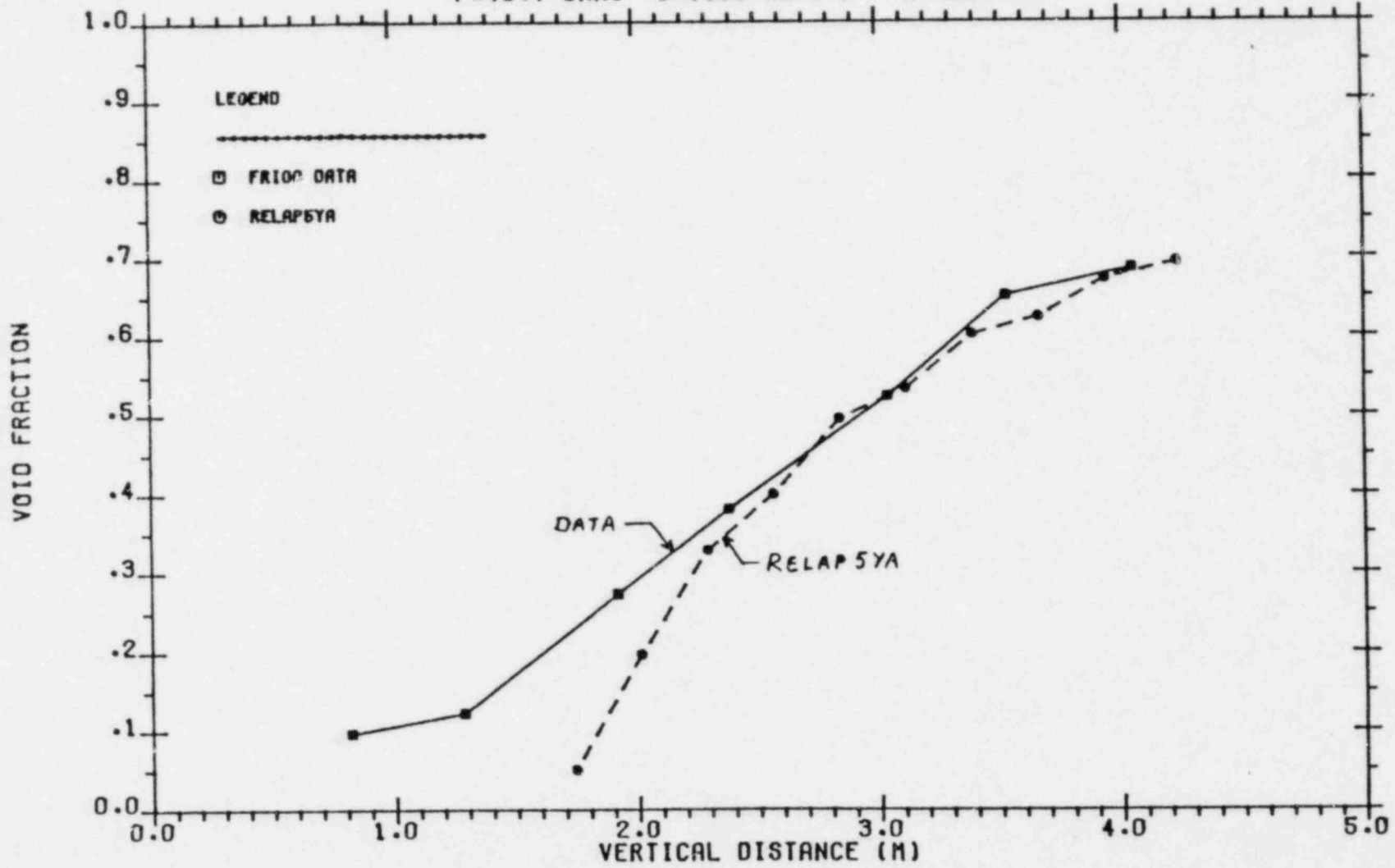


RELAP 5YA MODEL OF FRIGG BUNDLE

FRIGO LOOP TEST 913006 AXIAL VOID DISTRIBUTION  
P=50 BARS  $G=729$  KG/S-M<sup>2</sup>  $Q=1500$  KW

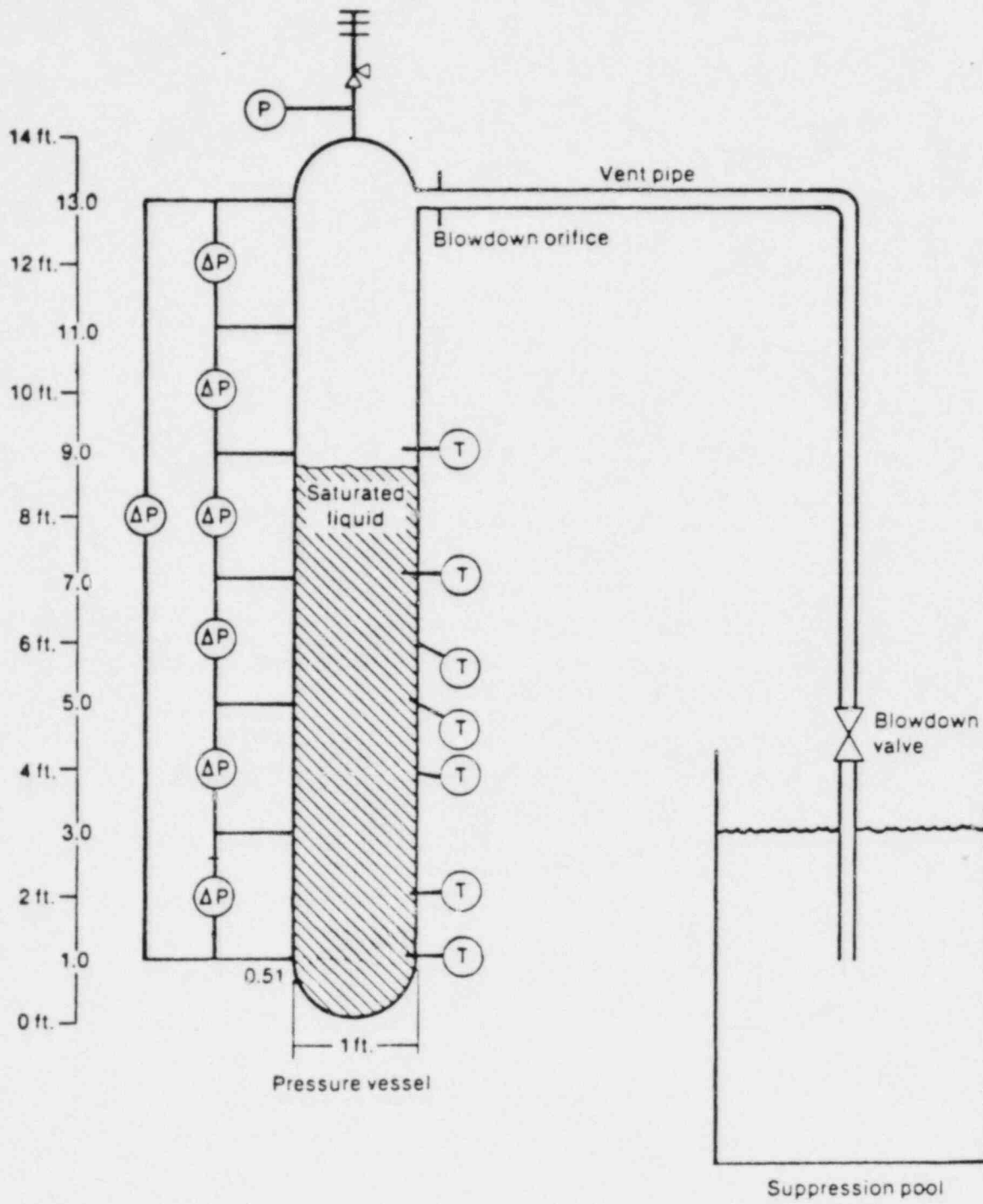


FRIGO LOOP TEST 913020 AXIAL VOID DISTRIBUTION  
P=49.7 BARS Q=1159 KO/S-M2 Q=4415 KW

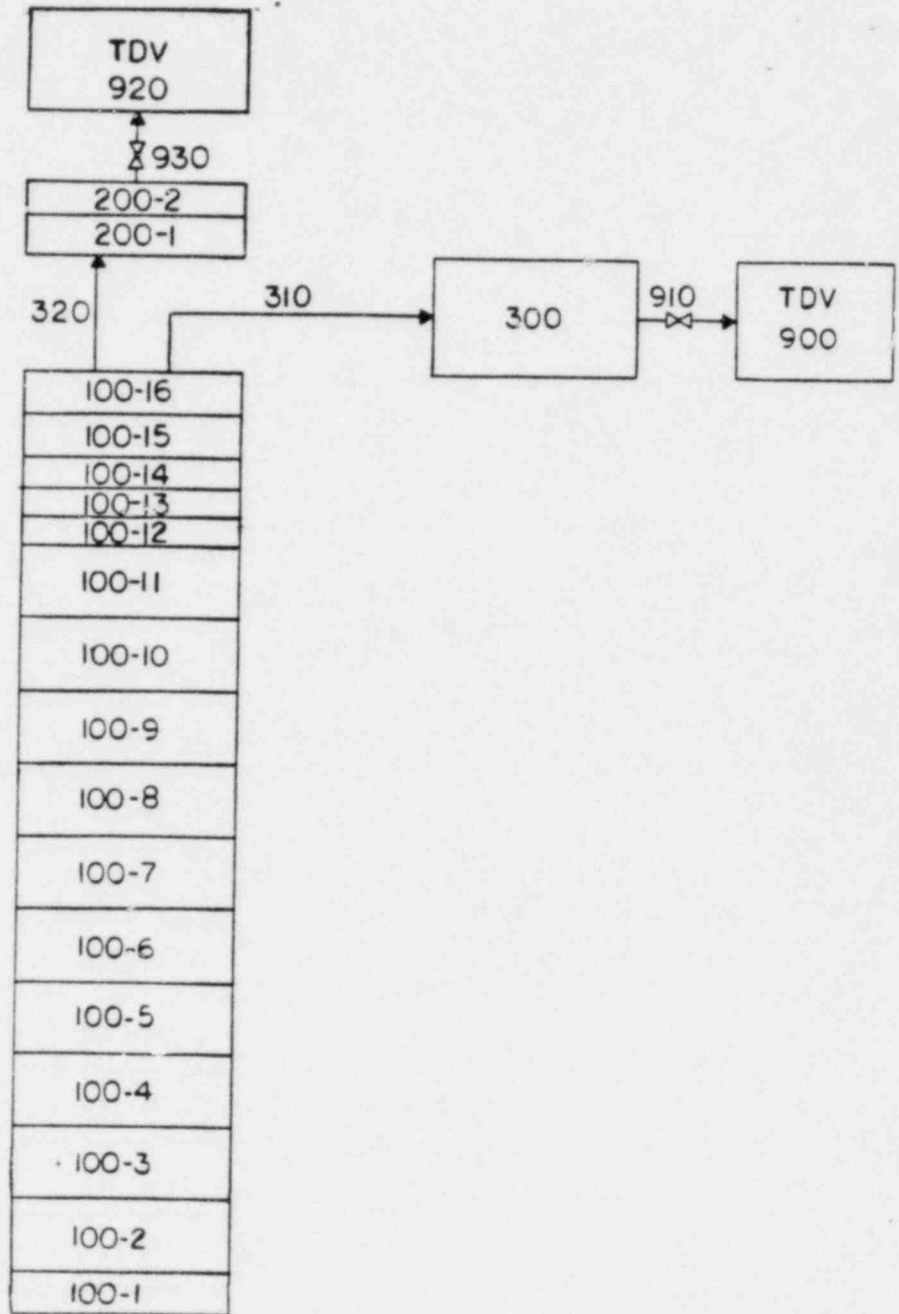




Small blowdown vessel

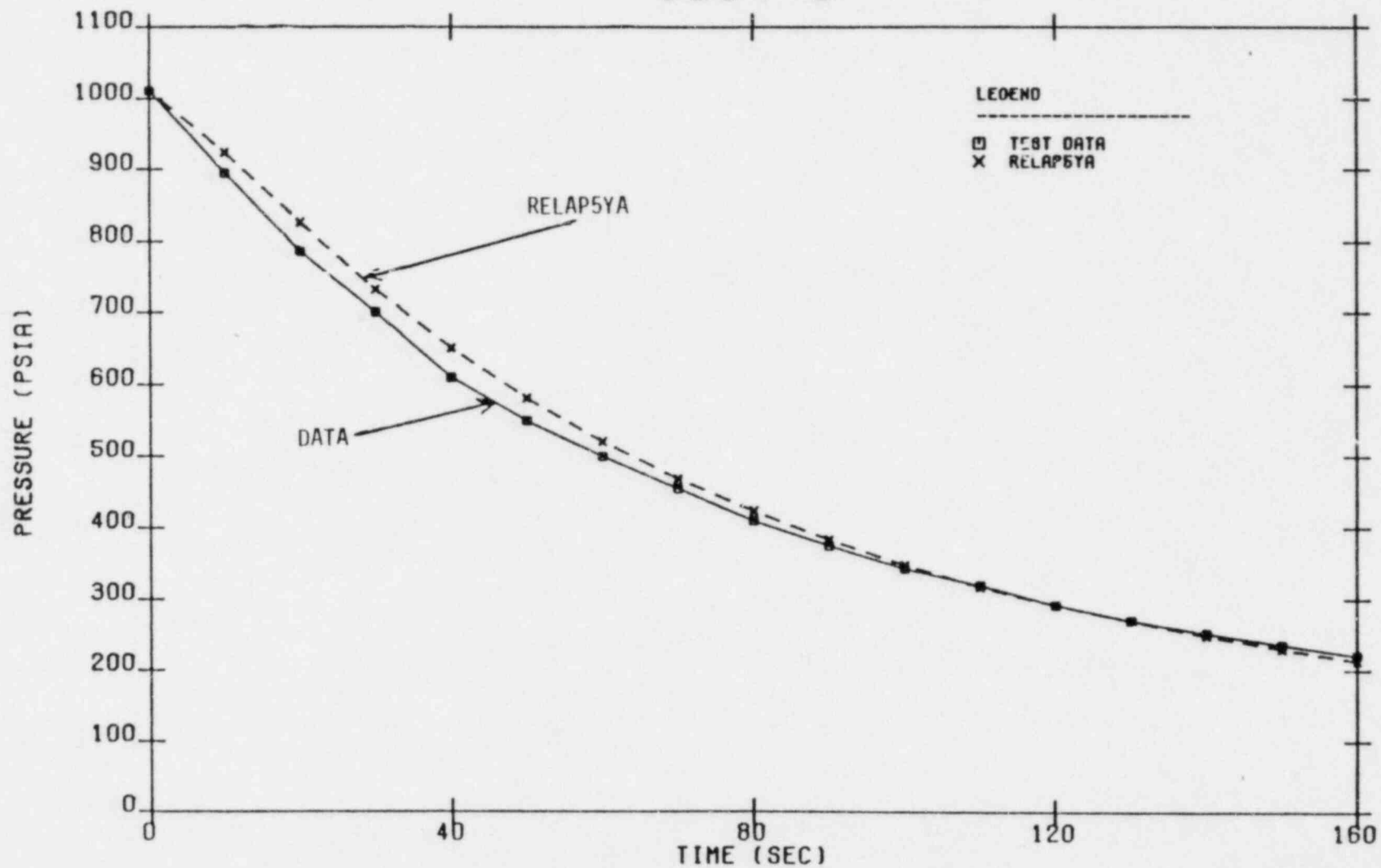


GE Level swell test configuration

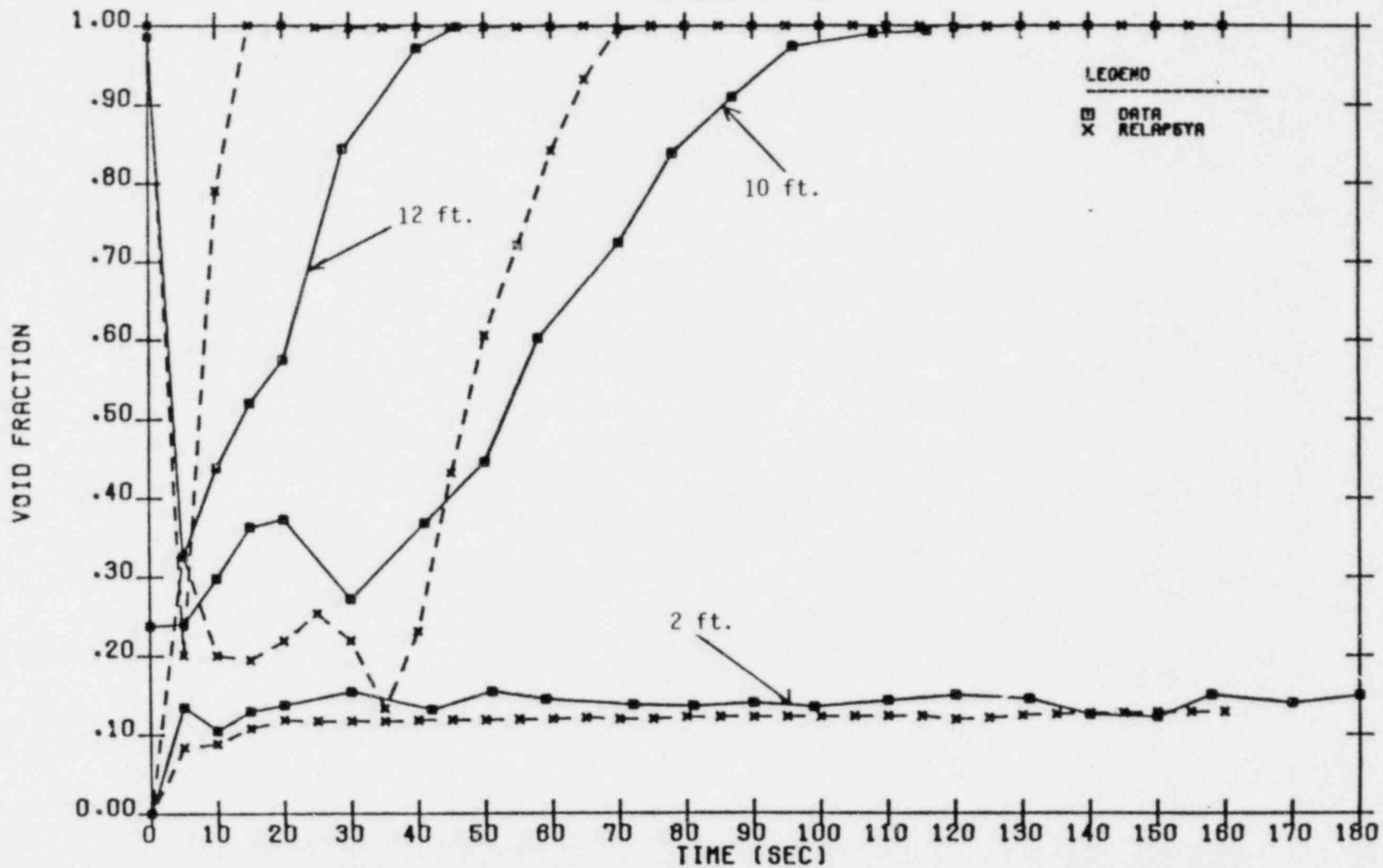


RELAP 5YA MODEL OF GE LEVEL SWELL TEST

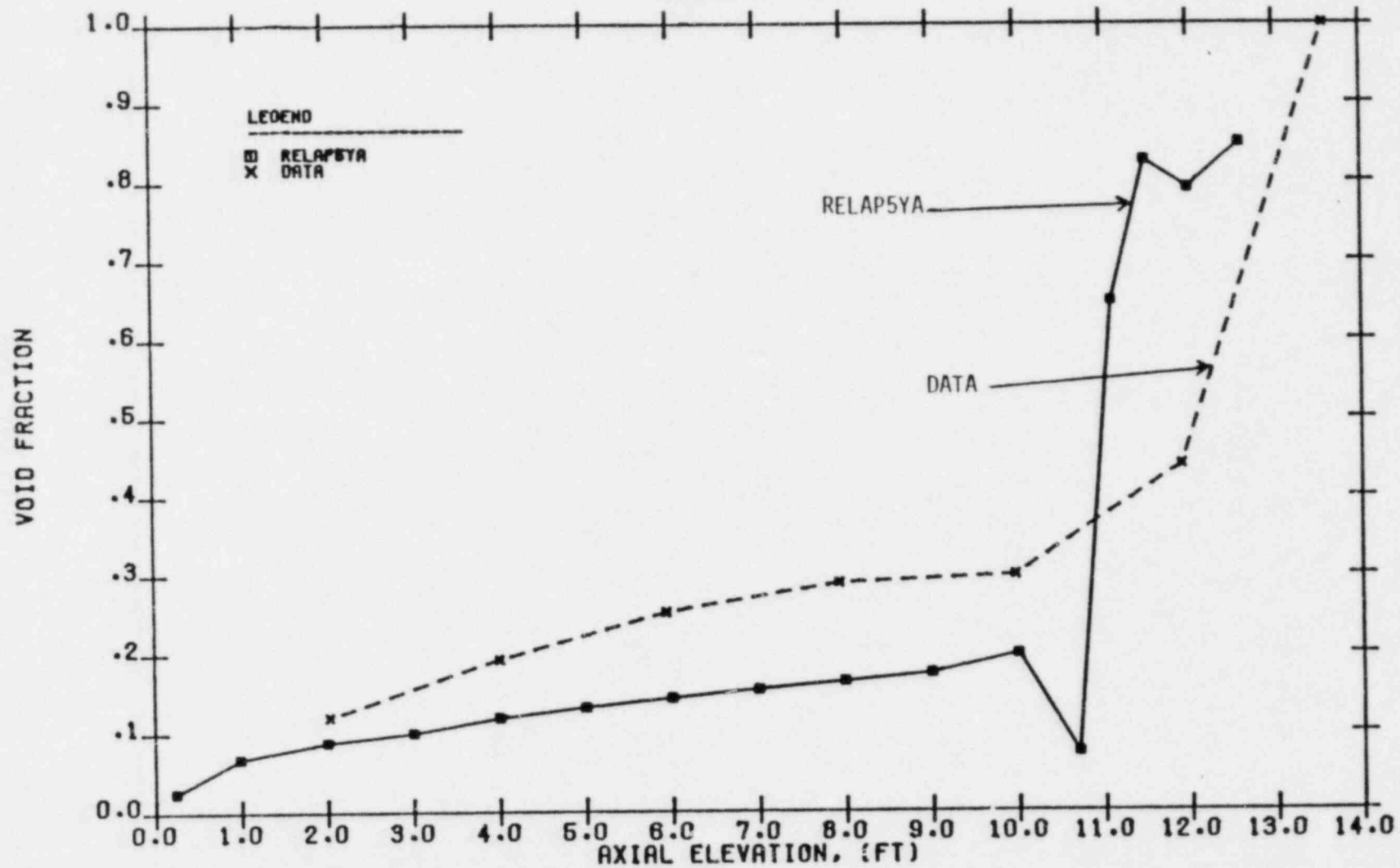
# VESSEL PRESSURE 1004-3



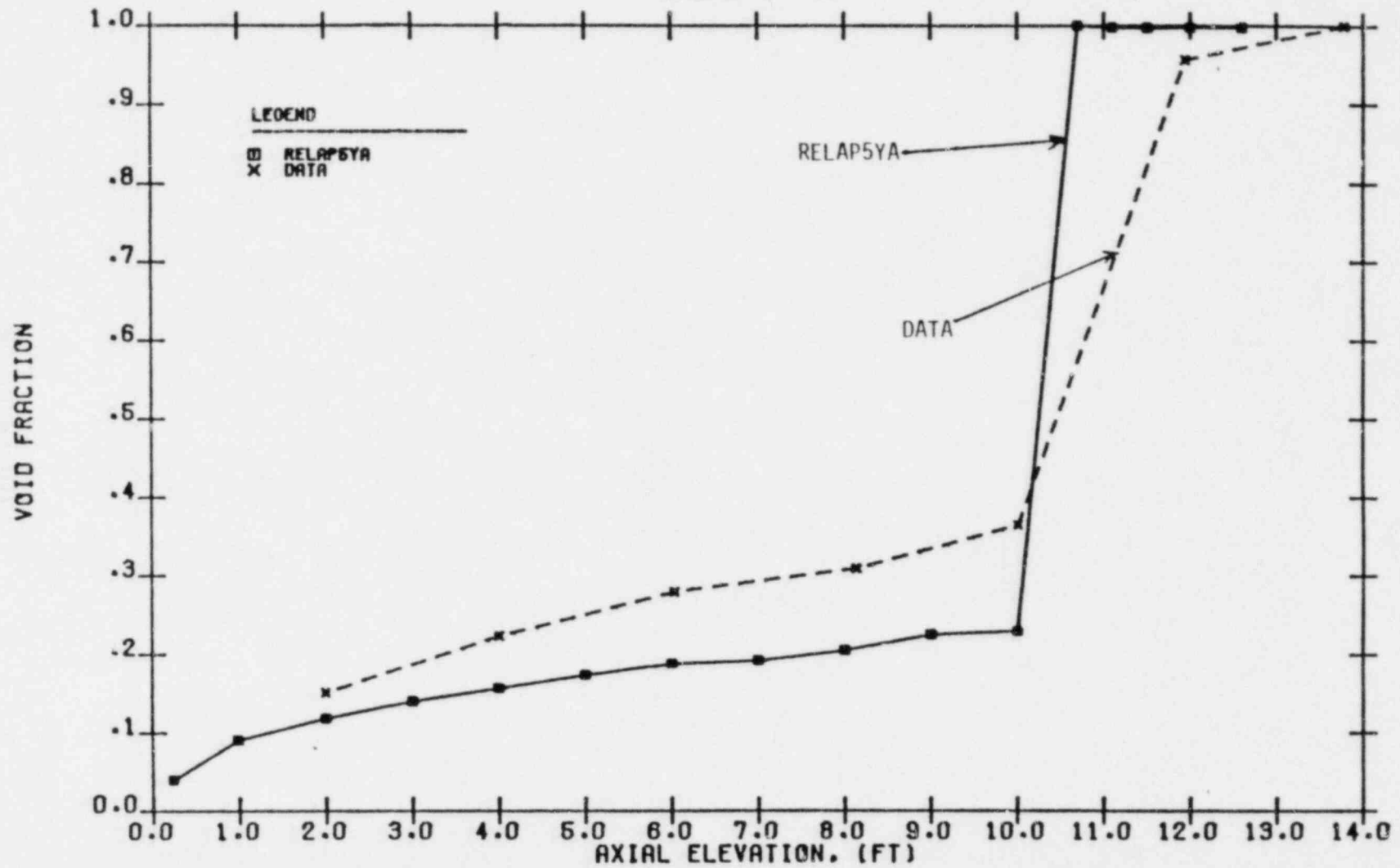
# NODAL VOID FRACTION 1004-3



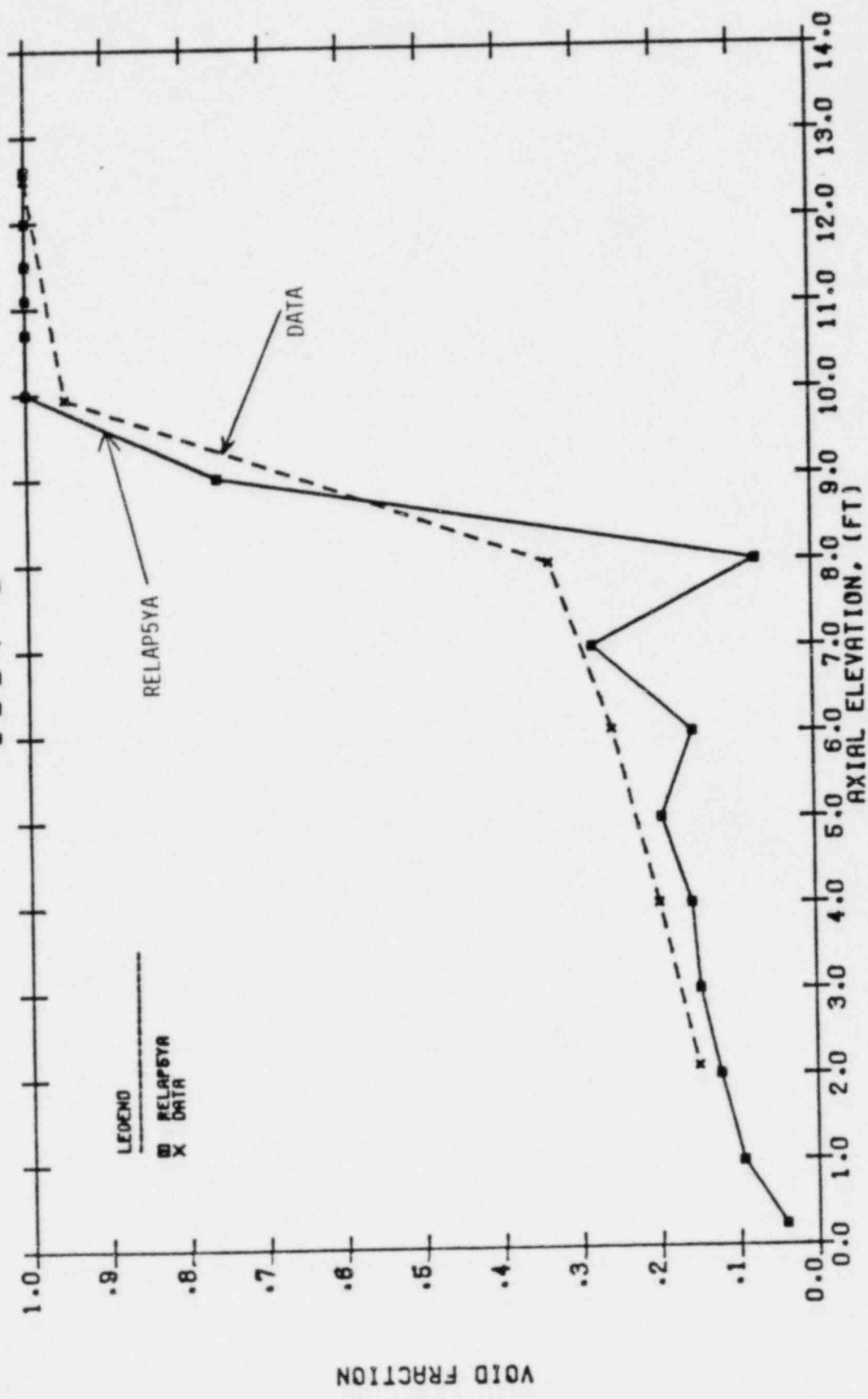
# LEVEL SWELL COMPARISON, T = 10 SEC. 1004-3



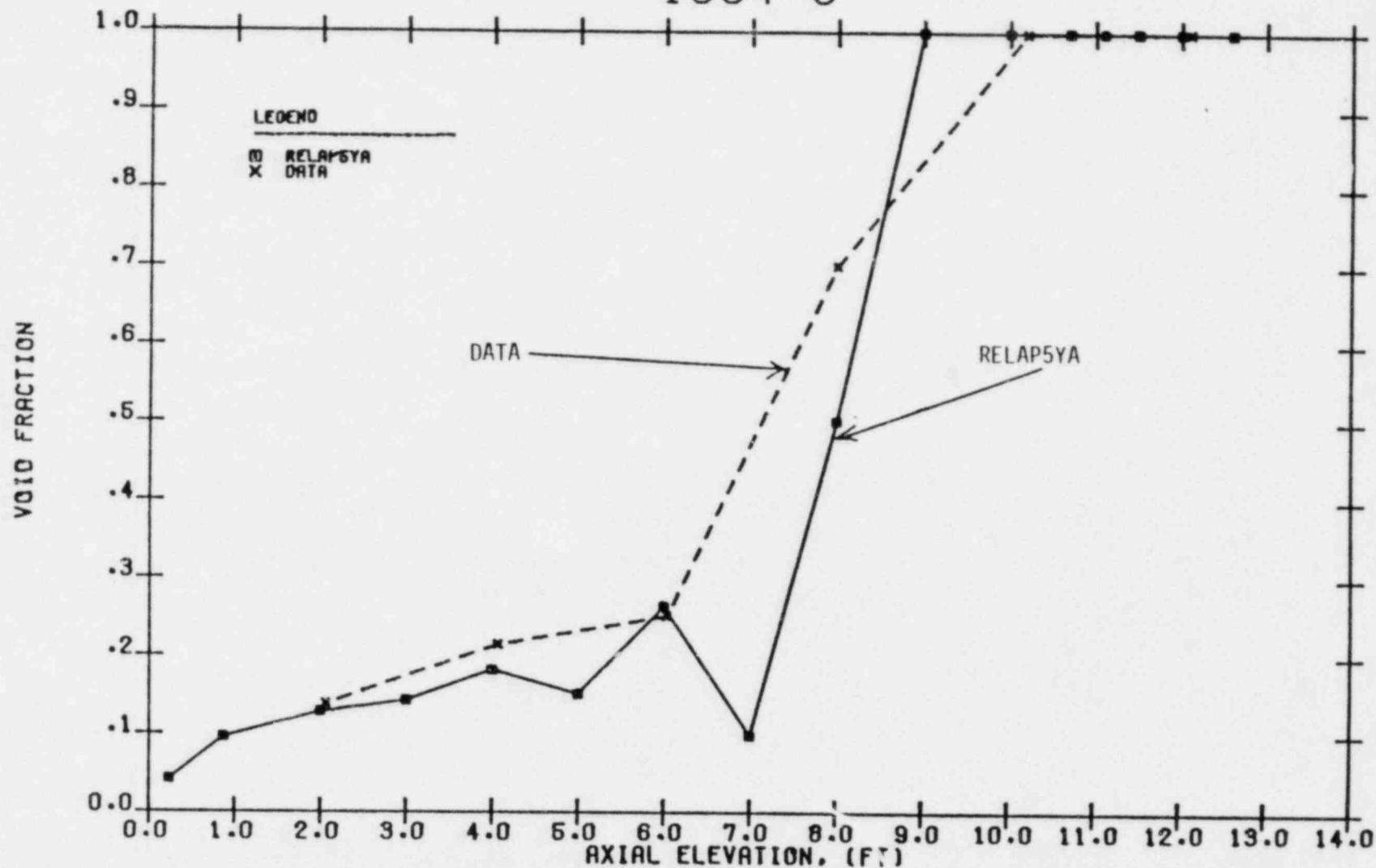
LEVEL SWELL COMPARISON, T = 40 SEC.  
1004-3



LEVEL SWELL COMPARISON, T = 100 SEC.  
1004-3



# LEVEL SWELL COMPARISON, T = 150 SEC. 1004-3





## MOODY TWO PHASE CRITICAL FLOW

- $\alpha_g \geq 0.05$  FOR TWO PHASE CRITICAL FLOW

- $G_{RS} = (\alpha \rho)_g^n V_g^{n+1/2} + (\alpha \rho)_f^n V_f^{n+1/2}$

- $G_{CRIT} = C_D \cdot G_M(P, h)$

- CHOKING CRITERIA

- \* INITIAL :  $G_{RS} \geq C_D \cdot G_M$

- \* UNCHOKING :  $G_{RS} < 0.95 C_D \cdot G_M$   
ONE TIME STEP LAG

- PHASIC VELOCITIES ( $V_g, V_f$ )<sup>n+1</sup>

- \* CONT.  $(\alpha \rho)_g^n V_g^{n+1} + (\alpha_f \rho)_f^{n+1} = G_{CRIT}$

- \* SUM AND DIFF. MOMENTA EQNS.

- FEATURES

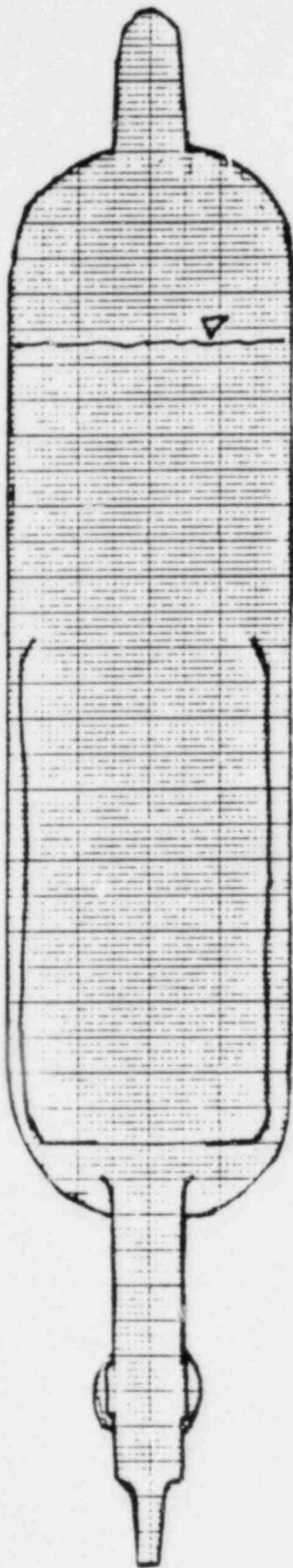
- $C_D$  RETAINED

- $FIJ = 1.0 E 15 \Rightarrow S \approx 1$

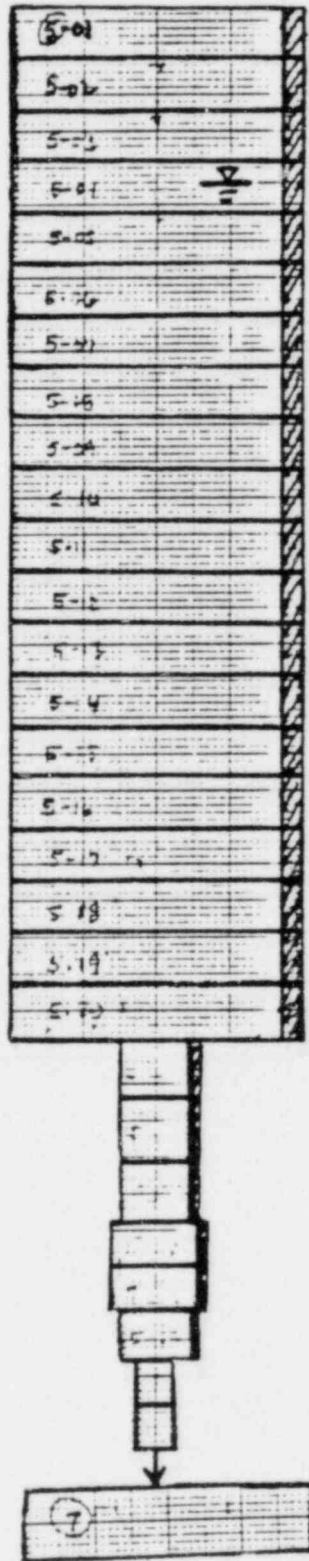
- STATIC ( $P_s, h_s$ ) OR STAGNATION ( $P_0, h_0$ ) OPTION

- MOODY TABLE  $G_M(P_0, h_0)$

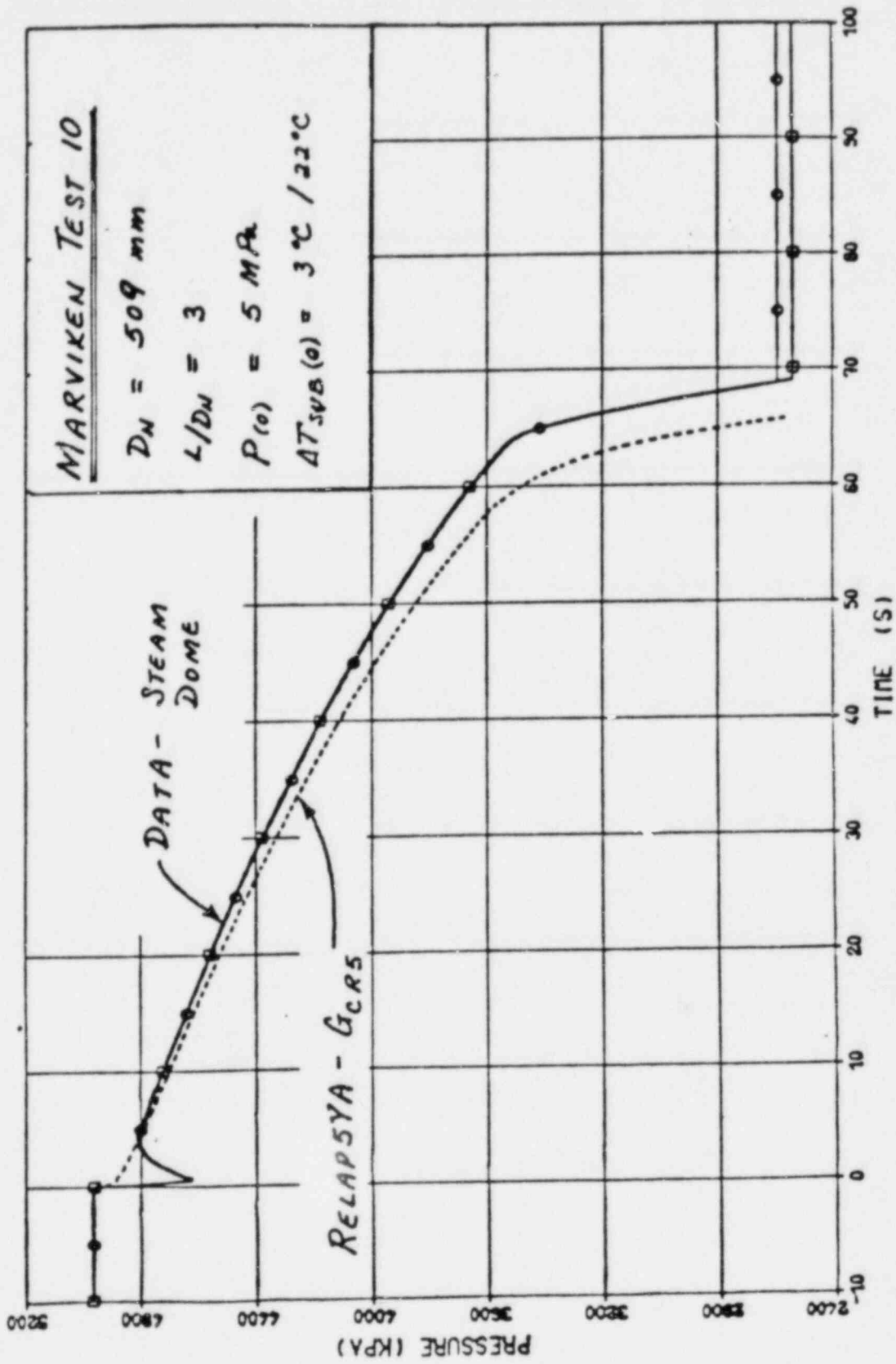
FACILITY



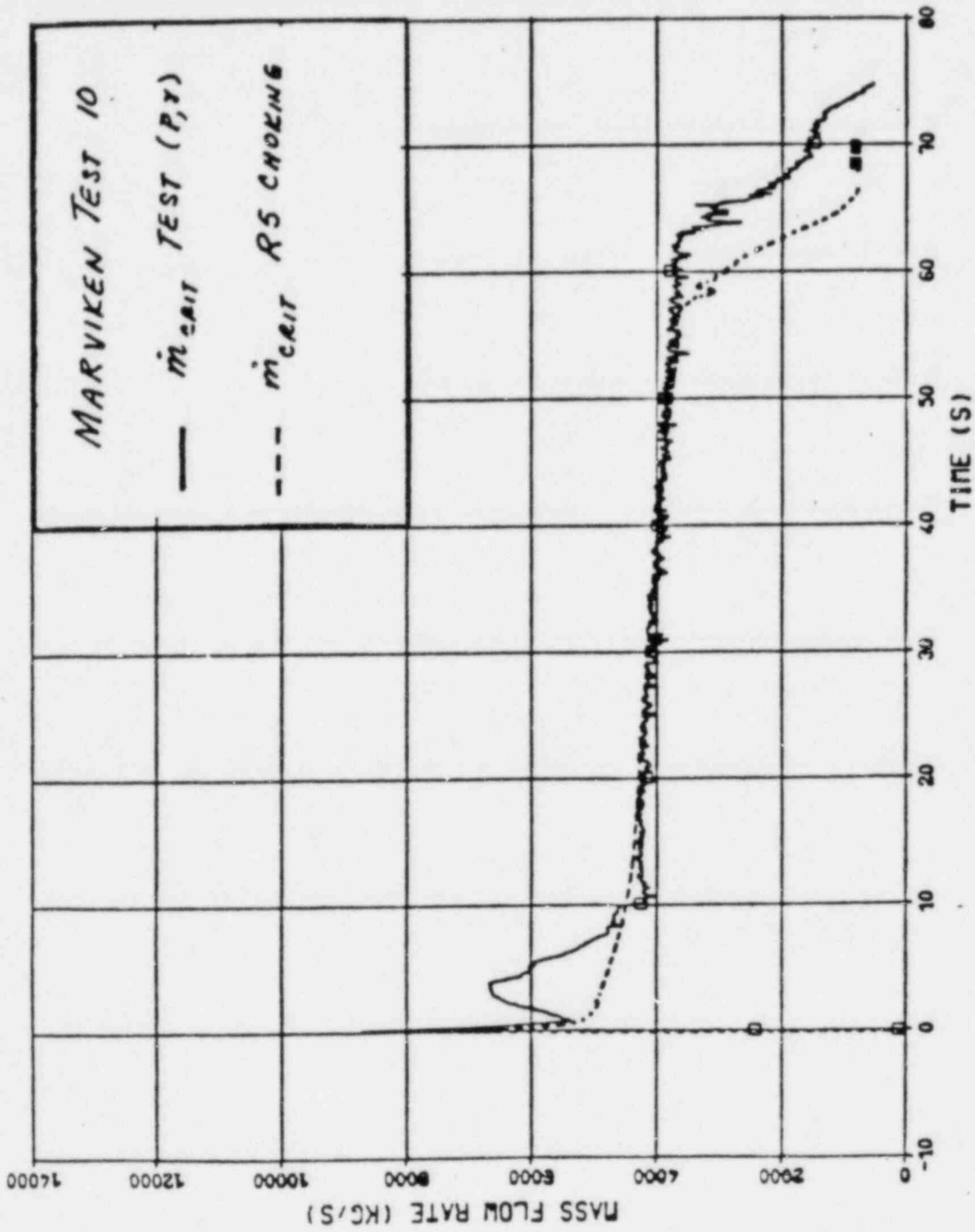
RELK.P5YA  
MODEL



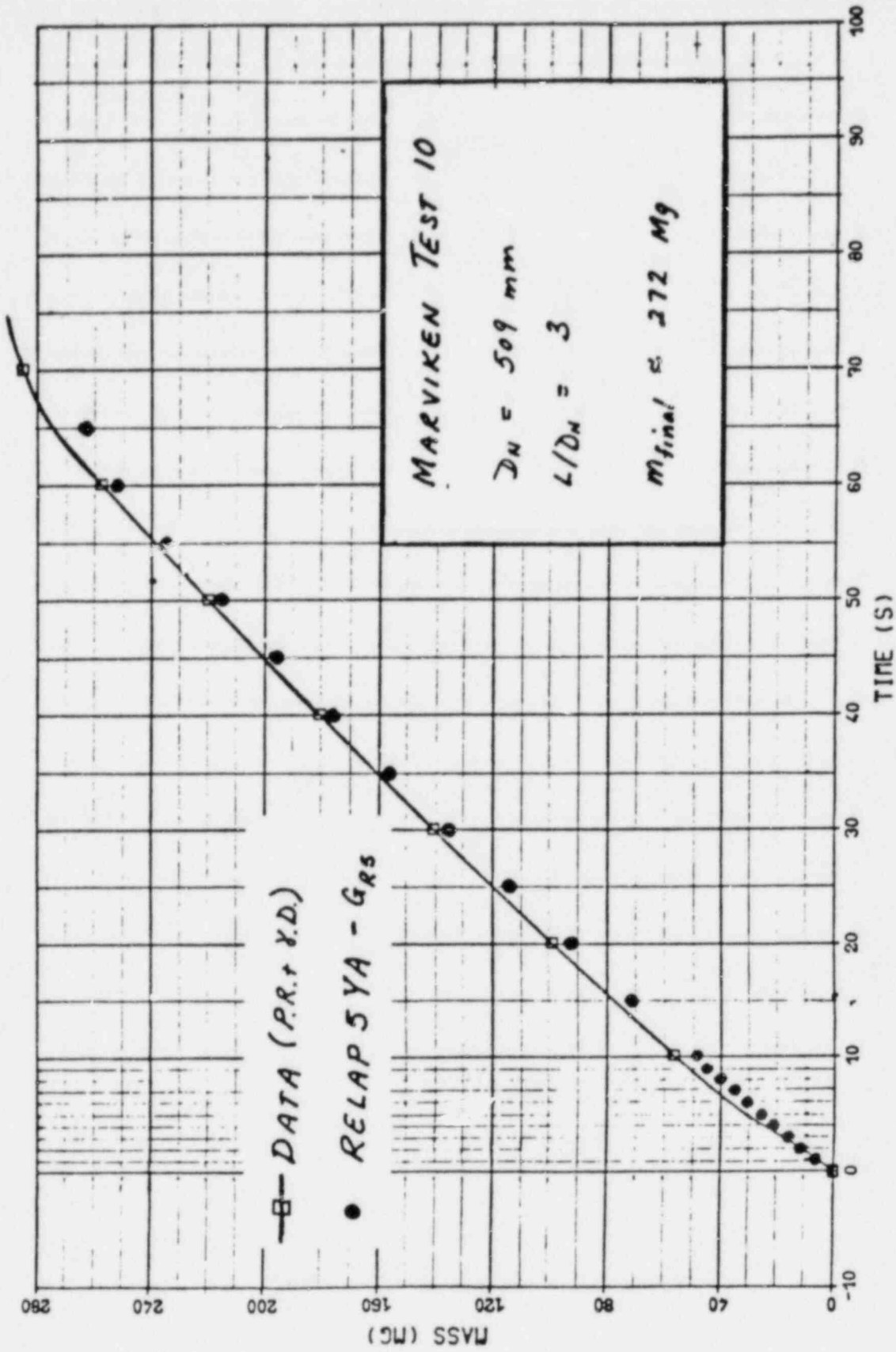
# PRESSURE HISTORY - RELAP5YA



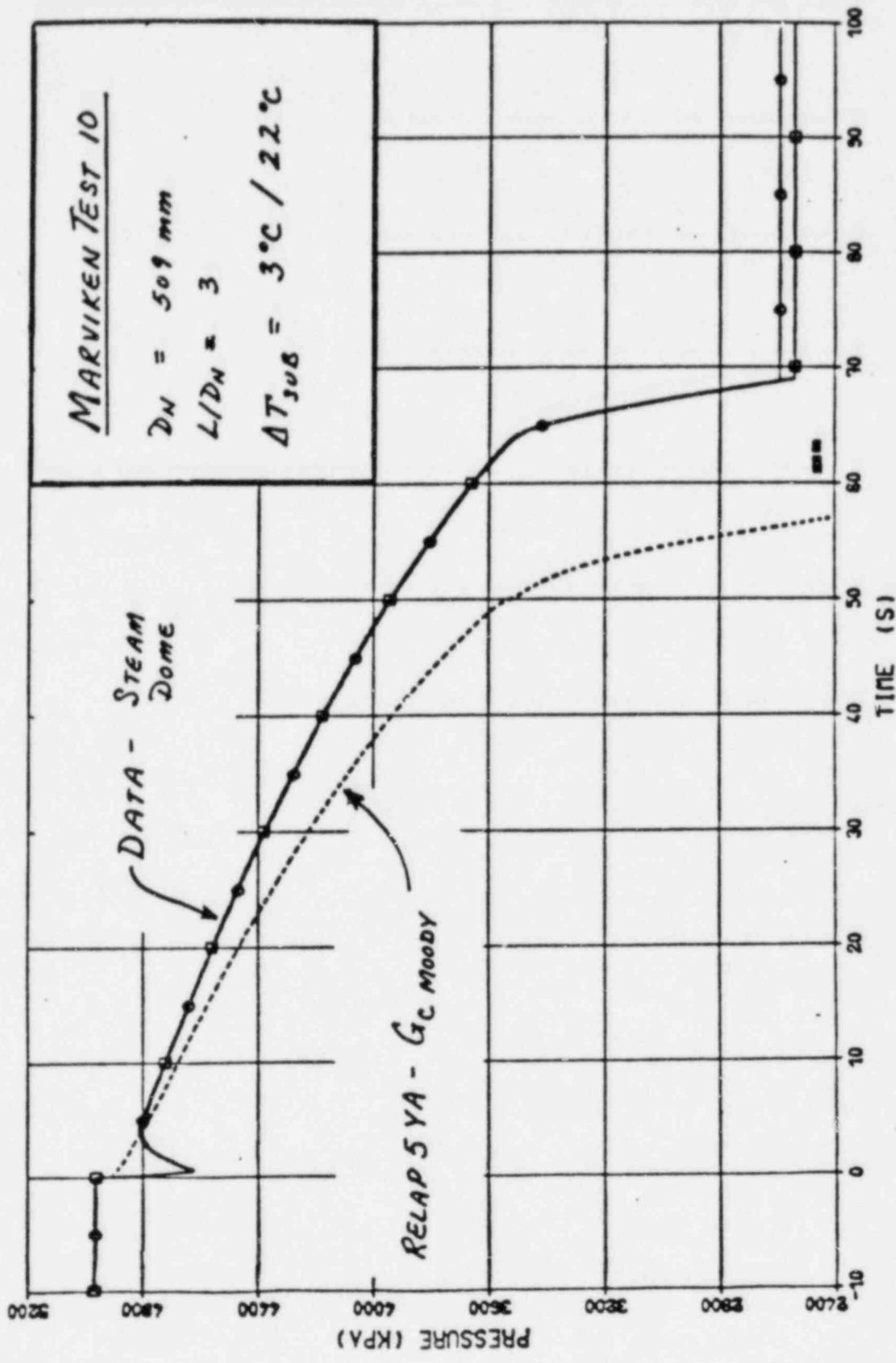
MASS FLOW RATE - RELAP 5YA



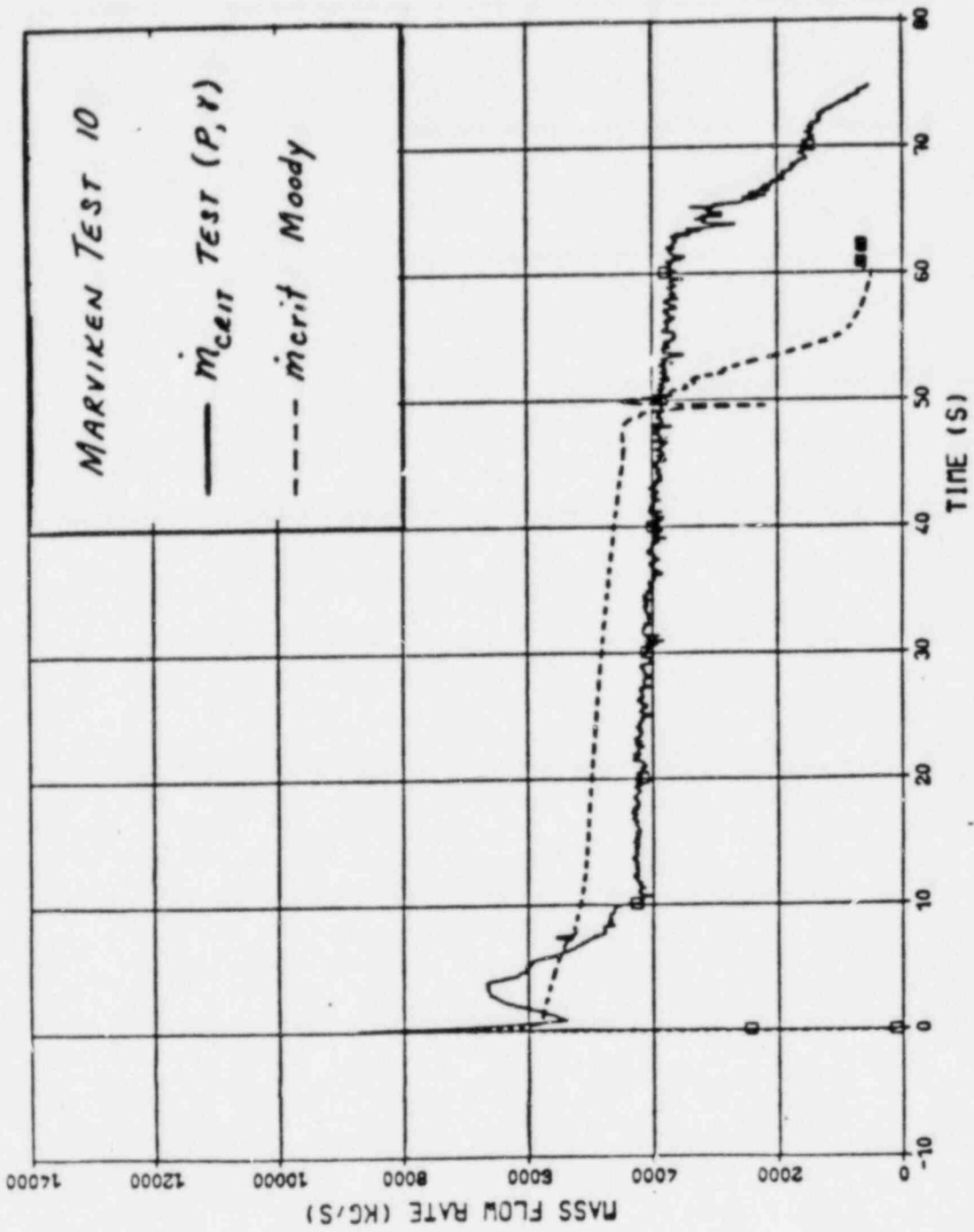
ESCAPED MASS HISTORY - RELAP 5YA



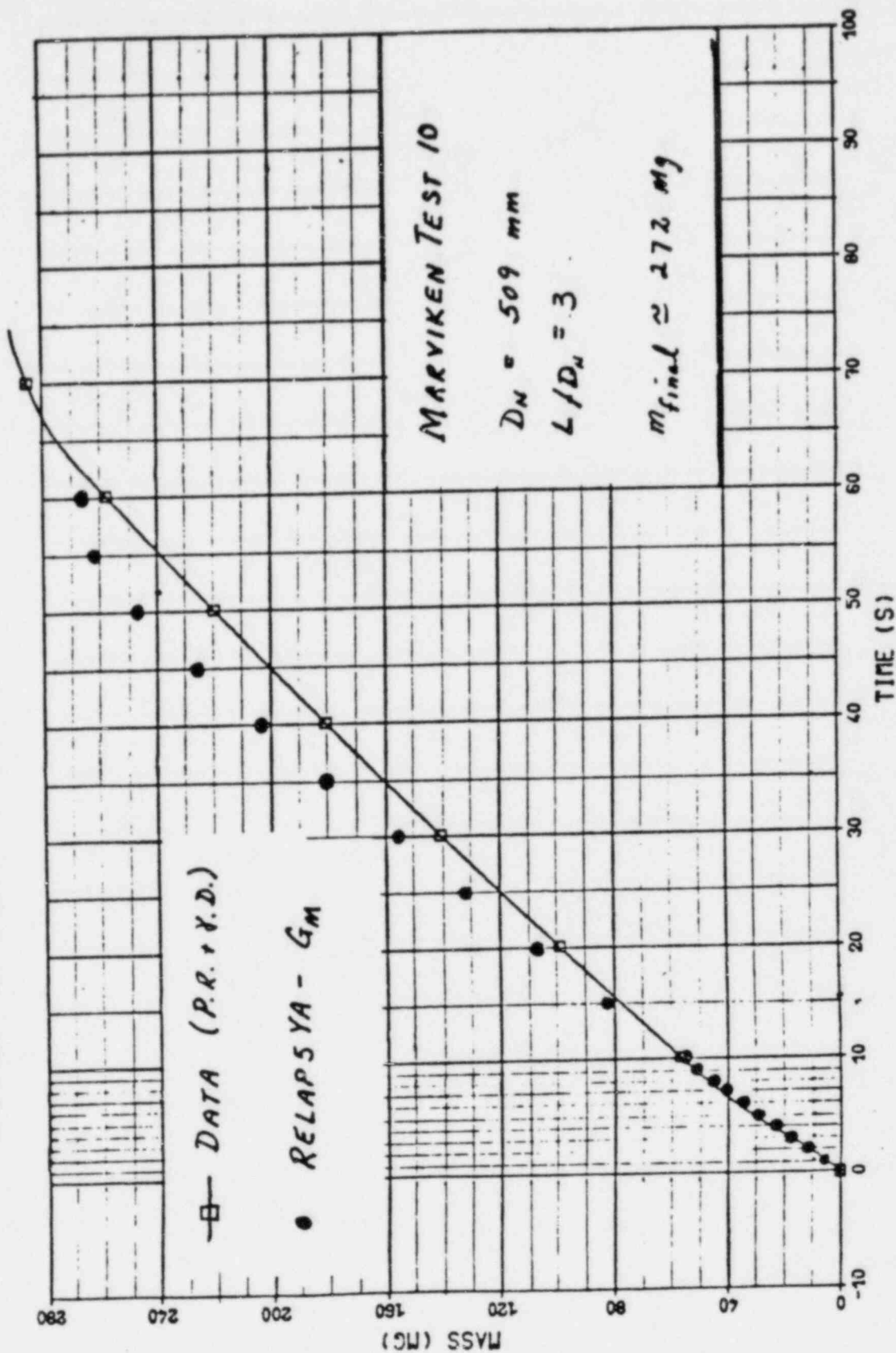
# PRESSURE HISTORY - MOODY



MASS FLOW RATE - MOODY



ESCAPED MASS HISTORY - MOODY





## ACCUMULATOR MODEL

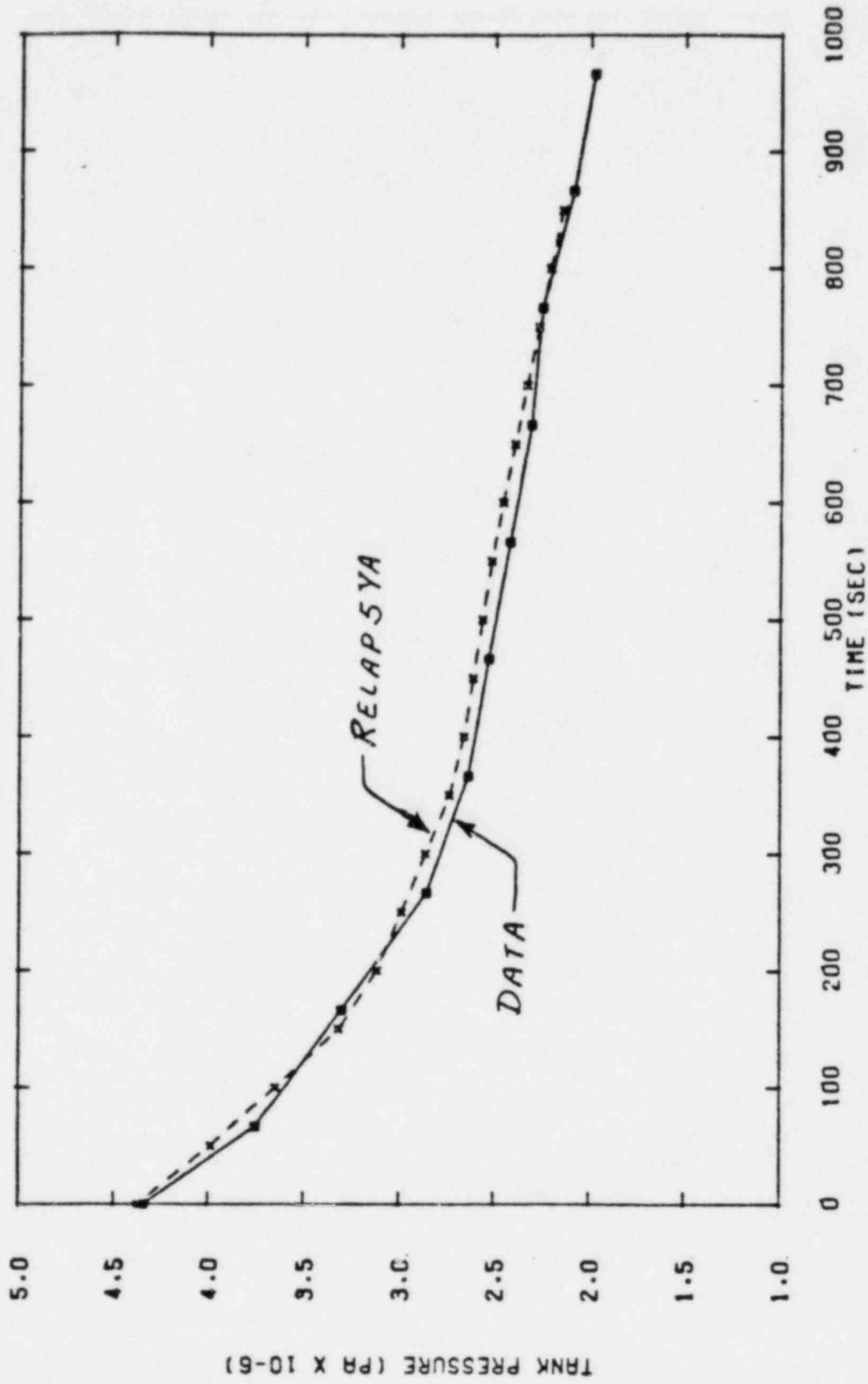
### ASSESSMENT - FIRST PASS

- MOMENTUM EQUATION ERROR
- HEAT TRANSFER
  - GRASHOF NUMBER
  - AIR - WATER DIFFUSIVITY
  - TOTAL TANK WALL TO NITROGEN
- REDERIVED AND REPROGRAMMED MODEL

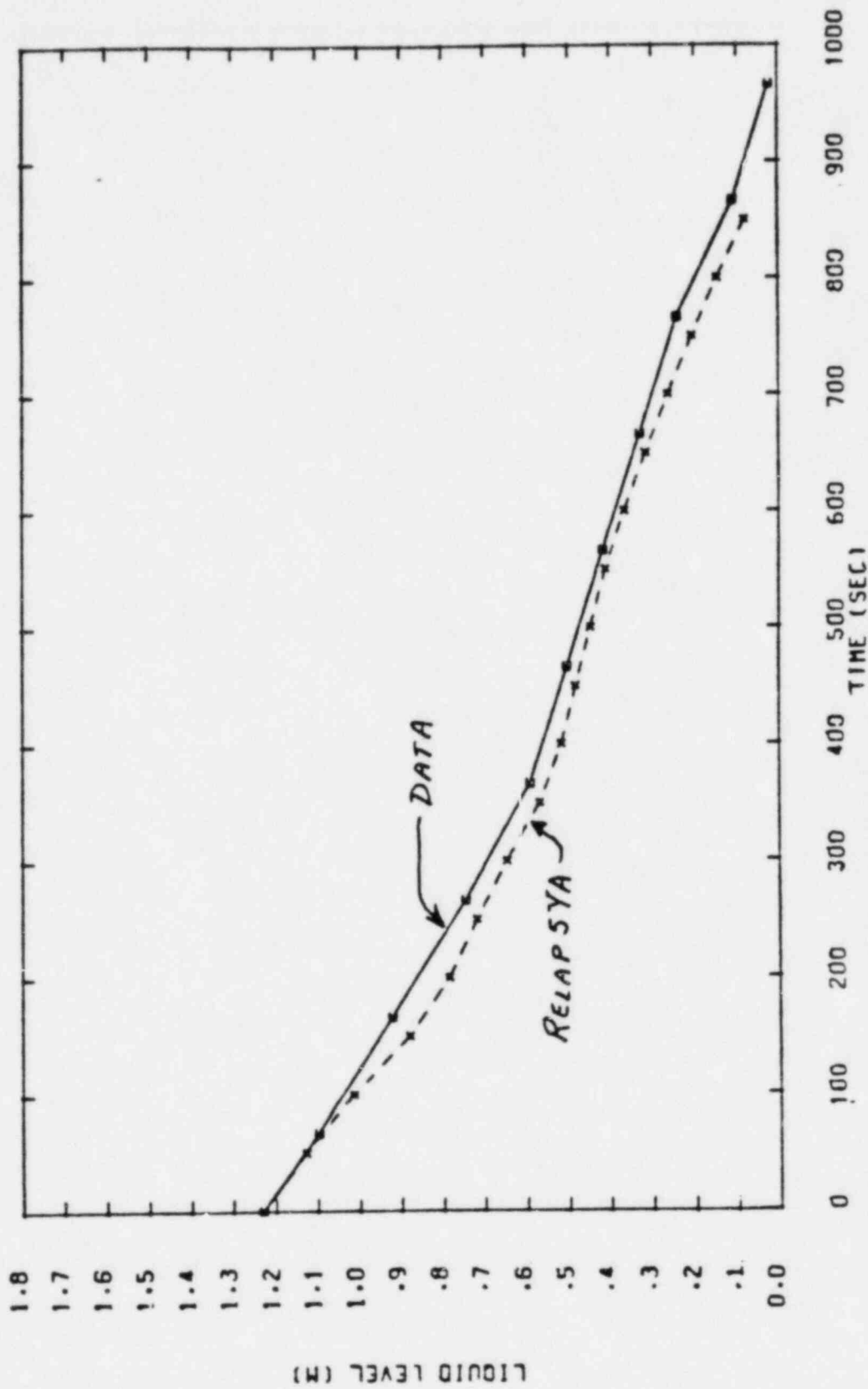
### MODEL DIFFERENCES

- MOMENTUM EQUATION GENERALIZED
  - FRICTION FACTOR USER SUPPLIED
  - FORM LOSS FROM TANK TO SURGE LINE
- HEAT TRANSFER CORRECTED
  - UNCOVERED TANK WALL TO NITROGEN
  - $Gr$ ,  $D_{w-a}$
- FOURTH ORDER RUNGE KUTTA INTEGRATION

LOFT3-1  
TANK PRESSURE VERSUS TIME

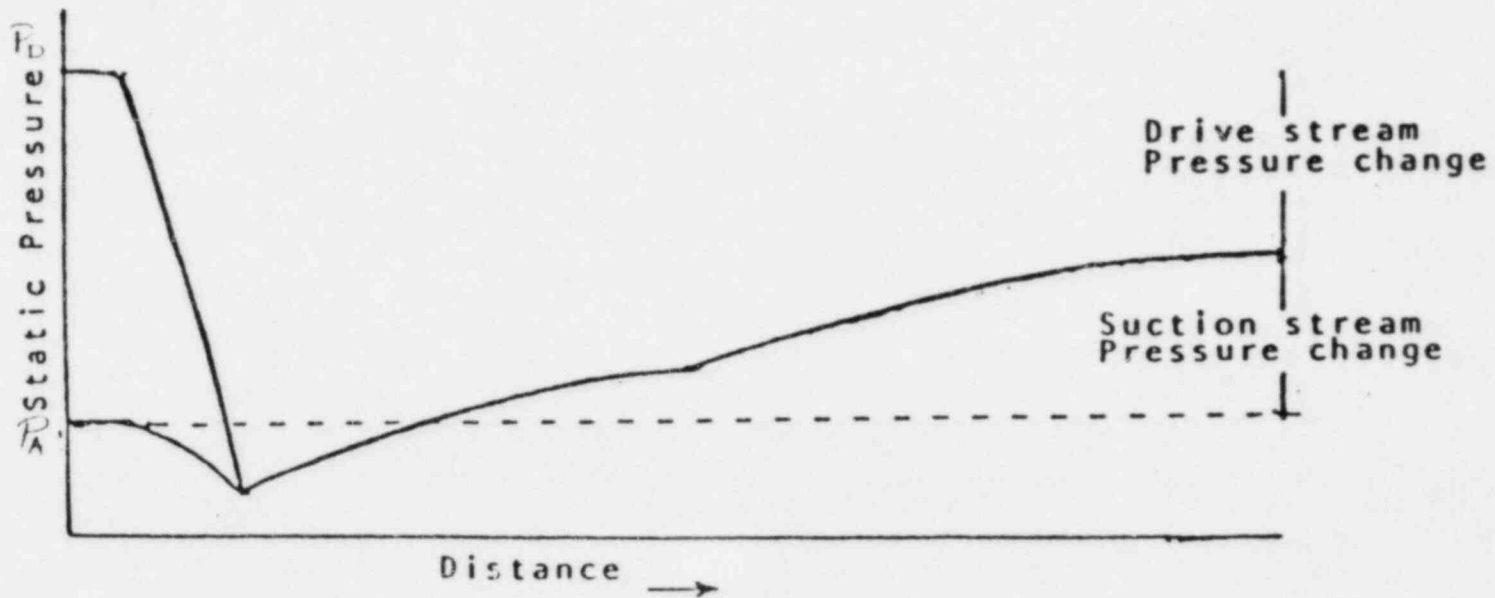
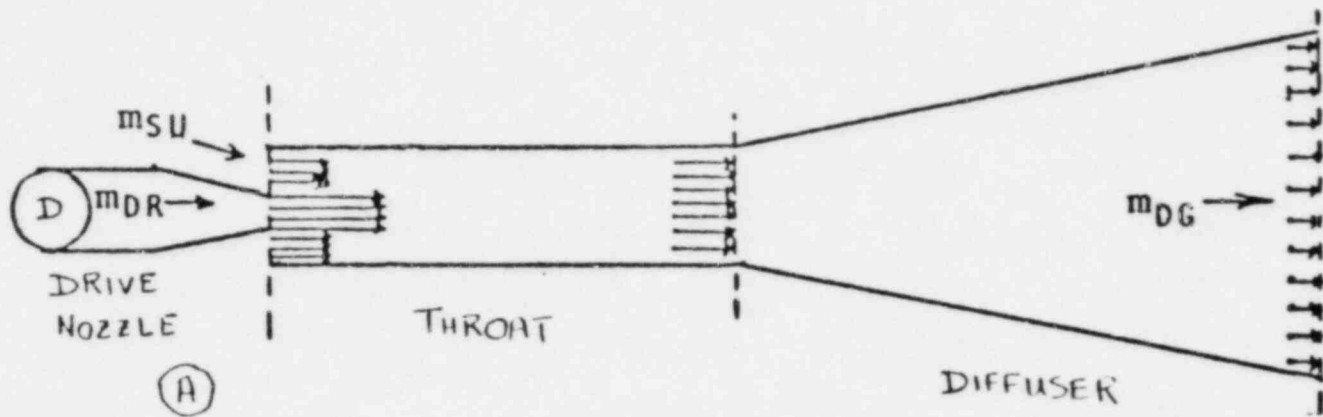


LOFT3-1  
LIQUID LEVEL VERSUS TIME

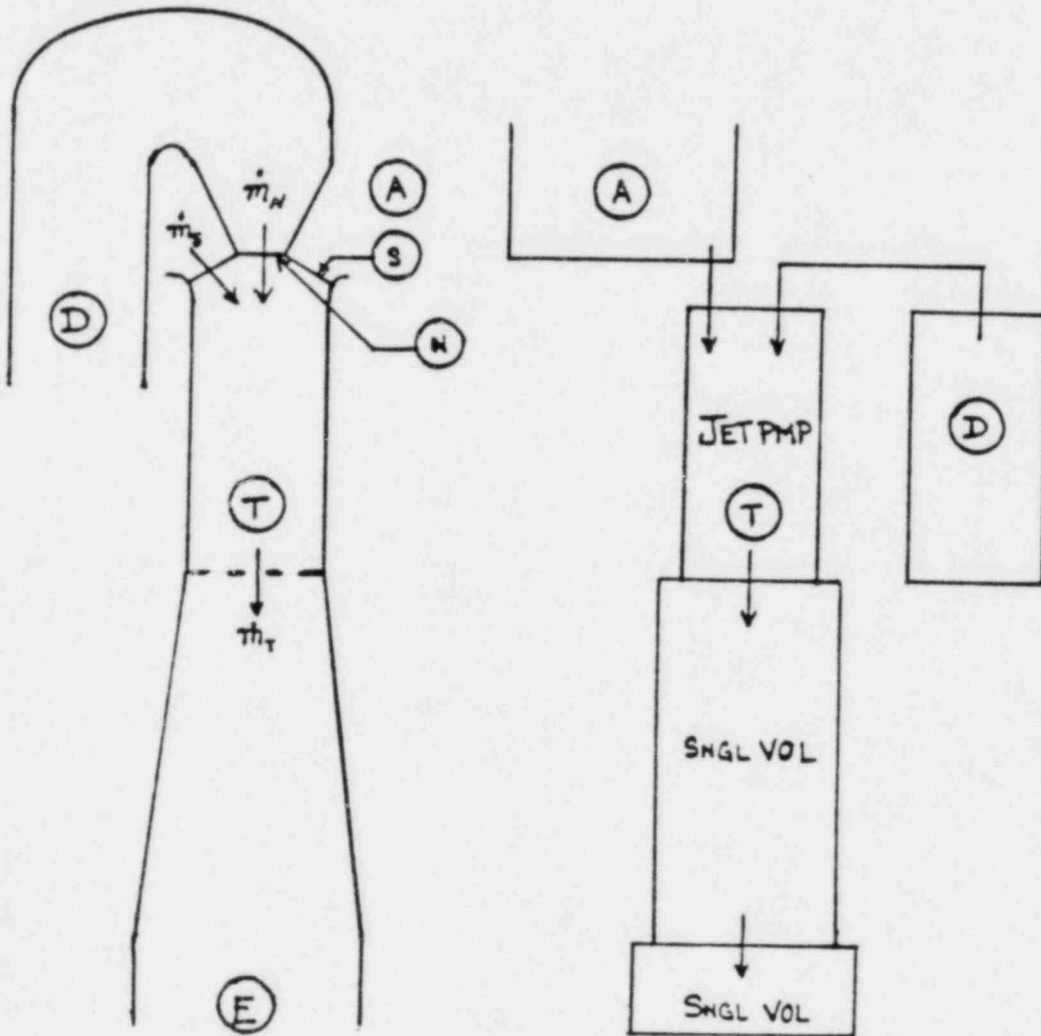


JET PUMP

MODEL



# JET PUMP MODEL



	$-\infty \leq M \leq -1$	$-1 \leq M \leq 0$	$0 \leq M \leq +\infty$
POSITIVE DRIVE			
CONTINUITY	$\dot{Q}_N - \dot{Q}_S = -\dot{Q}_T$	$\dot{Q}_N - \dot{Q}_S = \dot{Q}_T$	$\dot{Q}_N + \dot{Q}_S = \dot{Q}_T$
MECH. ENERGY	$\bar{P}_T - \bar{P}_N = EL_{TN}$ $\bar{P}_T - \bar{P}_S = EL_{TS}$	$\bar{P}_T - \bar{P}_S = EL_{TS}$ $\bar{P}_N - \bar{P}_S = EL_{NS}$	$\bar{P}_N - \bar{P}_T = EL_{NT}$ $\bar{P}_S - \bar{P}_T = EL_{ST}$
	$-\infty \leq M \leq -1$	$-1 \leq M \leq 0$	$0 \leq M \leq +\infty$
REVERSE DRIVE			
CONTINUITY	$-\dot{Q}_N + \dot{Q}_S = \dot{Q}_T$	$-\dot{Q}_N + \dot{Q}_S = -\dot{Q}_T$	$-\dot{Q}_N - \dot{Q}_S = -\dot{Q}_T$
MECH. ENERGY	$\bar{P}_S - \bar{P}_N = EL_{SN}$ $\bar{P}_S - \bar{P}_T = EL_{ST}$	$\bar{P}_T - \bar{P}_N = EL_{TN}$ $\bar{P}_S - \bar{P}_T = EL_{TS}$	$\bar{P}_T - \bar{P}_N = EL_{TN}$ $\bar{P}_T - \bar{P}_S = EL_{TS}$

MODEL DEVELOPMENT AND IMPLEMENTATION

(1) FOR STEADY STATE SINGLE PHASE

$$\bar{P}_D - \bar{P}_T = f(K, \text{Geom}, V)$$

$$\bar{P}_A - \bar{P}_T = f(K, \text{Geom}, V)$$

(2) EXTEND TO 2 $\phi$  CONDITIONS AND IMPLEMENT INTO RELAP5

POSITIVE DRIVE POSITIVE SUCTION FLOW

$$\text{MSOURCE}(\text{DRIVJ}) = f(\text{Geom}, V)$$

$$\text{MSOURCE}(\text{SUCTJ}) = f(\text{Geom}, V)$$

$$\text{FLOSS}(\text{DRIVJ}) = K$$

$$\text{FLOSS}(\text{SUCTJ}) = K$$

ALL OTHER FLOW CONFIGURATIONS

$$\text{MSOURCE}(\text{DRIVJ}) = 0.0$$

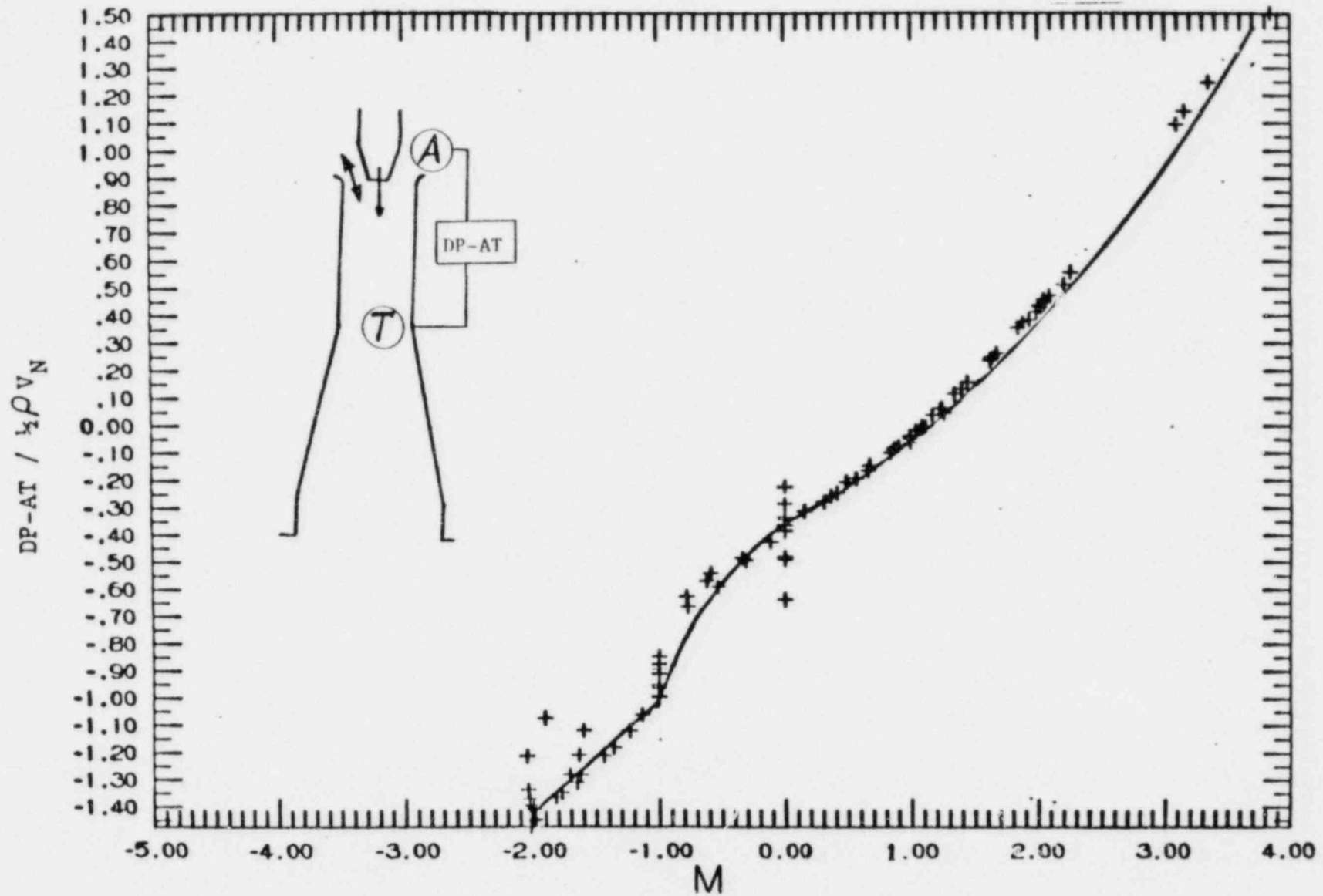
$$\text{MSOURCE}(\text{SUCTJ}) = 0.0$$

$$\text{FLOSS}(\text{DRIVJ}) = \frac{\bar{P}_D - \bar{P}_T}{\frac{1}{2} \rho V^2} = f(K, \text{Geom}, M)$$

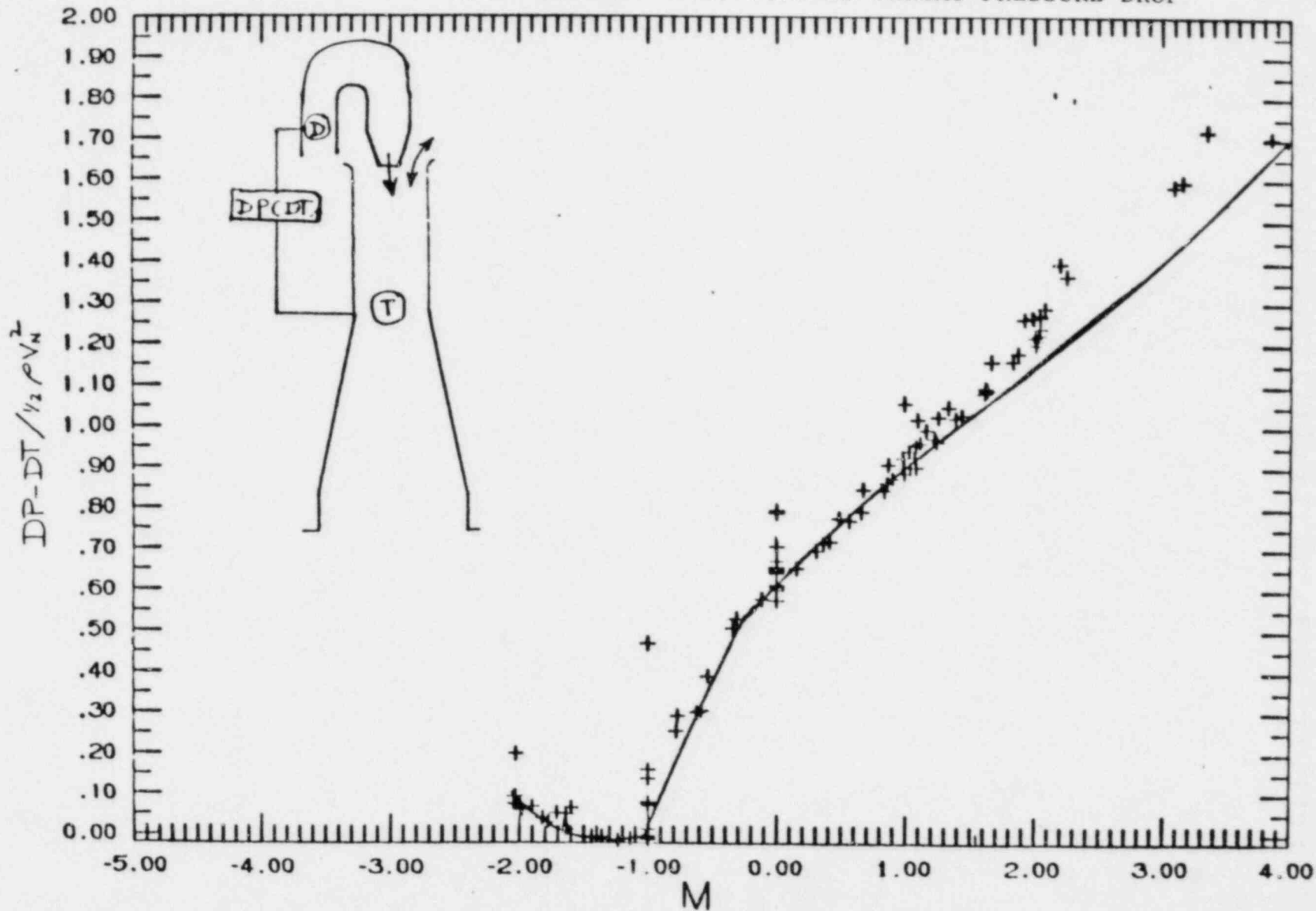
$$\text{FLOSS}(\text{SUCTJ}) = \frac{\bar{P}_A - \bar{P}_T}{\frac{1}{2} \rho V^2} = f(K, \text{Geom}, M)$$



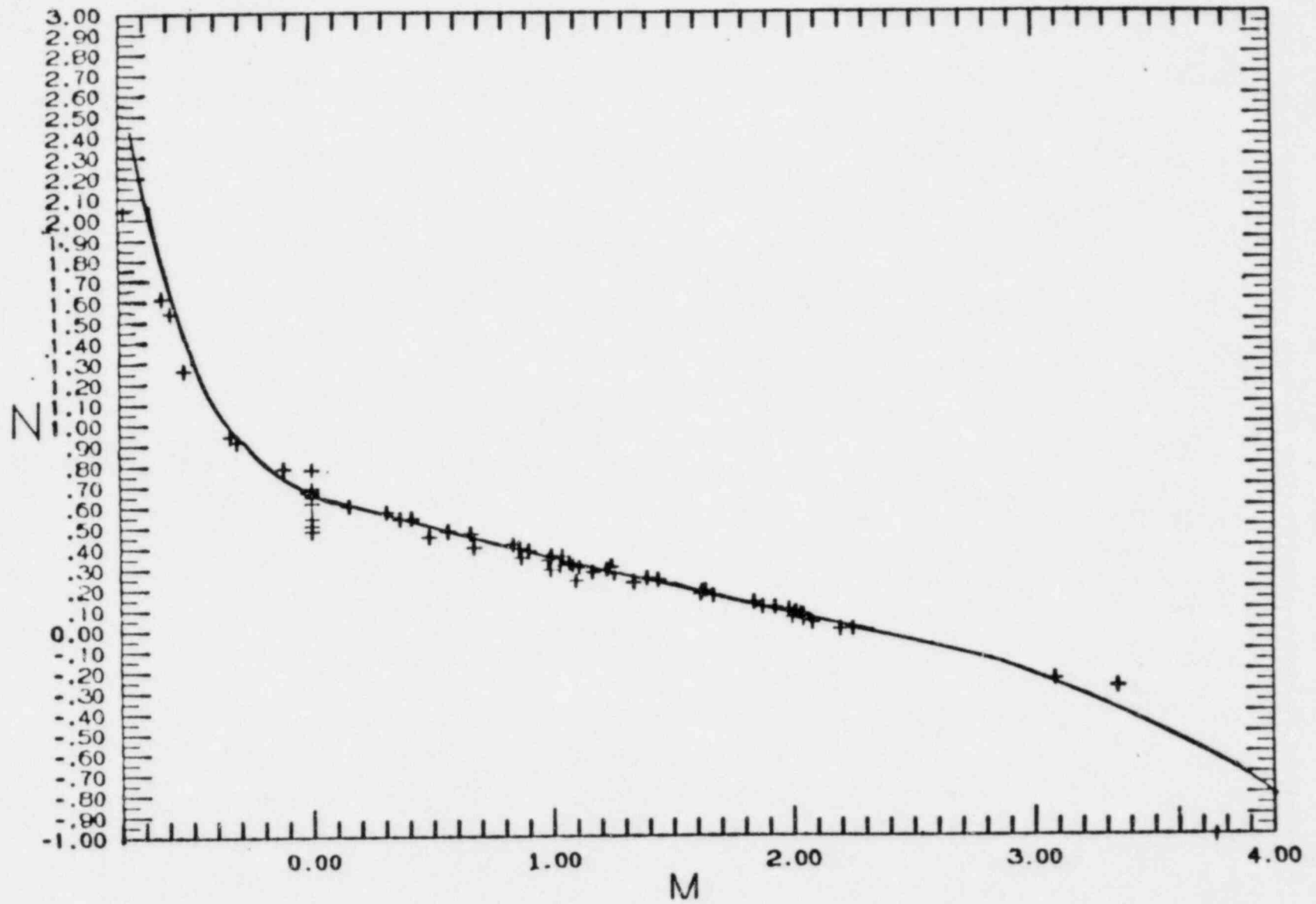
POSITIVE DRIVE; NORMALIZED DOWNCOMMER TO THROAT PRESSURE DROP



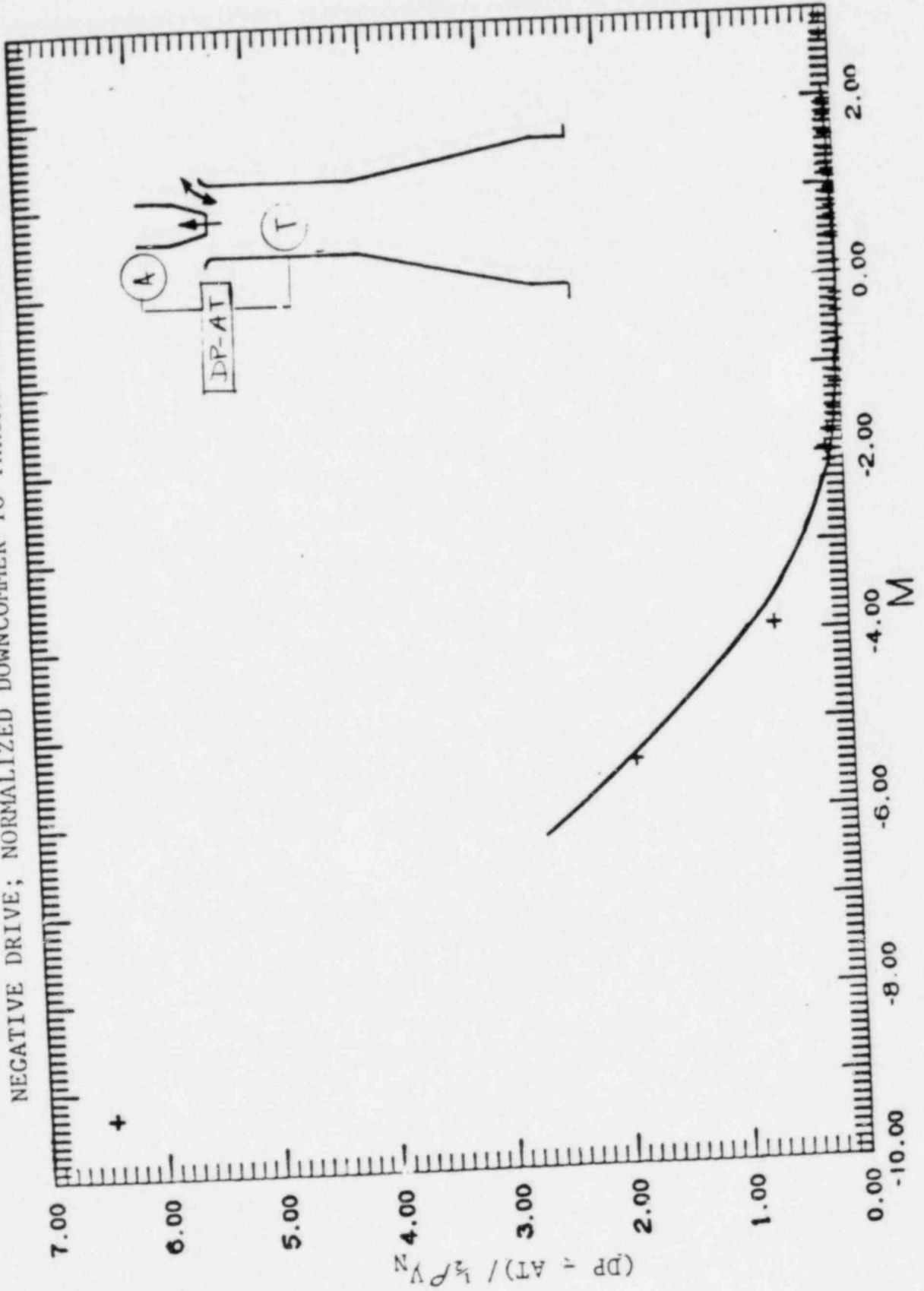
POSITIVE DRIVE; NORMALIZED DRIVE LINE TO THROAT PRESSURE DROP



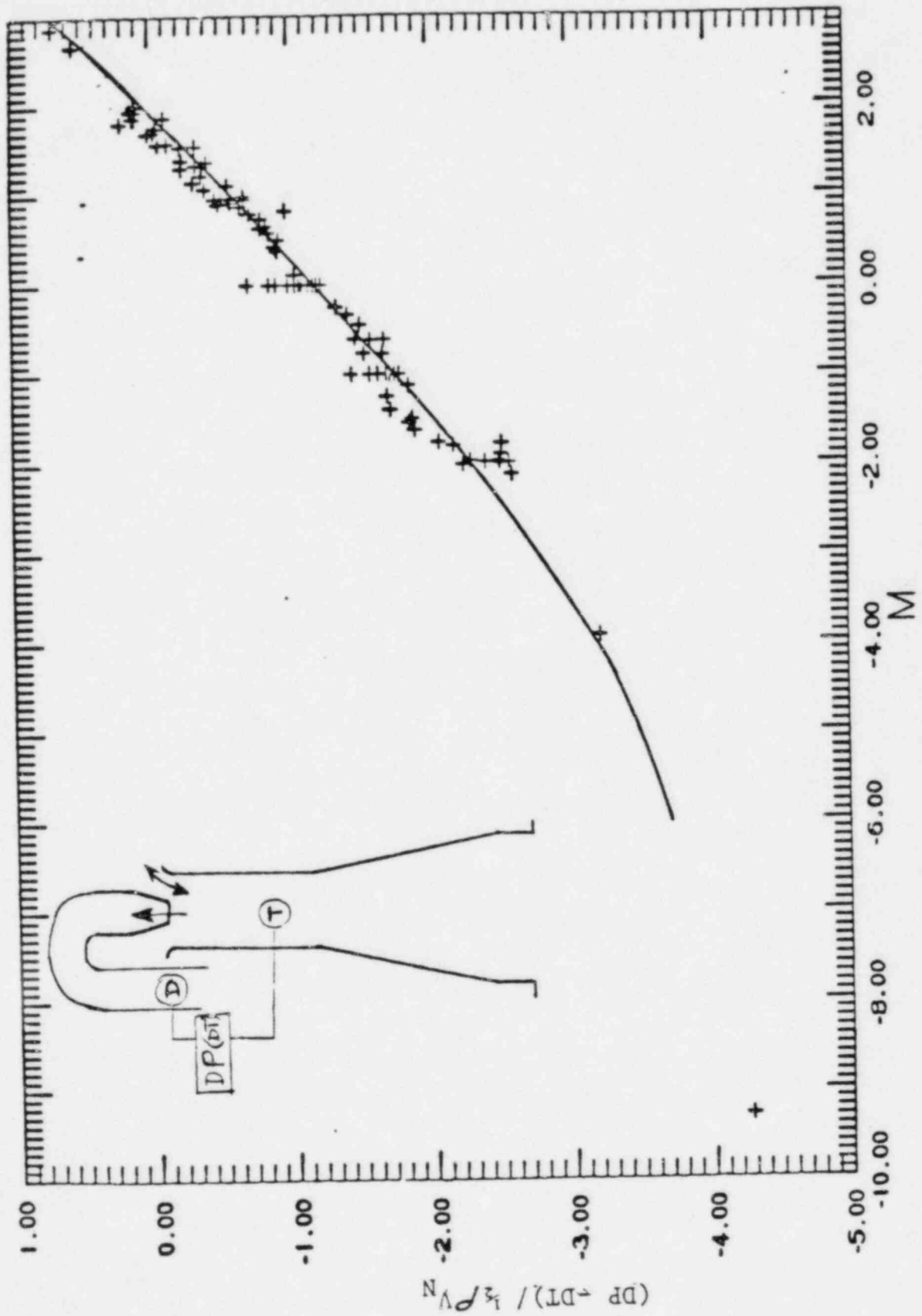
POSITIVE DRIVE; N VERSUS M



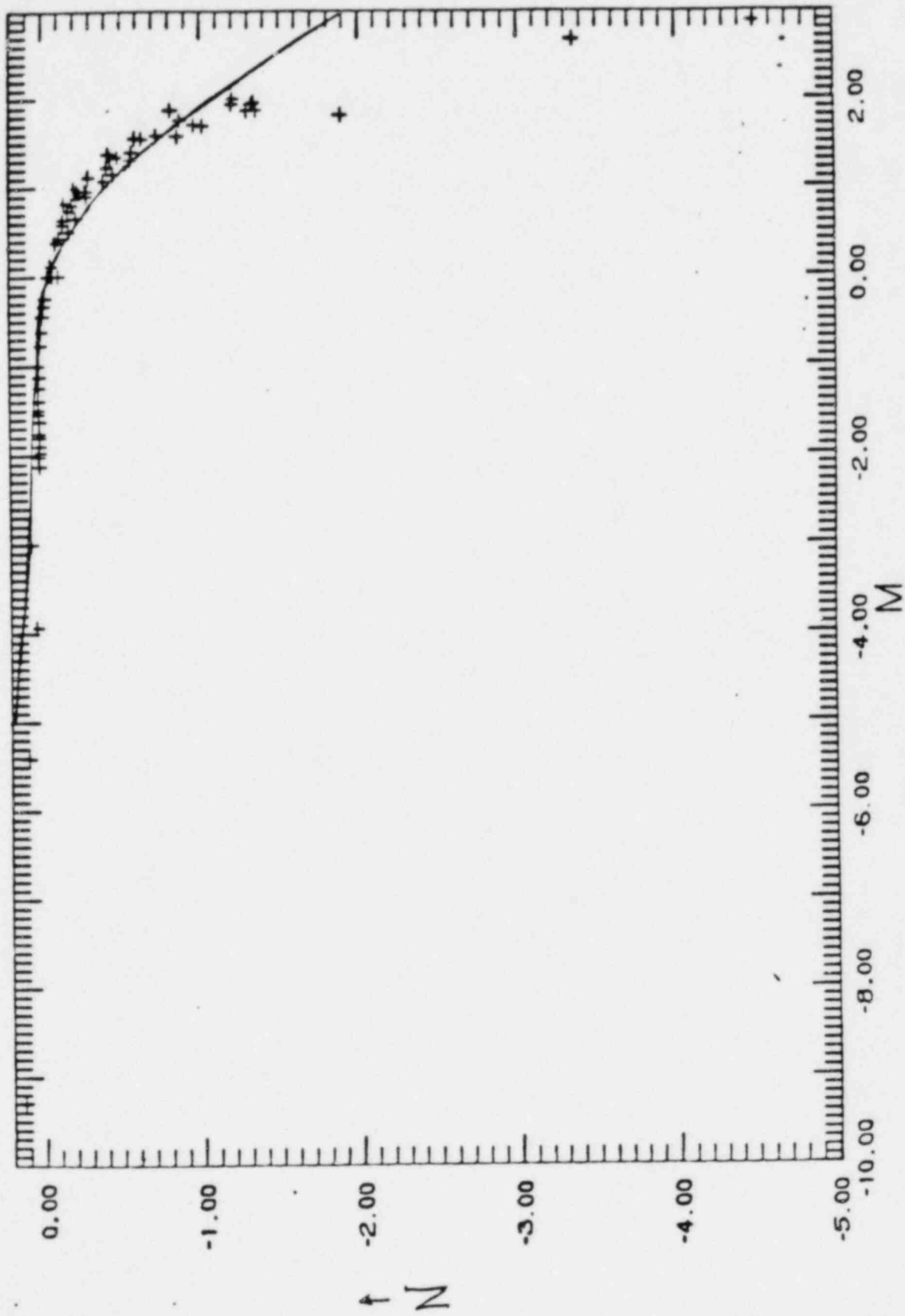
NEGATIVE DRIVE; NORMALIZED DOWNCOMER TO THROAT PRESSURE DROP



NEGATIVE DRIVE; NORMALIZED DRIVE LINE TO THROAT PRESSURE DROP



NEGATIVE DRIVE; N VERSUS M



# FORCED CONVECTIVE BOILING

REASON FOR MODIFICATION

JUSTIFICATION OF CHANGE

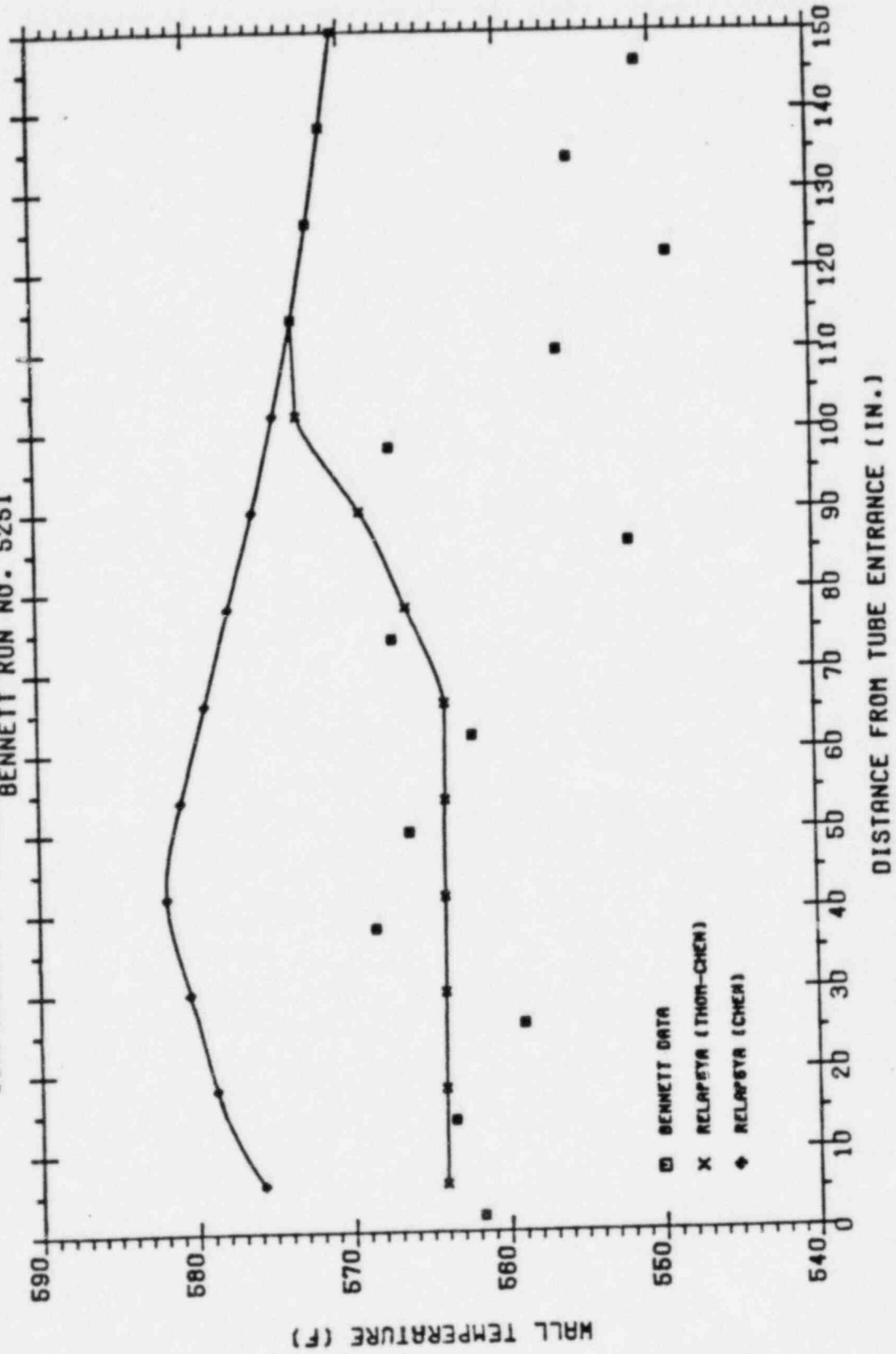
## NEW MODEL

	VOID FRACTION
THOM	0.0 - 0.8
interpolate	0.8 - 0.9
CHEN	0.9 - 1.0

RESULTS

CHEN correlation  
THOM-CHEN correlations

COMPARISON OF RELAP5 WALL TEMPERATURE PREDICTIONS WITH DATA  
 BENNETT RUN NO. 5251





## CRITICAL HEAT FLUX MODEL INCORPORATED IN RELAP5YA

### THEORY

- Modified Biasi correlation is used for high mass flux
- Griffith-Zuber correlation is used for low mass flux
- Linear interpolation between the two correlations is used for intermediate mass flux
- critical void fraction (used to determine dryout) depends on mass flux

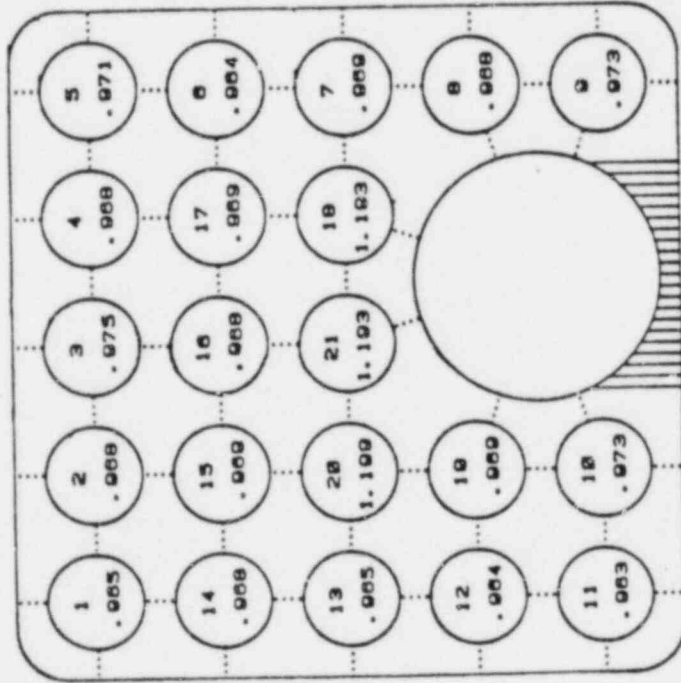
### SEPARATE EFFECTS TESTS

- Columbia University
- General Electric Co. Nine-Rod
- ORNL THTF

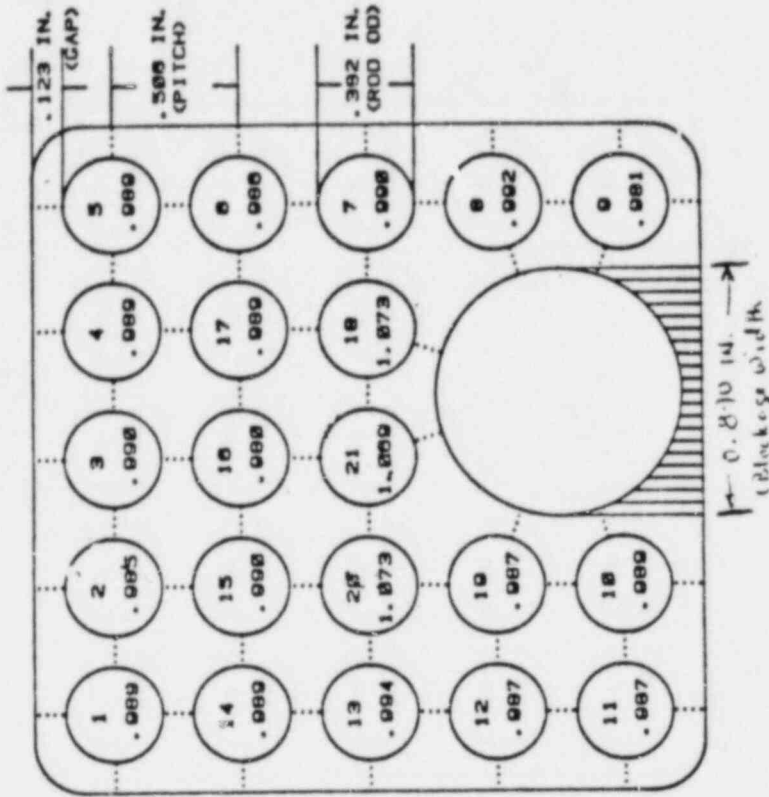
STEADY STATE CHF TEST CONDITIONS

Experiment	Pressure (psi)	Ave. Mass Flux (Mlb/hr-ft <sup>2</sup> )	Ave. Heat Flux (MBtu/hr-ft <sup>2</sup> )	Heated Rod Diameter (in.)	Heated Length (in.)
Columbia	1500 - 2005	1.968 - 2.008	0.282 - 0.436	0.382	150.0
GE 9 Rod	997 - 1005	0.249 - 1.248	0.289 - 0.522	0.570	72.0
ORNL THTF	635 - 1849	0.166 - 0.525	0.14 - 0.29	0.374	144.0

TEST SECTION NUMBER 08

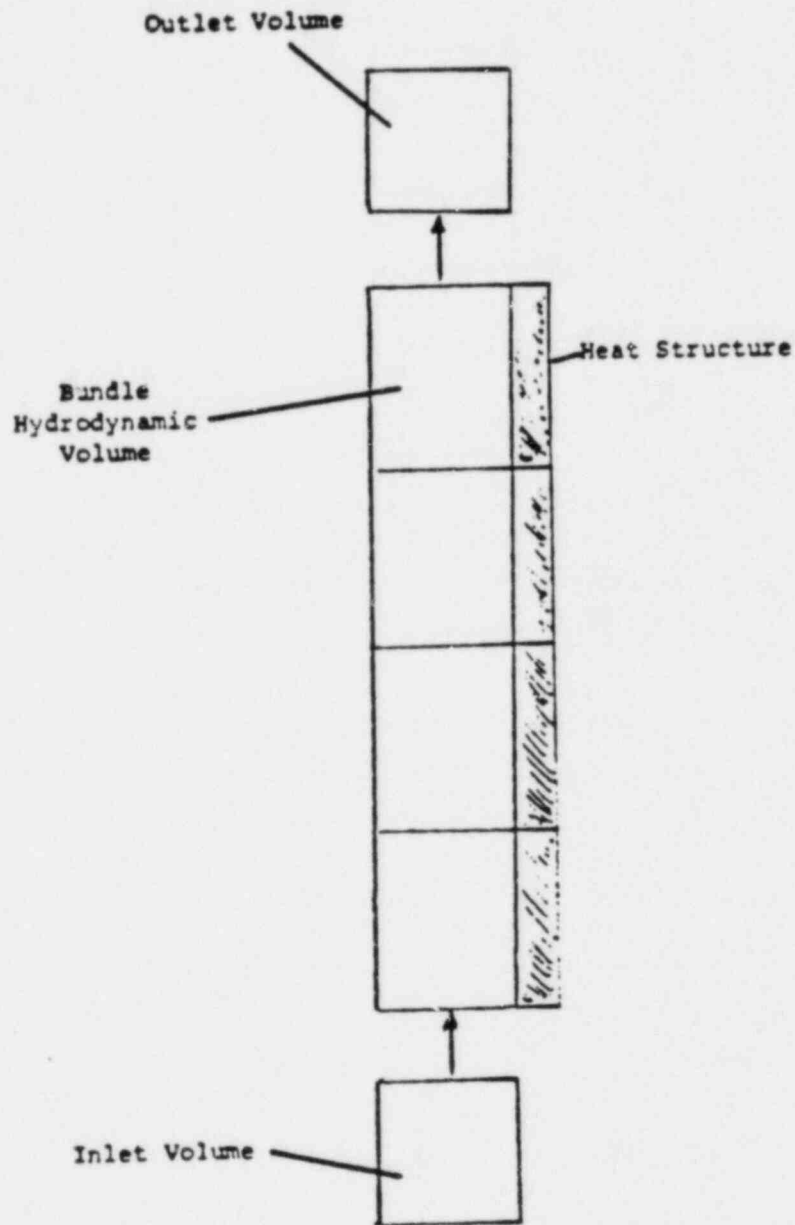


TEST SECTION NUMBER 71



XX  
 X,XXX  
 ----- HEATER ROD NUMBER  
 ----- NORMALIZED POWER FACTOR

Figure 3.1 - 1  
Columbia Radial Geometry and Power Distribution



RELAPS Model For Columbia Test Sections 68 and 71

TEST SECTION 68:

Run Number 18

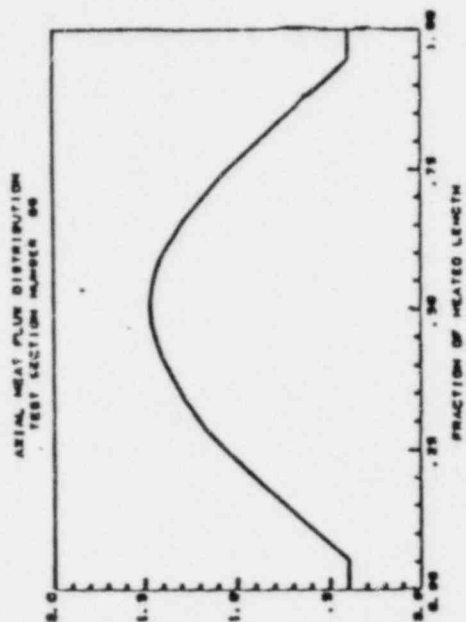
Inlet Temp. = 581.7°F  
 P = 1755. psia  
 G = 1.980 Mlb/hr-ft<sup>2</sup>

Run Number 22

Inlet Temp. = 553.4  
 P = 1755.  
 G = 2.008

Run Number 27

Inlet Temp. = 520.5  
 P = 1500.  
 G = 1.995



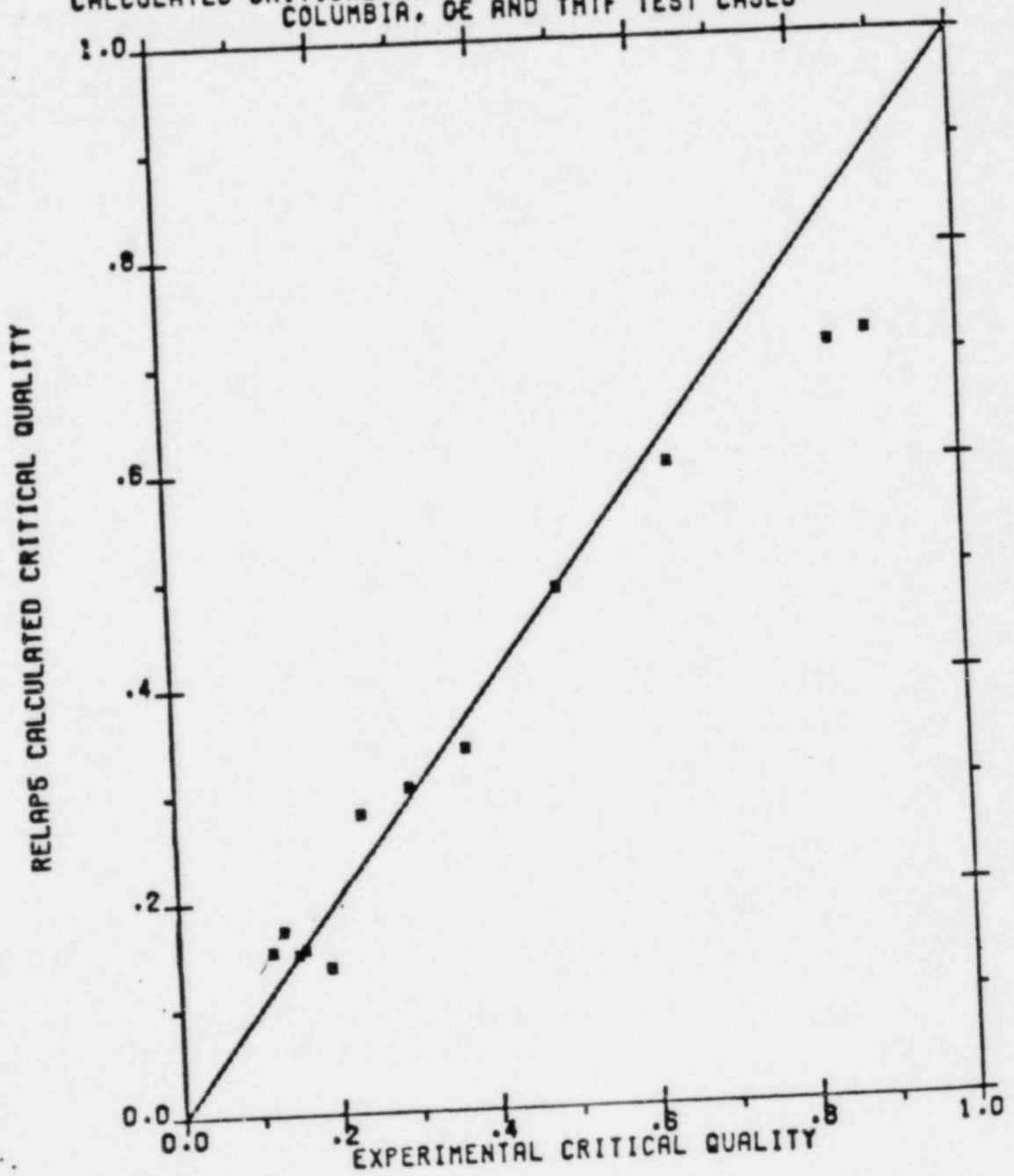
Normalized heat flux

Run Number 18		Run Number 22		Run Number 27	
Experimental Data	RELAPS Calculation	Experimental Data	RELAPS Calculation	Experimental Data	RELAPS Calculation
Outlet	Outlet	Outlet	Outlet	Outlet	Outlet
○	○	○	○	○	○
○	●	○	○	●	○
●	●	●	●	●	●
●	●	●	○	○	○
○	○	○	○	○	○
○	○	○	○	○	○
	○		○		○
	○		○		○
	○		○		○
	○		○		○
Inlet	Inlet	Inlet	Inlet	Inlet	Inlet

KEY	
○	CHF not exceeded
●	CHF exceeded

MEASURED AND CALCULATED AXIAL CHF LOCATIONS

CALCULATED CRITICAL QUALITY VS. EXPERIMENTAL CRIT. QUALITY  
COLUMBIA, DE AND THF TEST CASES



# RADIATION HEAT TRANSFER

## ASSUMPTIONS

- GRAY, DIFFUSE SURFACES; NON-PART. FLUID
- UNIFORM  $T_i$ ,  $\rho_i$ ,  $R_i$  ON EACH SURFACE
- $0 \leq \epsilon_i$ , CONSTANT  $\leq 1.0$

## THEORY

- $R_i = [\delta_{ij} - \rho_i F_{ij}]^{-1} [\epsilon_i \sigma T_i^4]$
- $q''_{\text{RAD},i} = \frac{\epsilon_i}{\rho_i} [\sigma T_i^4 - R_i]$
- $A_i F_{ij} = A_j F_{ji}$  Reciprocity
- $\sum_{j=1}^n F_{ij} = 1.0$  Continuity
- $\epsilon_i = (1 - \rho_i)$

## IMPLEMENTATION

- $T_i \geq T_{\text{RAD ON}}$ ,  $\alpha_i \geq \alpha_{\text{RAD ON}}$
- $-K \frac{\partial T}{\partial r}^{n+1} = h_c^{n+1} [T_i^{n+1} - T_f^n] + q''_{\text{RAD},i}^n$

## HEAT TRANSFER LOCKOUT OPTIONS

### RETURN TO NUCLEATE BOILING LOCKOUT

- USER SELECTED STRUCTURES (JEM1=1)  
IF  $q''_{WALL} > q''_{CHF}$ , IEM1 = 1
- USER SUPPLIED NUCLEATE BOILING MULTIPLIER  
$$h_{NB-EM} = X_{MNB} \cdot h_{NB-Pool, SUB, SAT}$$
- UNLOCKED BY  
REWET - QUENCH MODEL AFTER QUENCHING  
RESTART

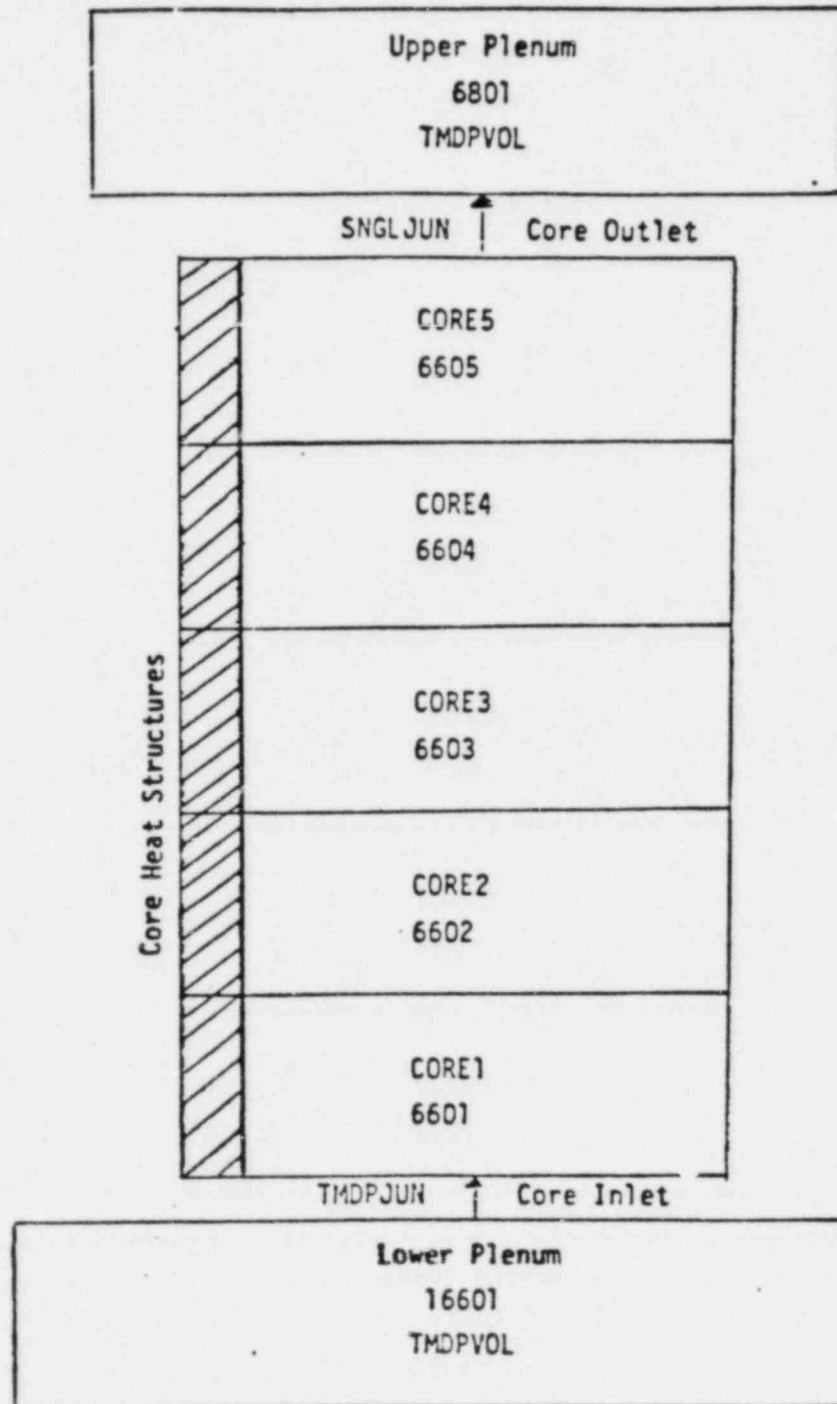
### RETURN TO TRANSITION BOILING LOCKOUT

- USER SELECTED STRUCTURES (JEM2=1)  
IF  $\Delta T_{W, SUP} \geq 300^\circ F$ , IEM2 = 1  
$$h_{RTB-EM} = h_{RNB-EM} = h_{FILM BOILING, HI OR LO FLOW}$$
- UNLOCKED BY  
REWET - QUENCH MODEL AFTER QUENCHING  
RESTART



# RELAP5YA NODALIZATION DIAGRAM

## HEAT TRANSFER LOCKOUT CASES

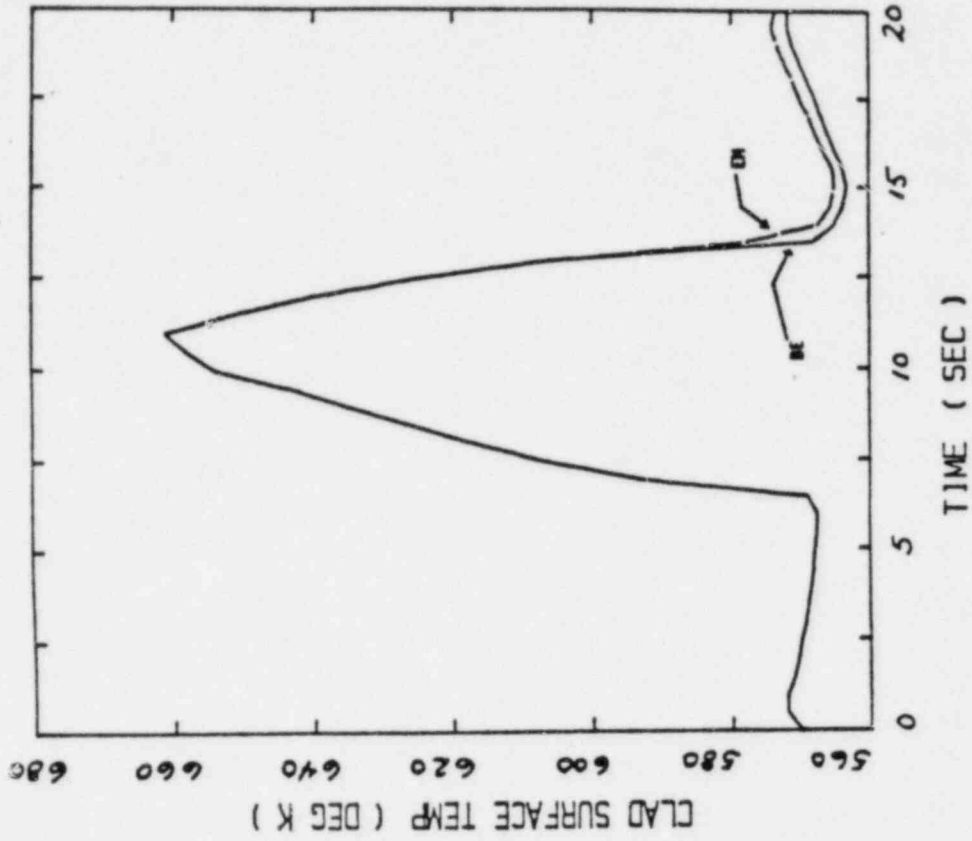
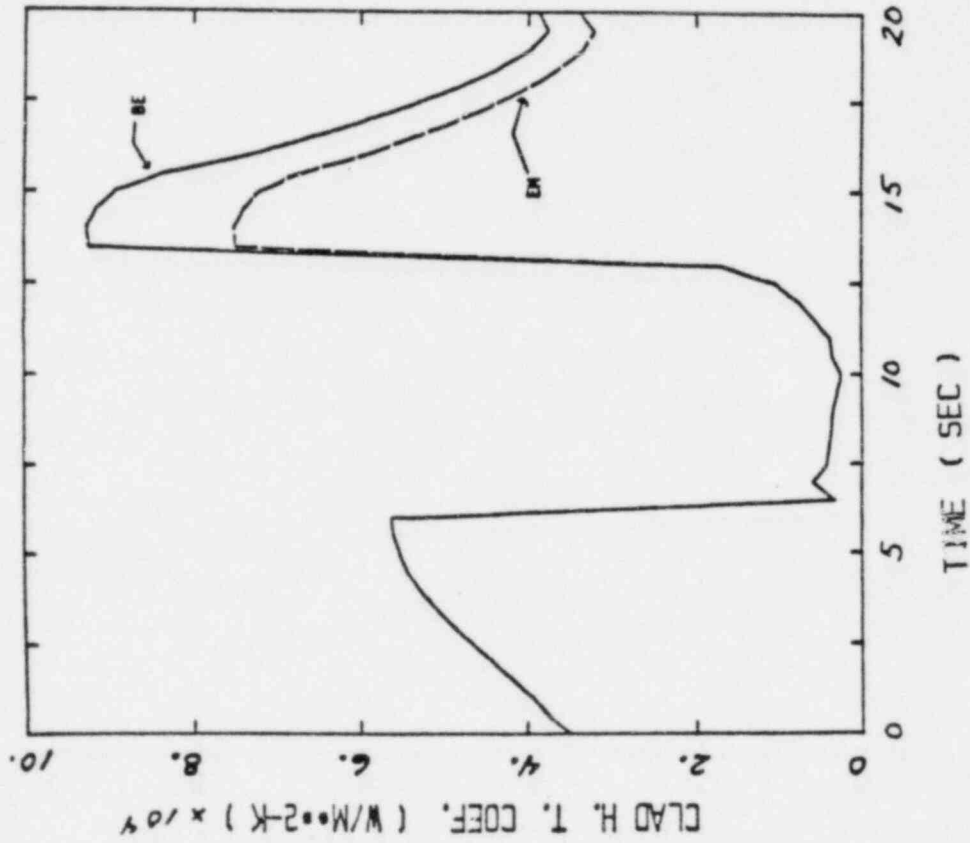


Comparison of BE and EM clad surface temperature under rewet conditions

(IEMI Flag Activated).

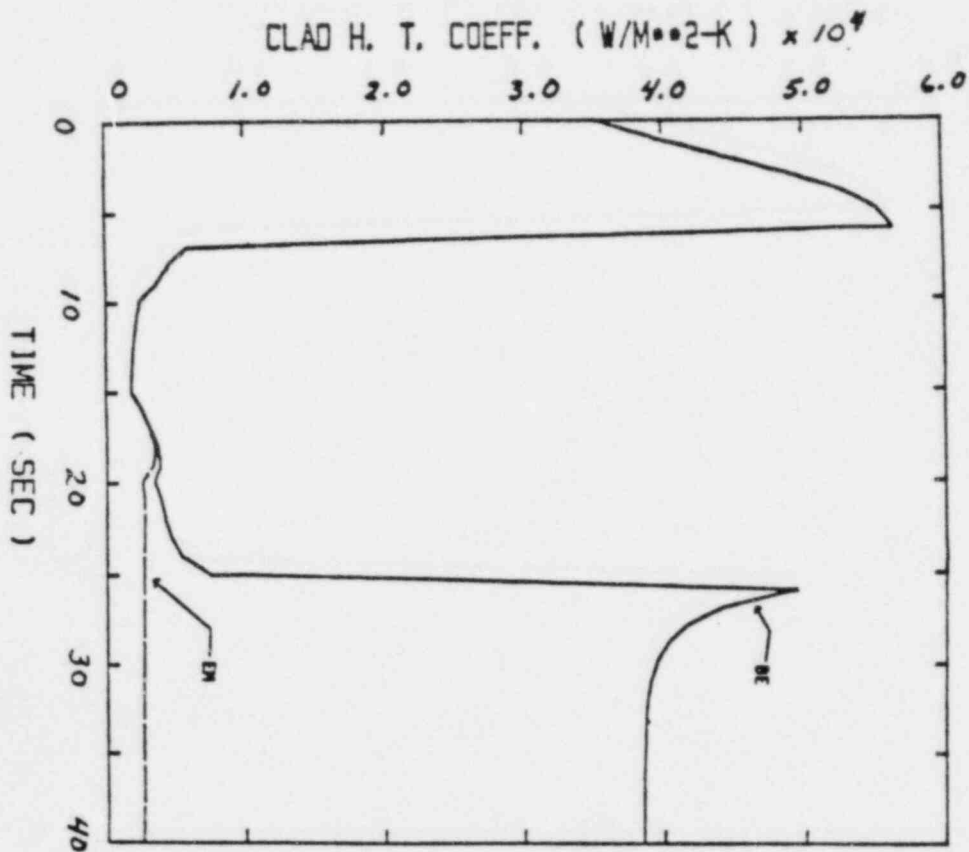
TEST PROBLEM 2

IEMI FLAG TEST  
BE VS EM (0.8 MULT)

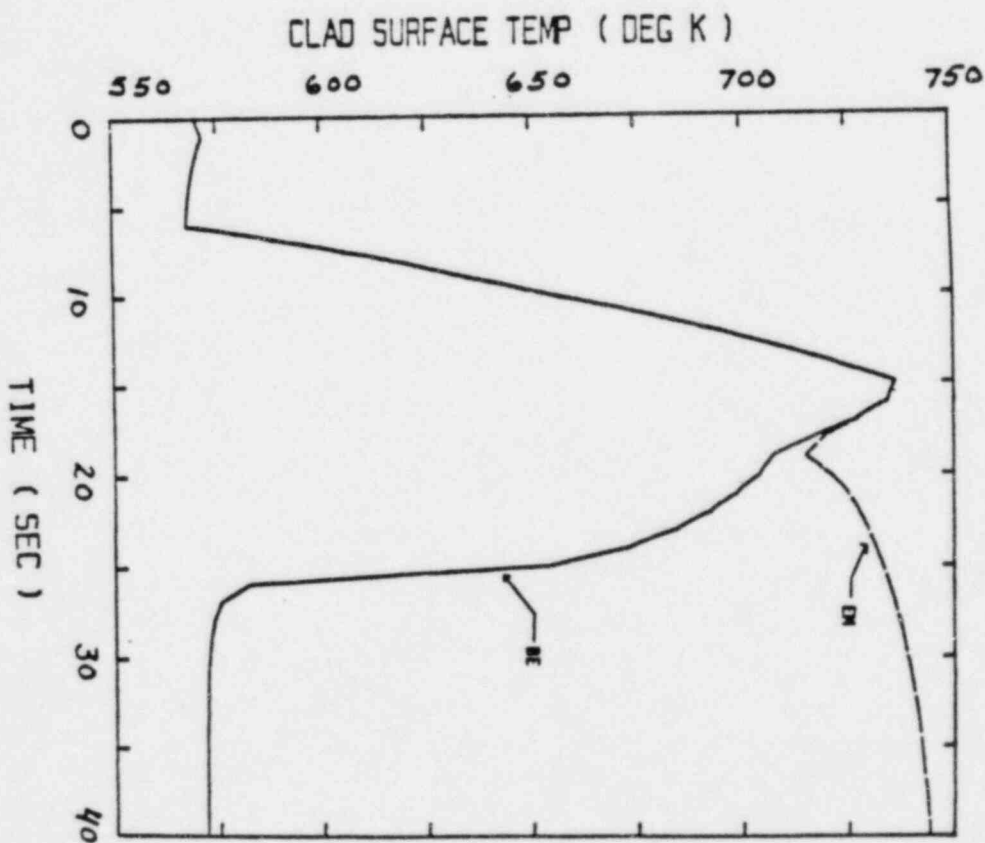


Comparison of BE and EM clad surface temperature using transition boiling  
lockout (IEM2 Flag Activated).

TEST PROBLEM 3  
IEM2 FLAG TEST

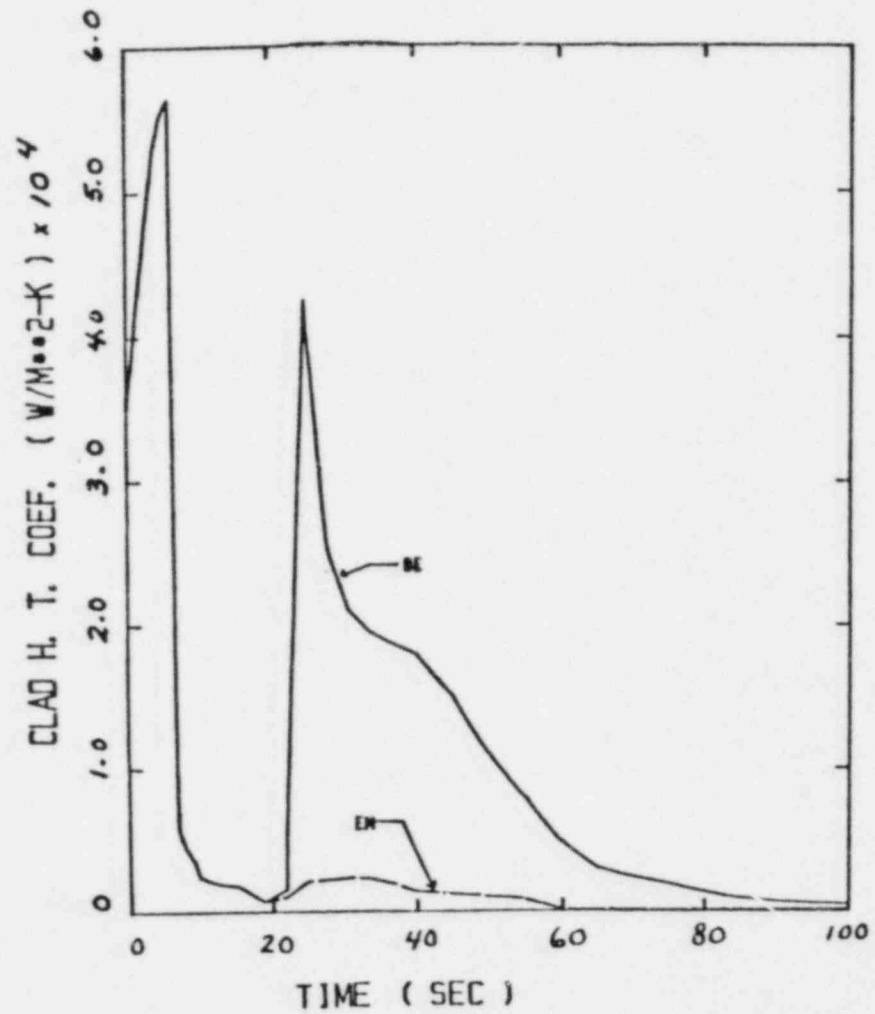


TEST PROBLEM 3  
IEM2 FLAG TEST

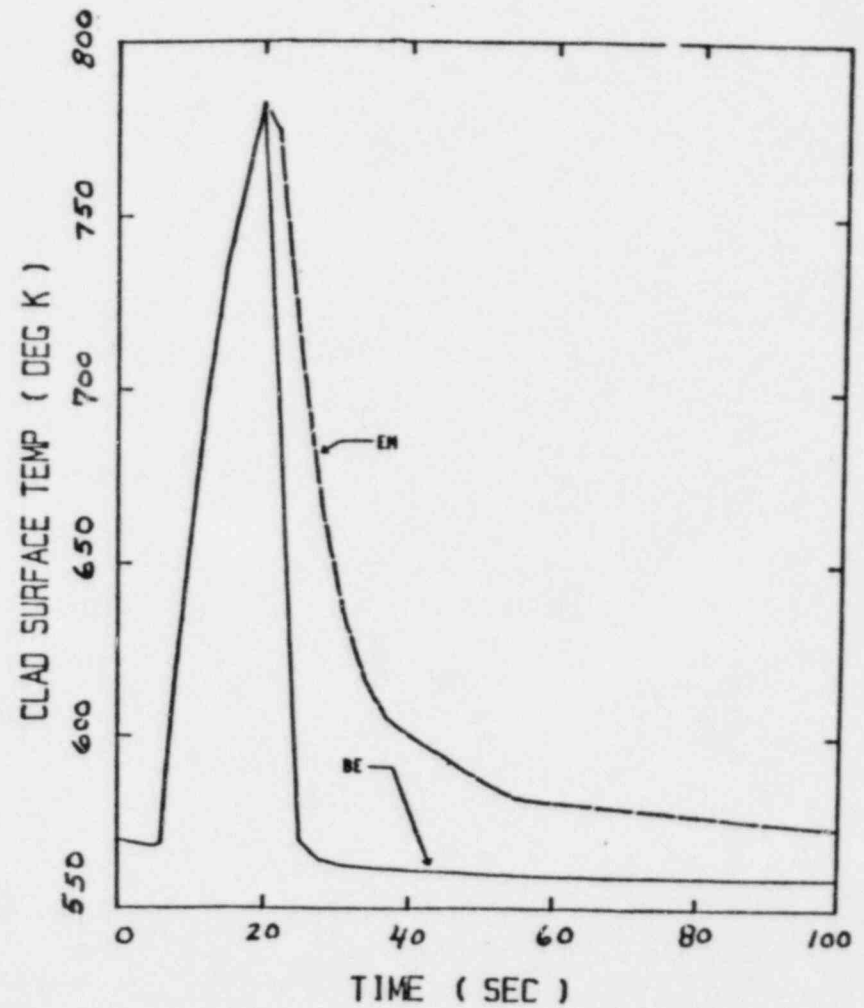


Comparison of BE and EM clad heat transfer coefficients under low flow conditions using transition boiling lockout.

TEST PROBLEM 4  
IEM2 FLAG TEST



TEST PROBLEM 4  
IEM2 FLAG TEST



FUEL BEHAVIOR MODELS INCORPORATED IN RELAP5YA

- fuel rod internal pressure
- fuel rod deformation
- gap heat transfer
- metal-water reaction

Basis of models is fuel rod evaluation model routines from YAEC's RELAP4/MOD3 licensed code.

Major modifications made in clad deformation and rupture models to comply with new requirements mandated by the NRC.

Fuel rod internal pressure model simplified.

ASSUMPTIONS IN RELAP5YA FUEL BEHAVIOR MODELS

1. Fuel rod deformation is calculated only for the radial direction, no axial deformation is calculated.
2. Fuel rod clad strain and rupture calculations are based on a single rod rather than a rod bundle.
3. Fuel rod deformation does not affect flow channel geometry.
4. In clad elastic and plastic deformation calculations, cladding is assumed to be divided into nodes of equal thickness and clad strain is applied to clad mid-radius.
5. Fuel and clad thermal expansion calculations assume input radii are specified at 68°F.
6. Emissivity of fuel pellets and cladding are constant.
7. Gap heat transfer coefficient calculation does not consider contact heat transfer.
8. Metal-water reaction is not steam-limited.
9. Fuel pellet melting is assumed not to occur.
10. Ideal gas behavior in fuel rod.

FUEL BEHAVIOR MODELS INCORPORATED IN RELAP5YA

Model	Calculated Quantities
1. Fuel rod internal pressure	1. fuel rod internal pressure
2. Fuel rod deformation	1. fuel pellet thermal expansion 2. clad temperature ramp rate 3. clad hoop stress 4. clad elastic strain 5. clad plastic strain 6. total clad strain(mid-radius of clad) 7. clad rupture temperature 8. clad rupture 9. flow blockage
3. Gap heat transfer	1. gap heat transfer coefficient 2. gap heat capacity
4. Metal-water reaction	1. heat produced by metal-water reaction inside and outside of clad 2. oxide layer thickness at inner and outer clad surfaces

FIGURE 6.3

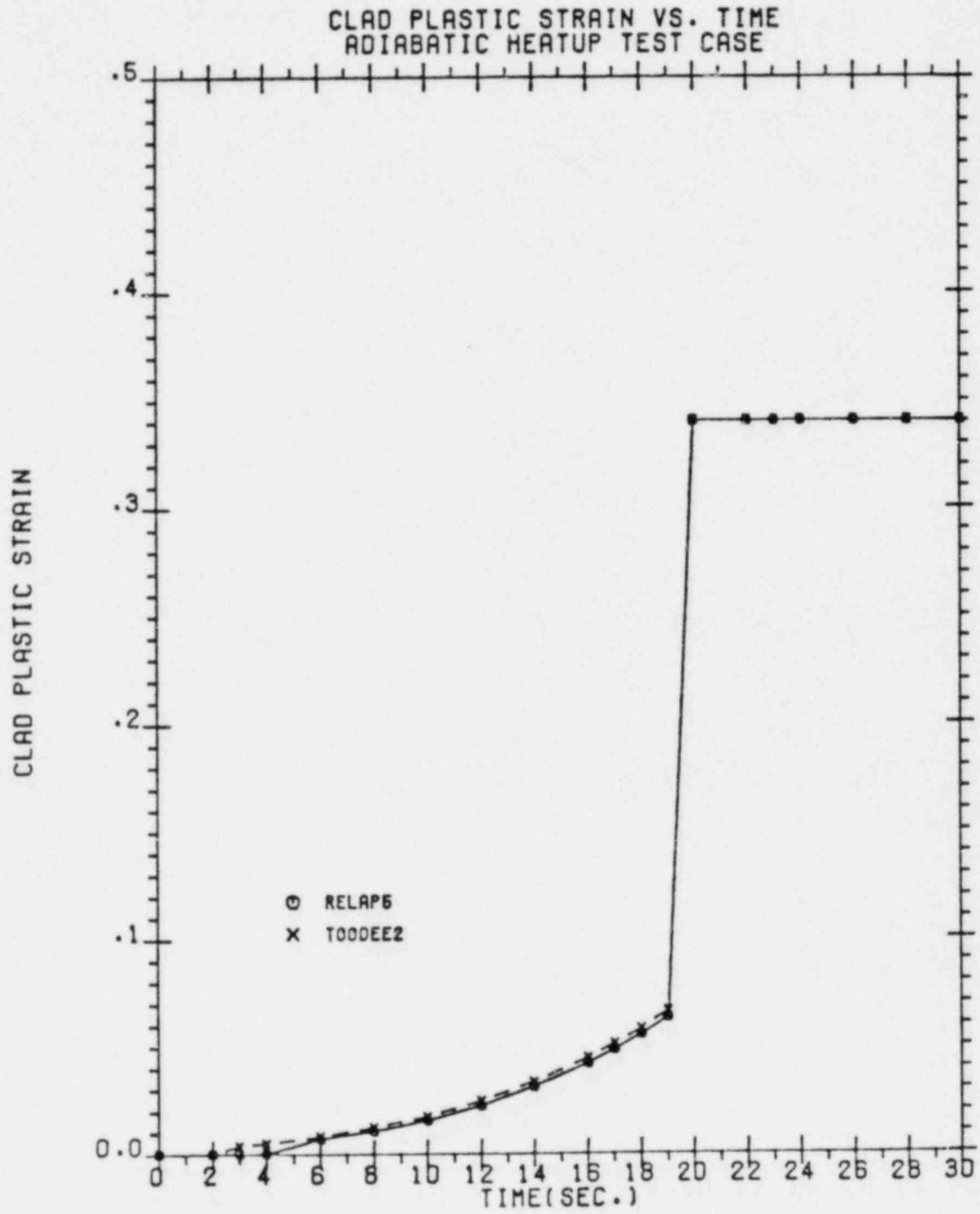




FIGURE 5.4

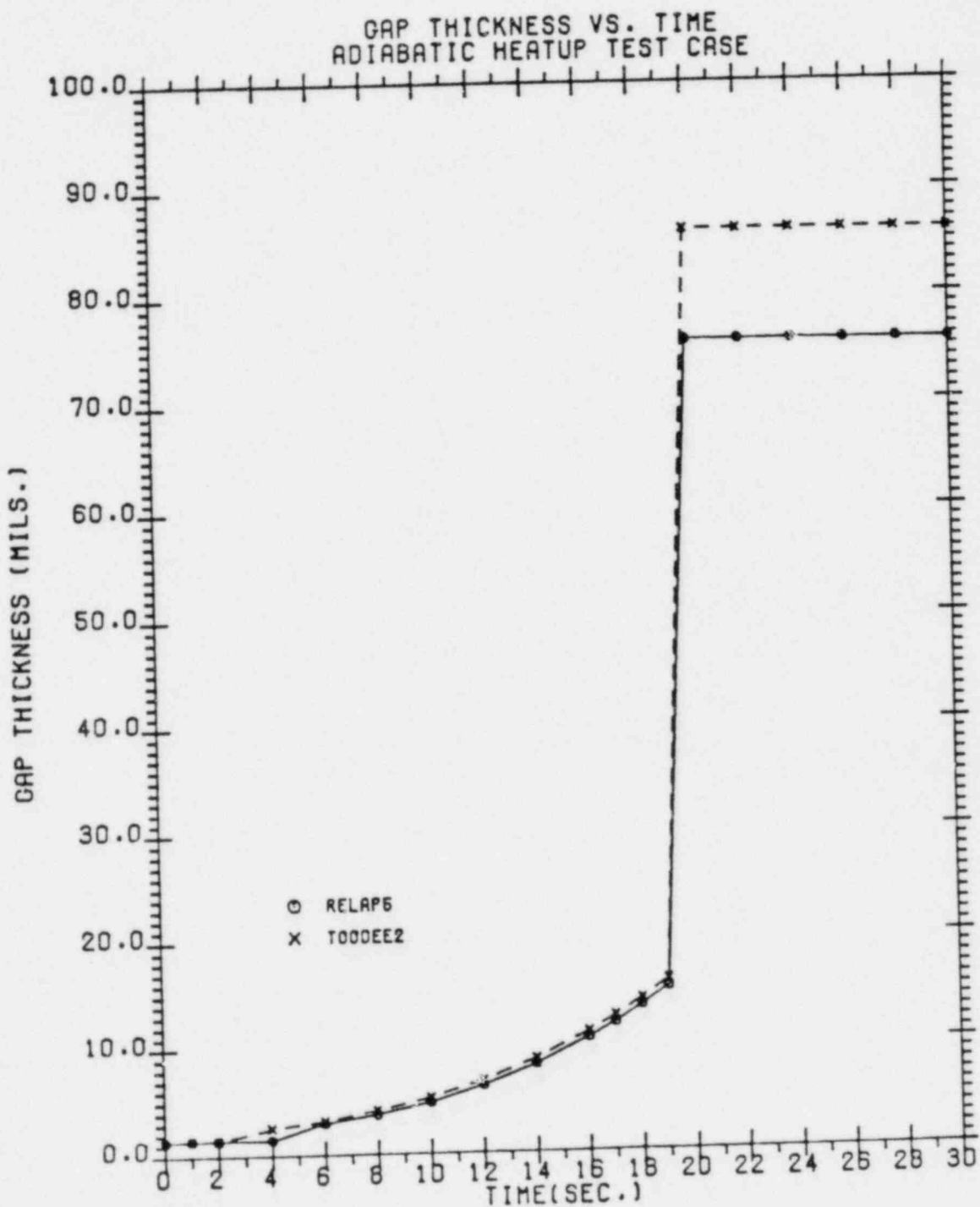


FIGURE 5.6

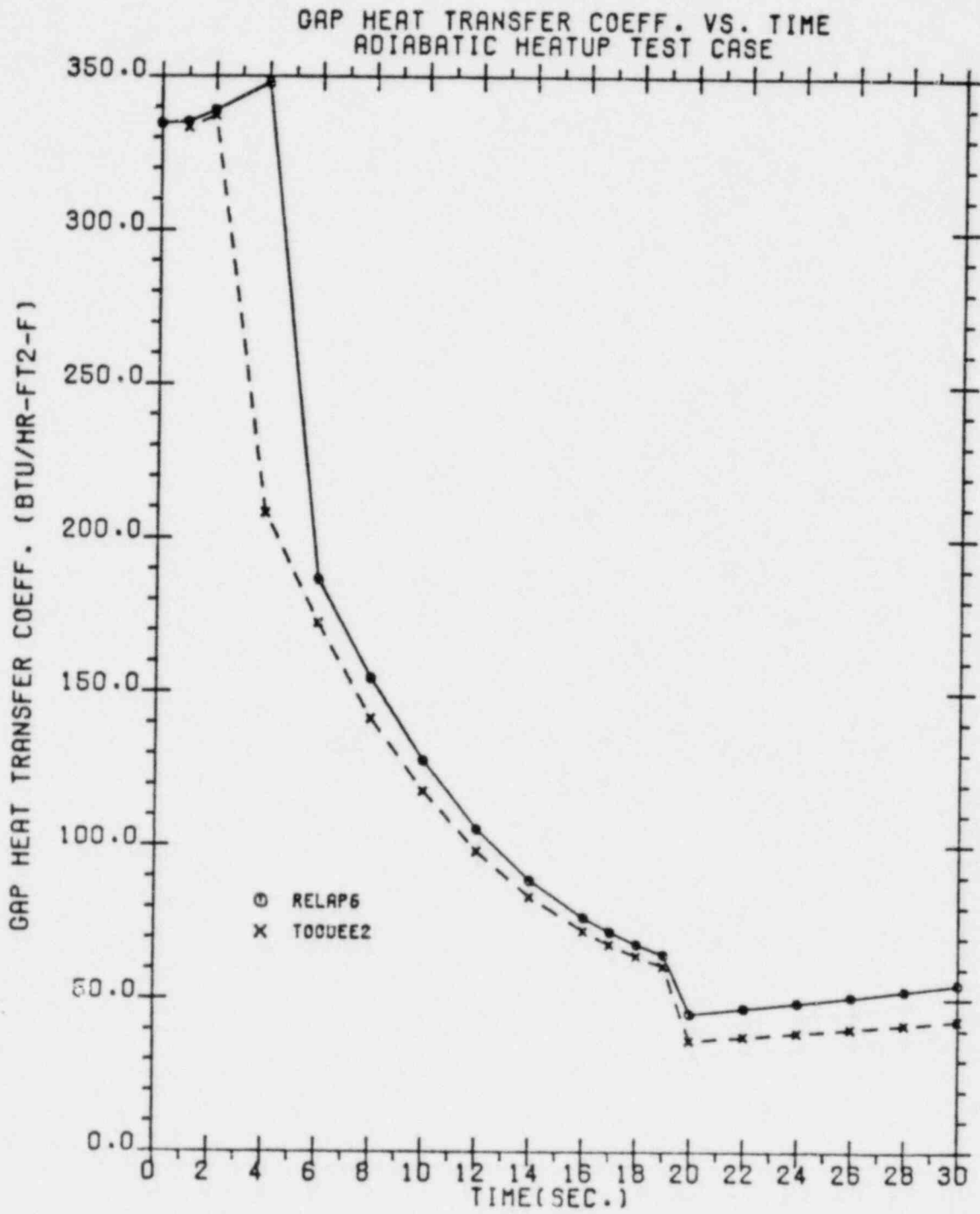


FIGURE 5.6

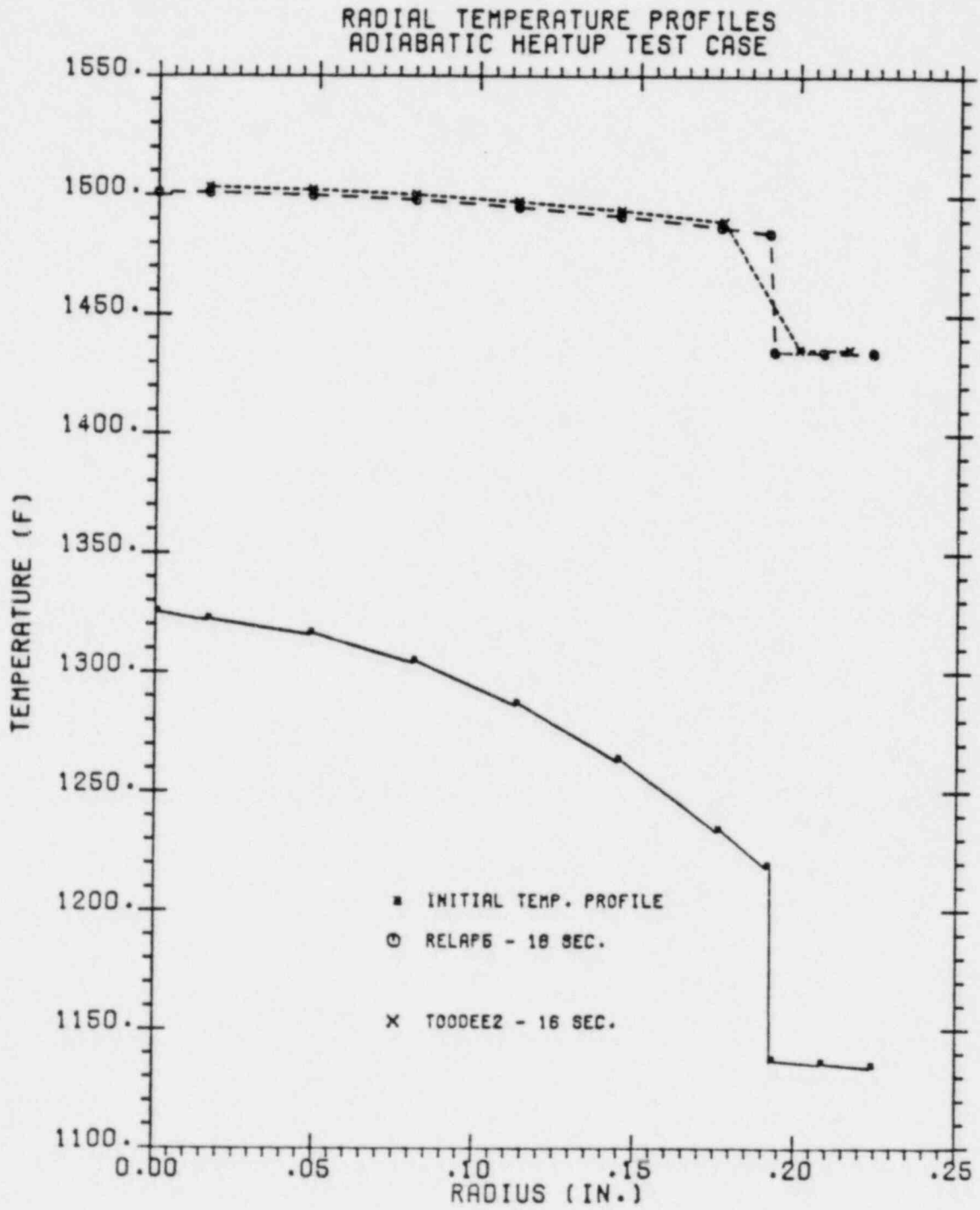


FIGURE 5.7

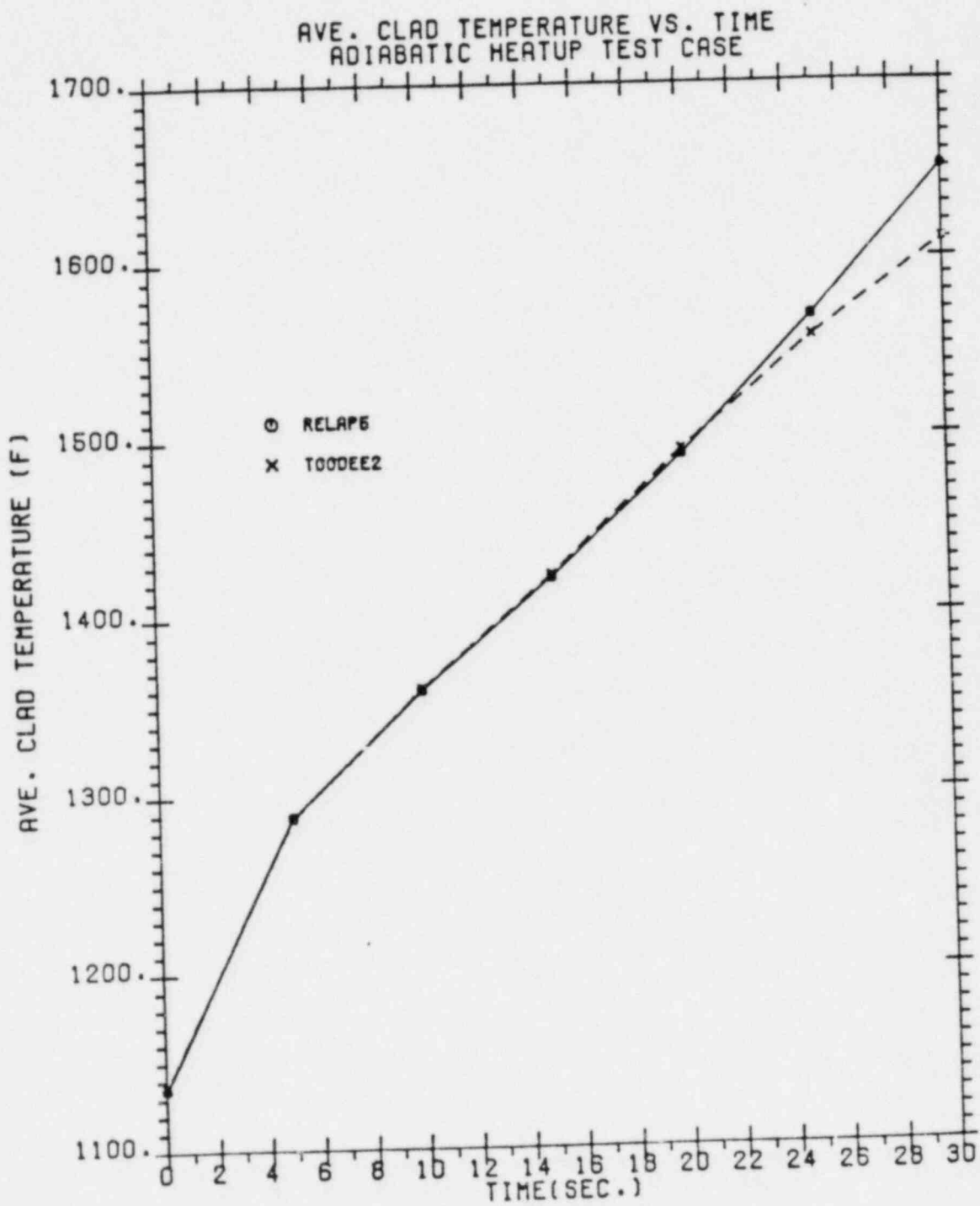


FIGURE 5.8

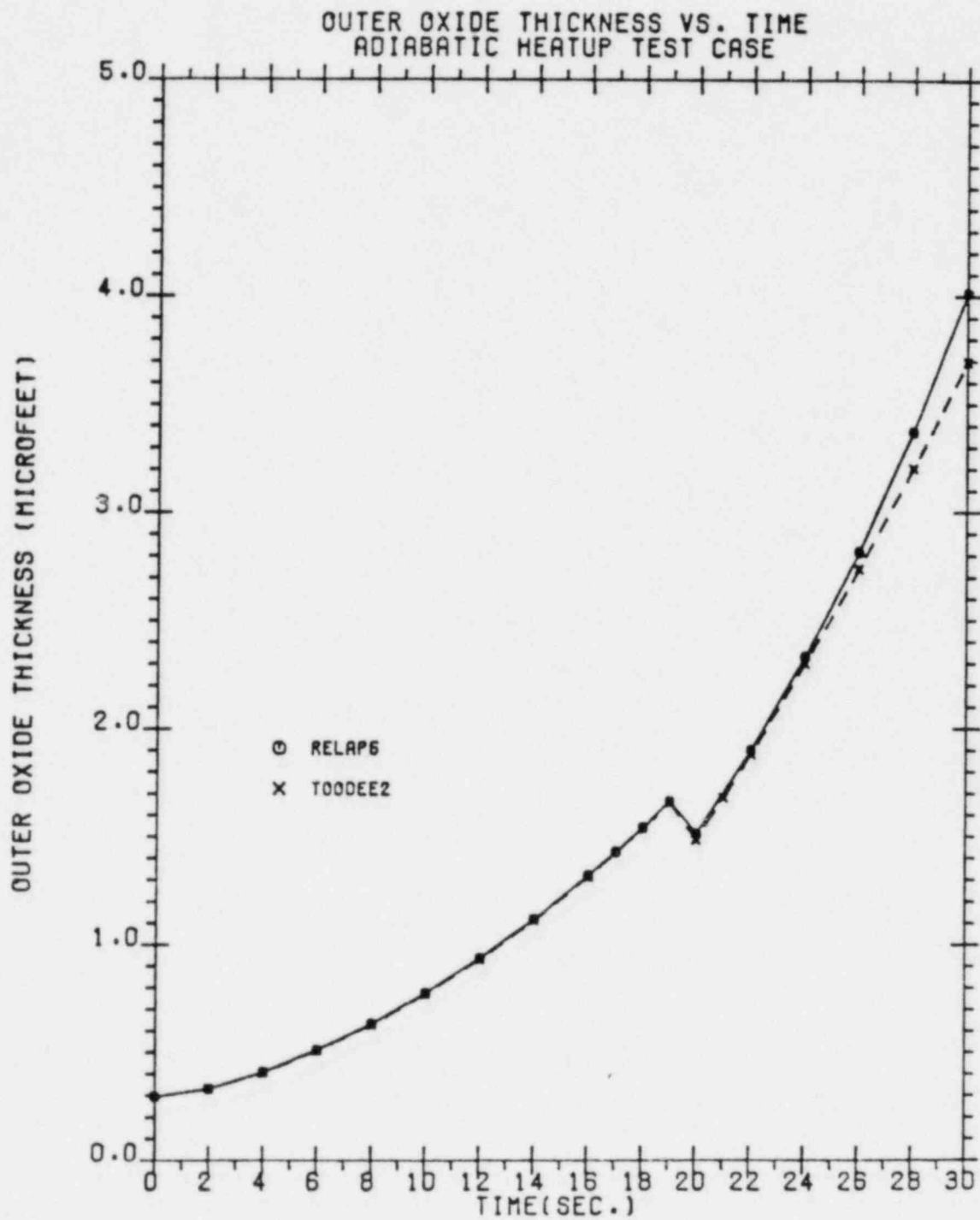


FIGURE 5.9

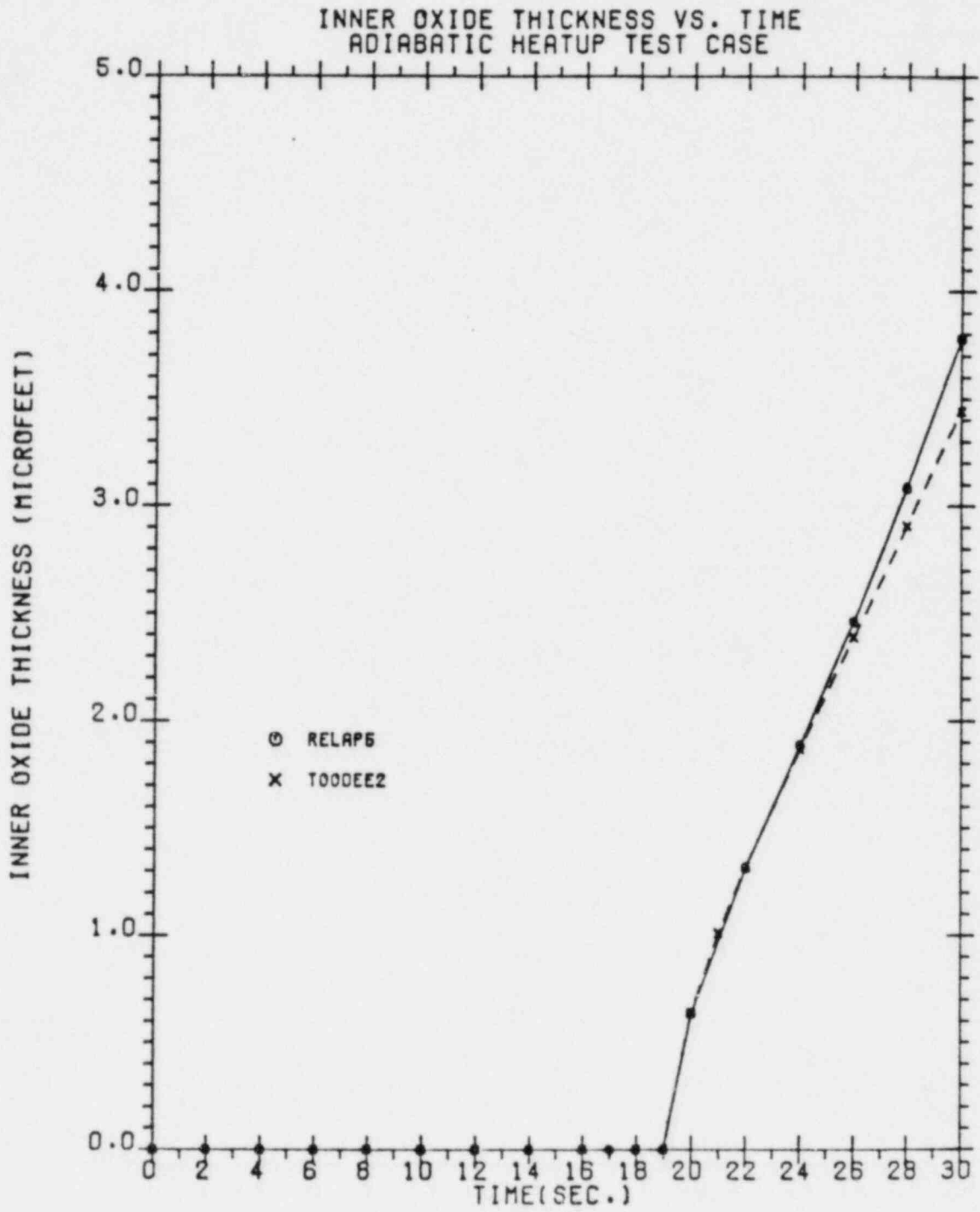
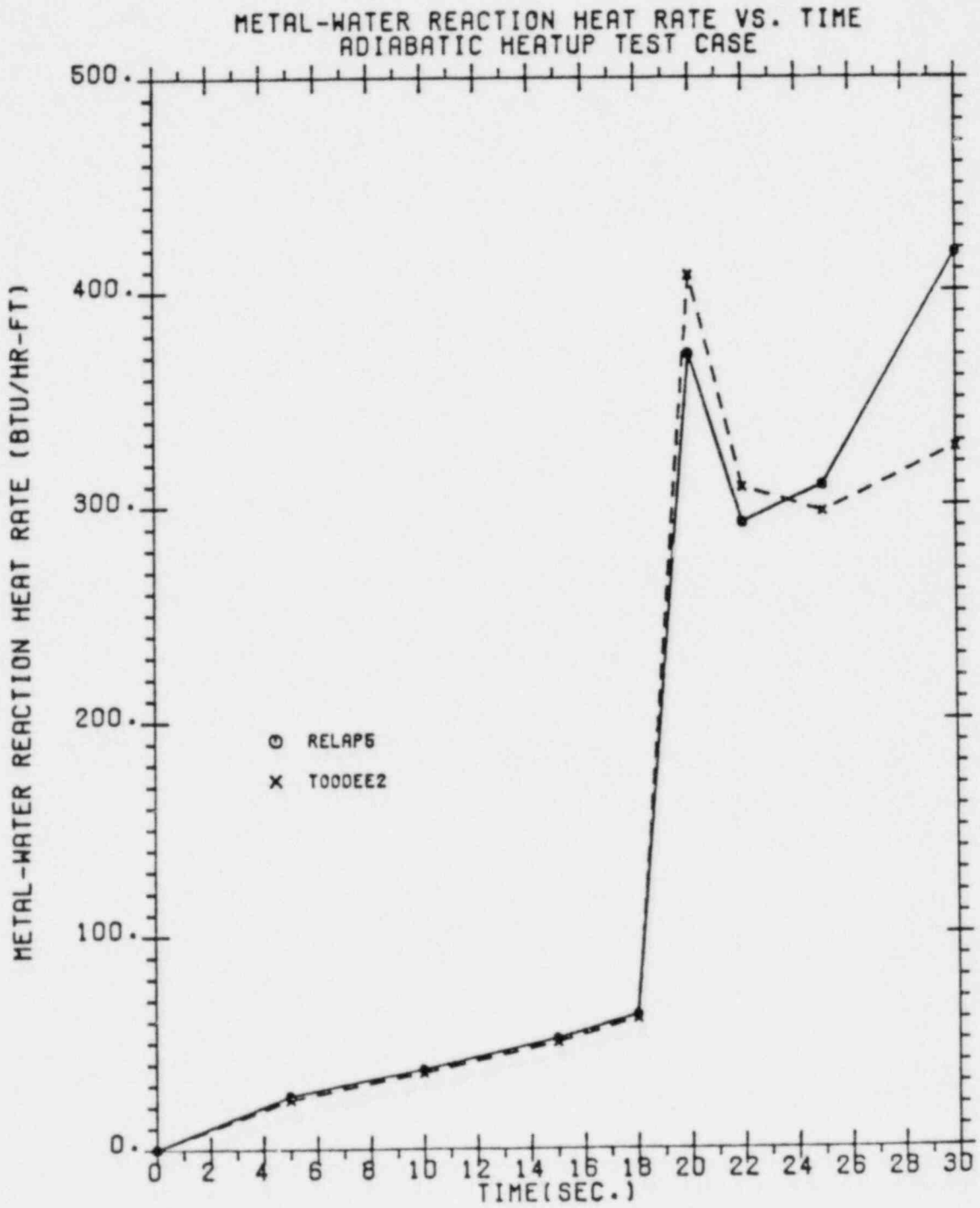


FIGURE 5.10



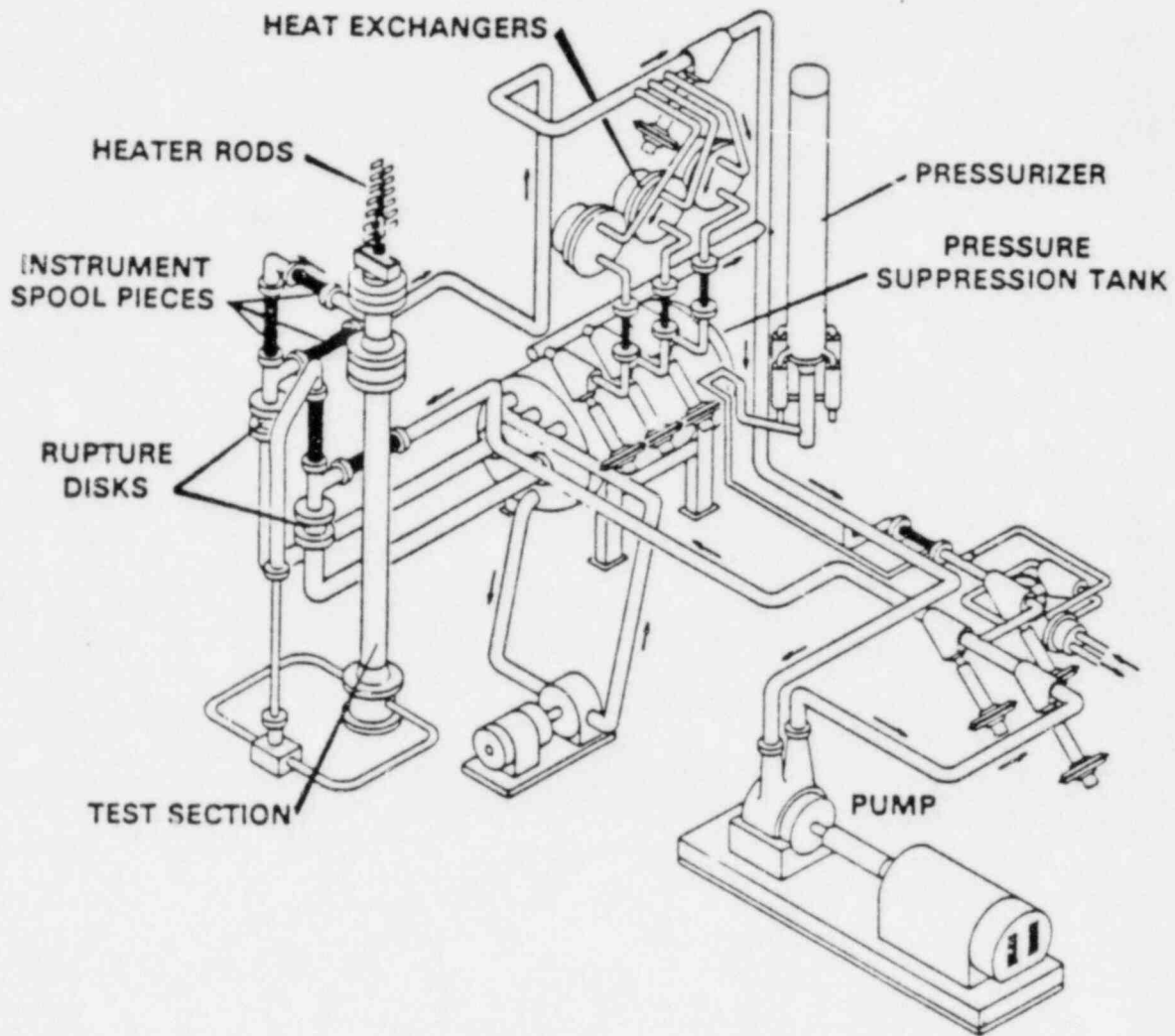
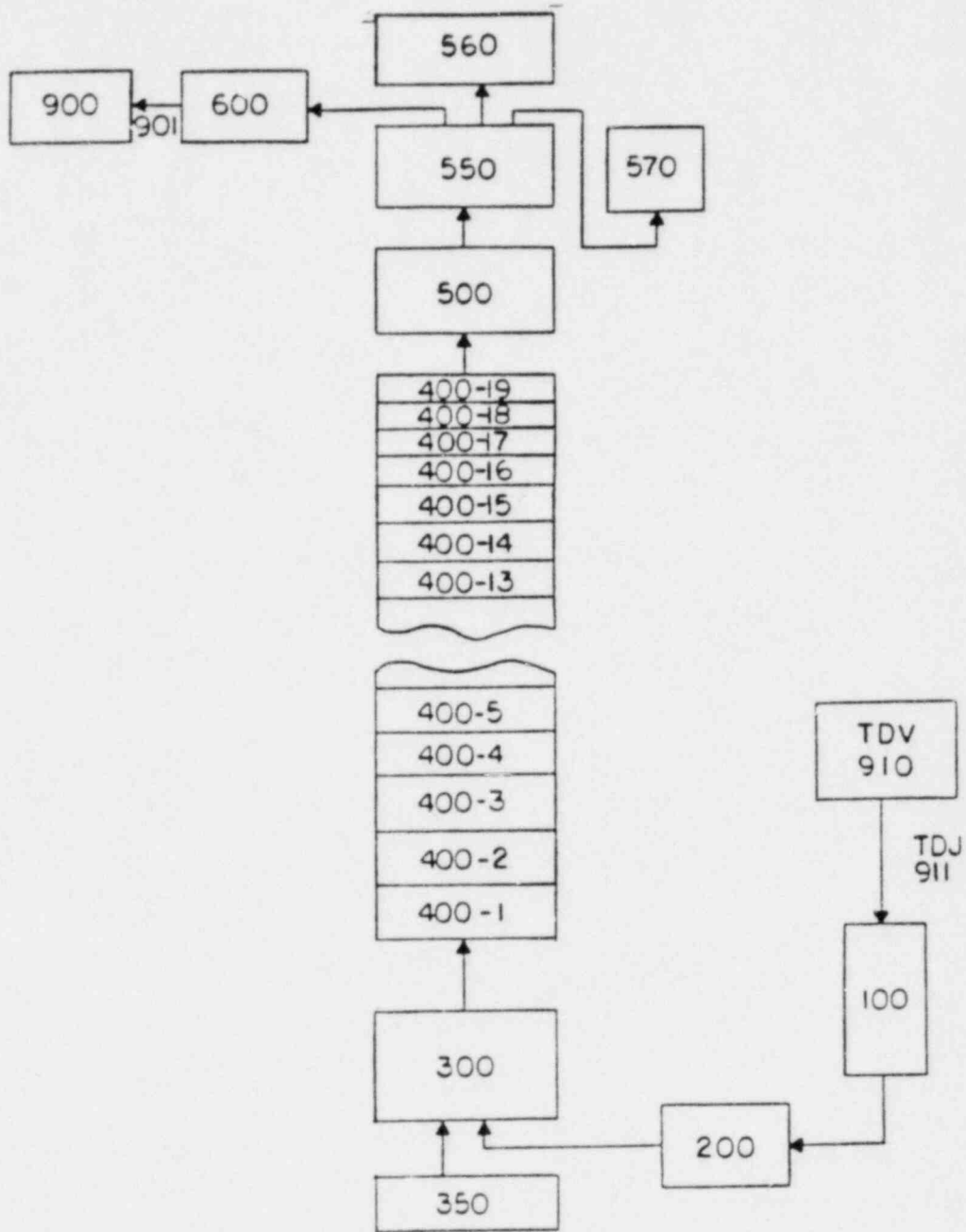


Fig. 2.1. THF in standard configuration.



THTF-RELAP5YA TEST MATRIX

<u>TEST NO.</u>	<u>TYPE OF TEST</u>
3.07.9B	Steady State Film Boiling
3.07.9K	Steady State Film Boiling
3.07.9X	Steady State Film Boiling
3.08.6C	Transient Film Boiling
3.09.10I	Quasi-Steady State Boiloff Test
3.09.100	Reflood Test
3.09.10Q	Reflood Test

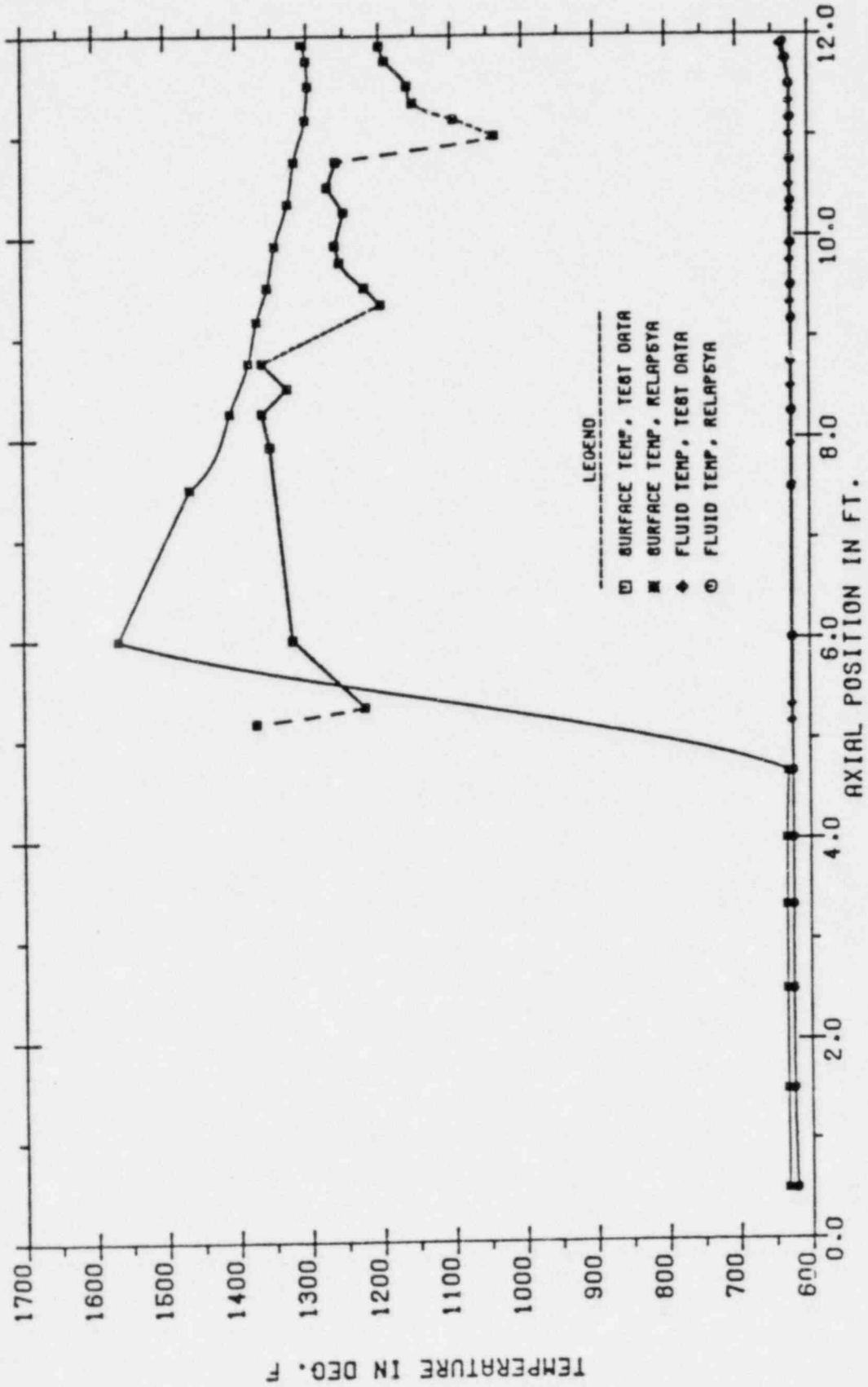


RELAP 5YA MODEL OF THTF TEST SECTION

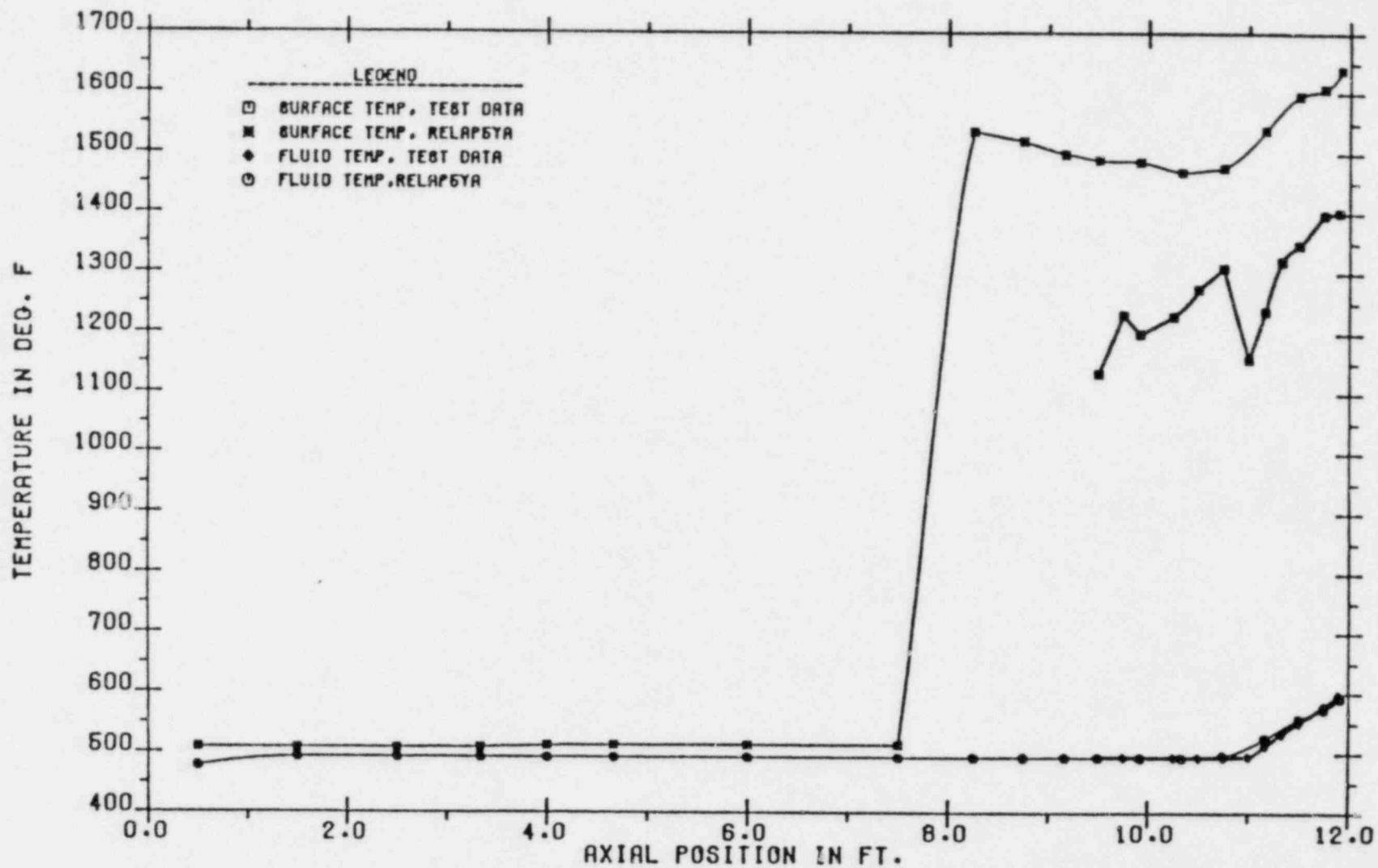
THTF STEADY STATE UP-FLOW  
FILM BOILING TESTS

<u>TEST NO.</u>	<u>PRESSURE (psia)</u>	<u>MASS FLUX (lbm/hr-ft<sup>2</sup>)</u>	<u>HEAT FLUX (Btu/hr-ft<sup>2</sup>)</u>
3.07.9B	1849	5.25.10 <sup>5</sup>	2.5.10 <sup>5</sup>
3.07.9K	635	1.66.10 <sup>5</sup>	1.4.10 <sup>5</sup>
3.07.9 X	872	2.50.10 <sup>5</sup>	1.9.10 <sup>5</sup>

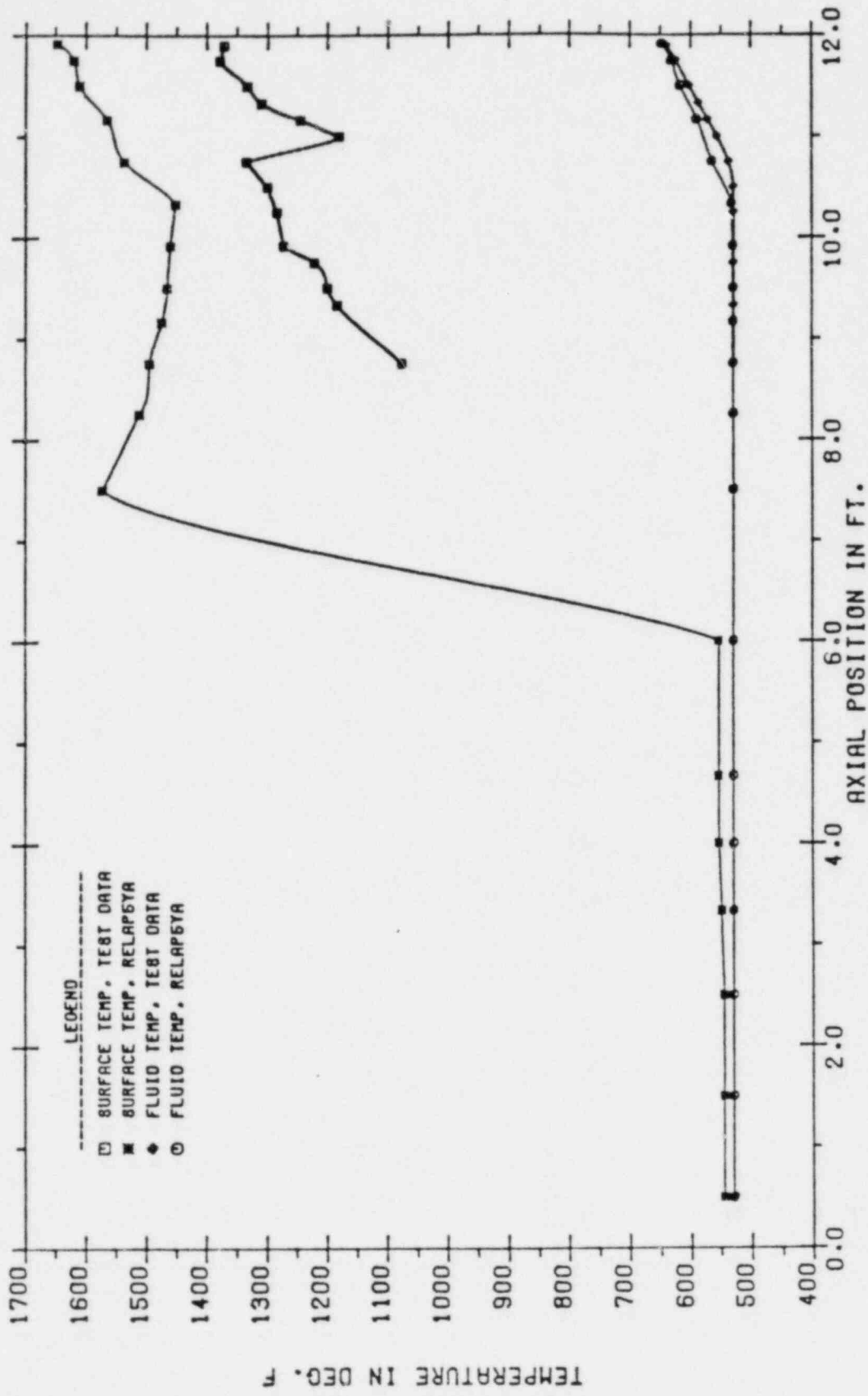
# AXIAL TEMPERATURE PROFILE FOR TEST 3.07.98



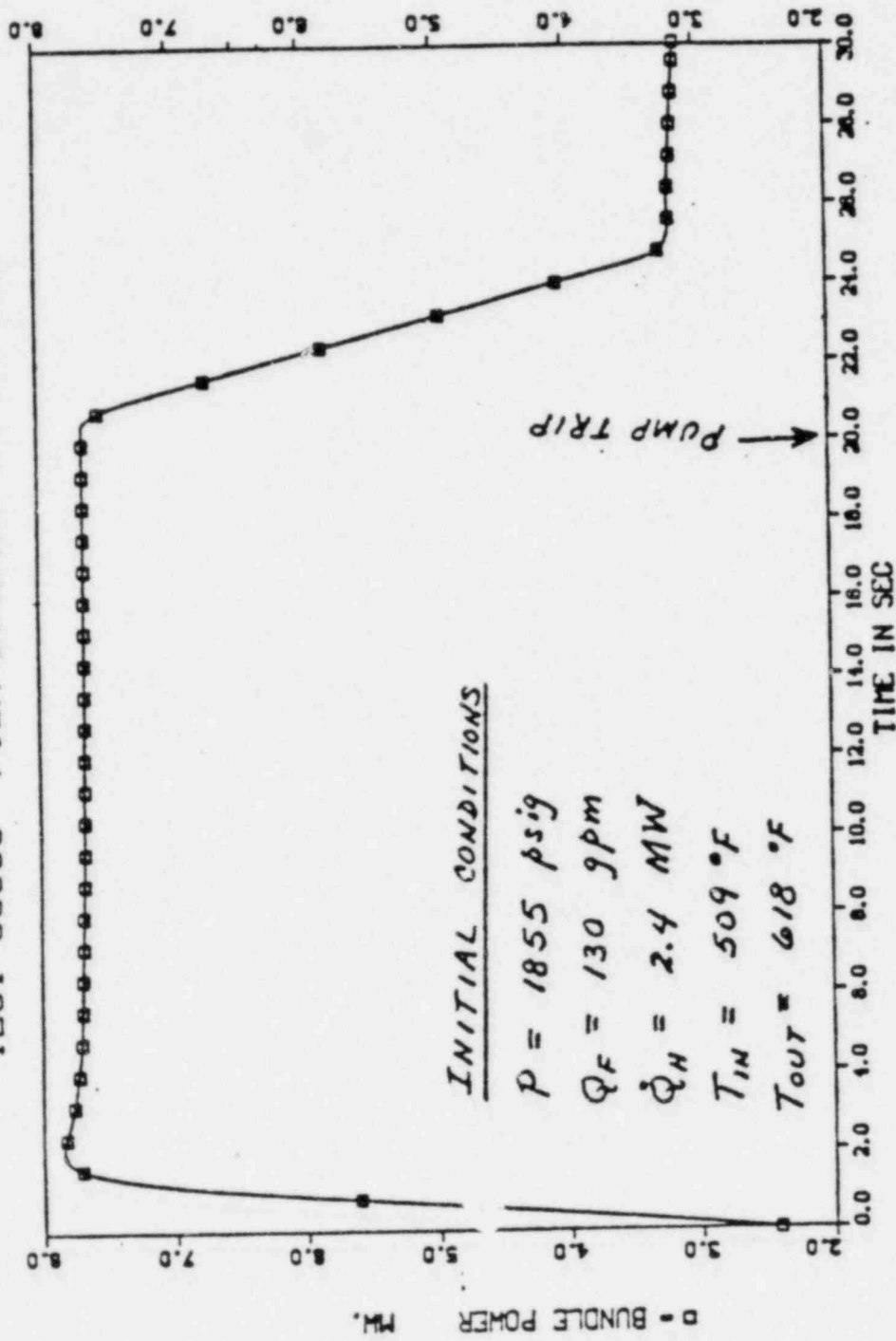
# AXIAL TEMPERATURE PROFILE FOR TEST 3.07.9K



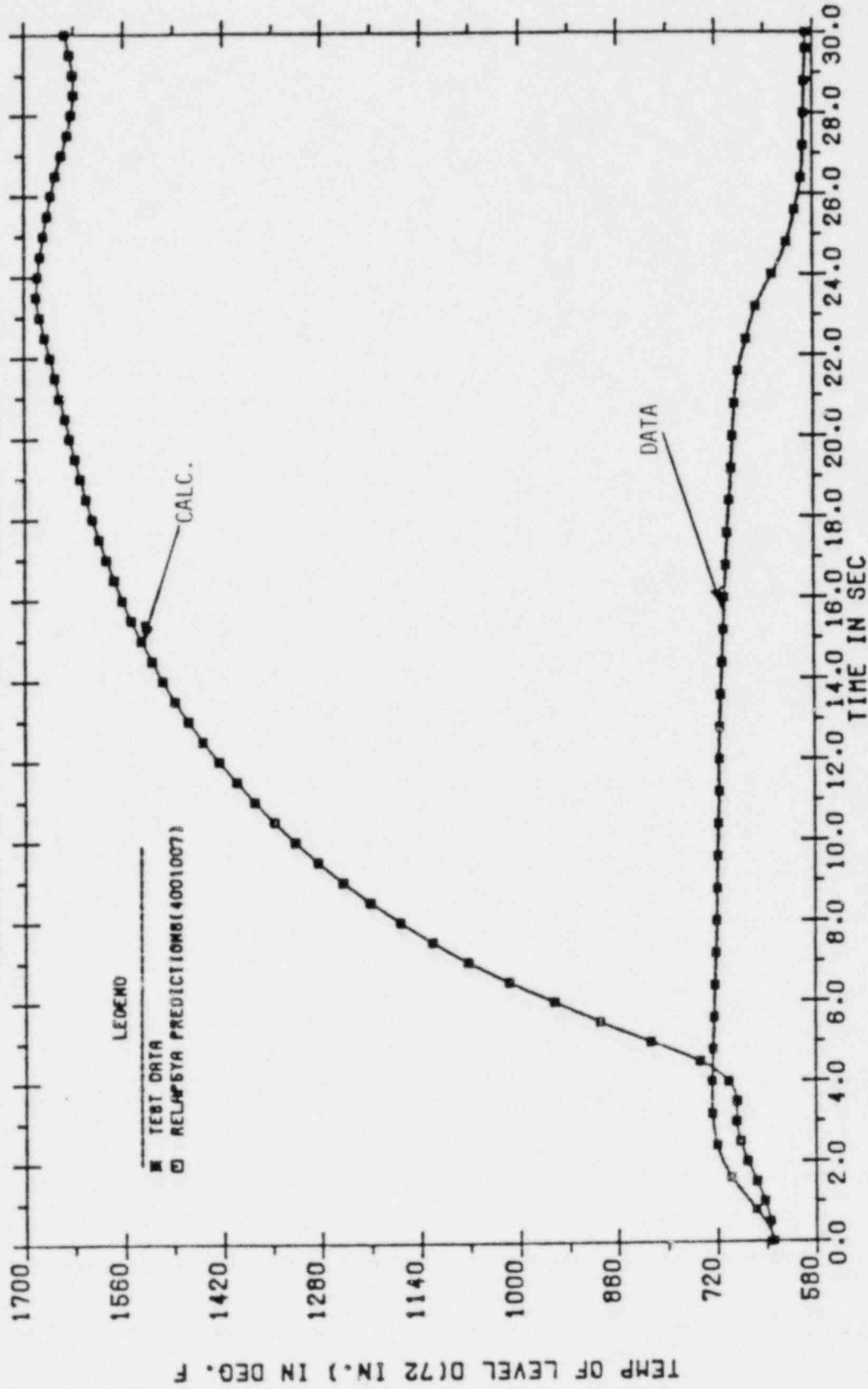
# AXIAL TEMPERATURE PROFILE FOR TEST 3.07.9X



# TEST 3086C -FILM BOILING IN UPFLOW

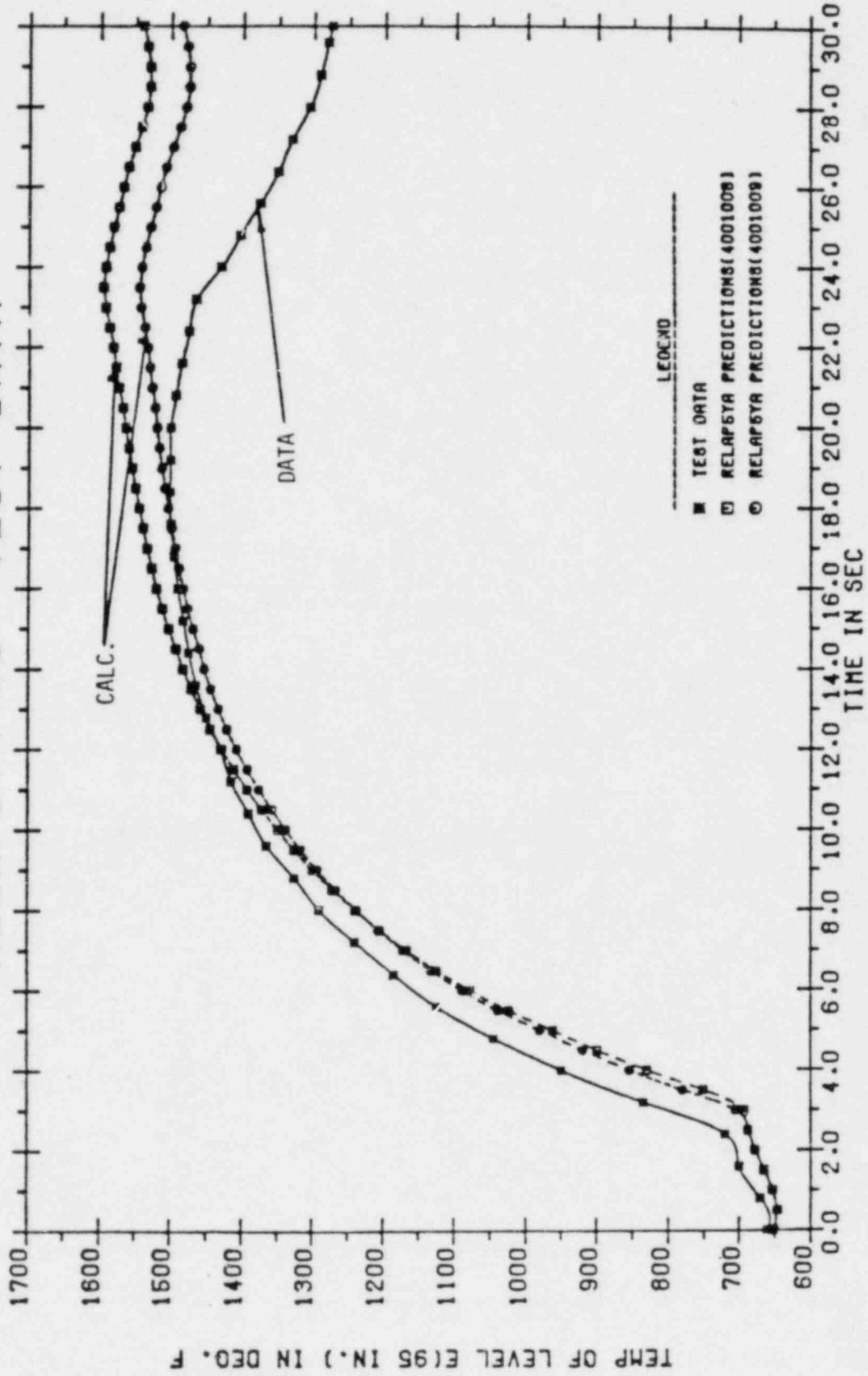


# FILM BOILING TEST 3086C RELAP5YA VS. TEST DATA

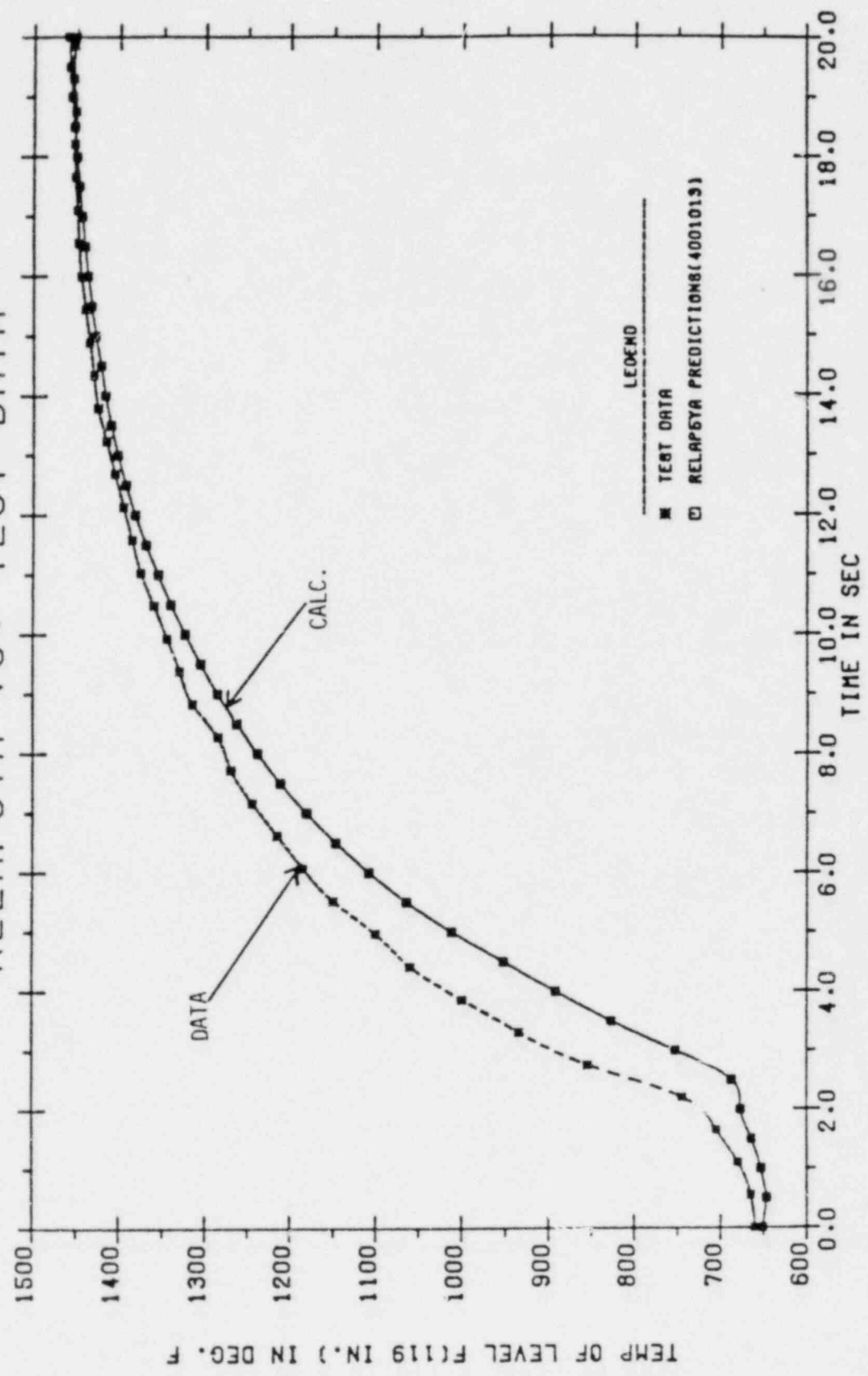




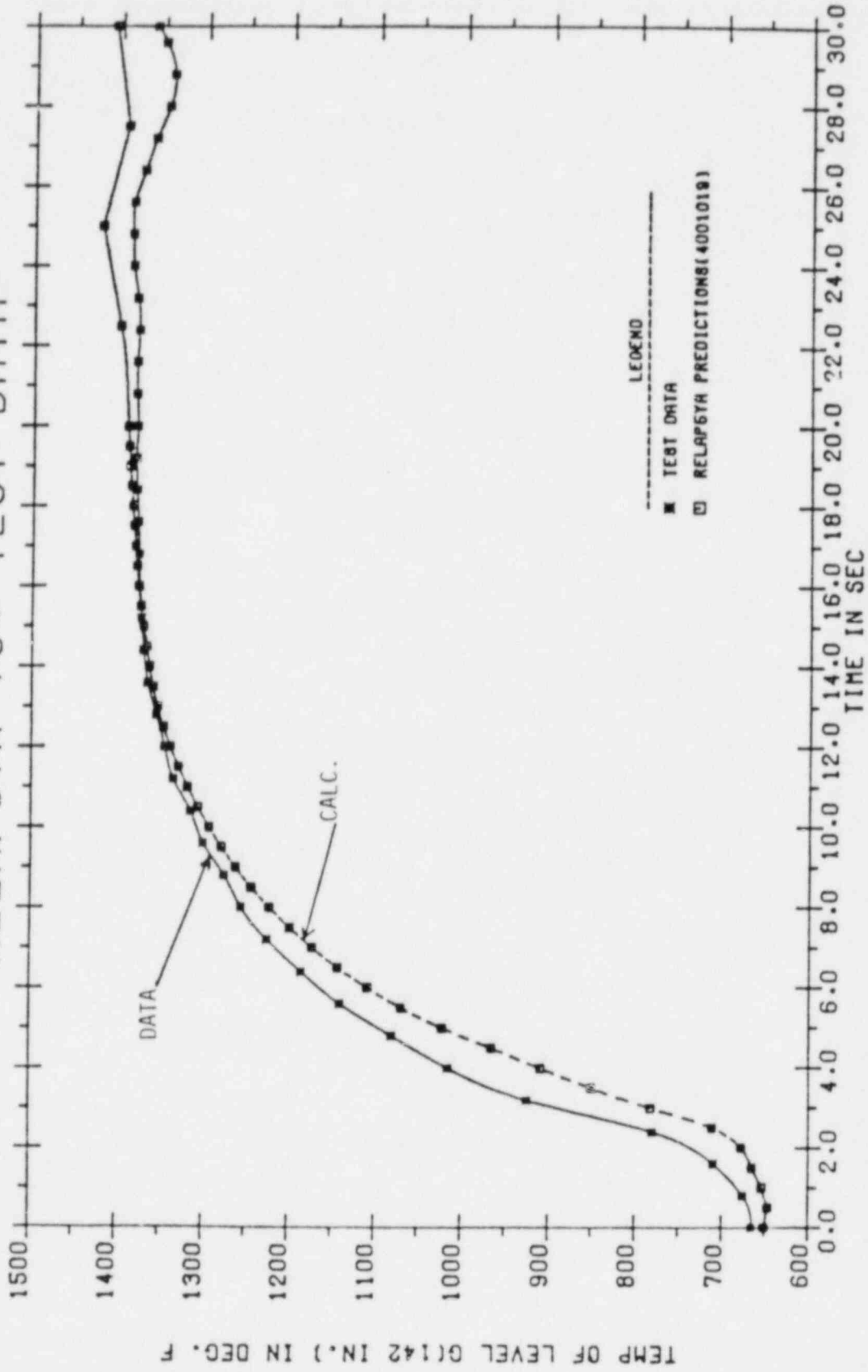
# TEST 3086C -FILM BOILING IN UPFLOW RELAP5YA VS. TEST DATA



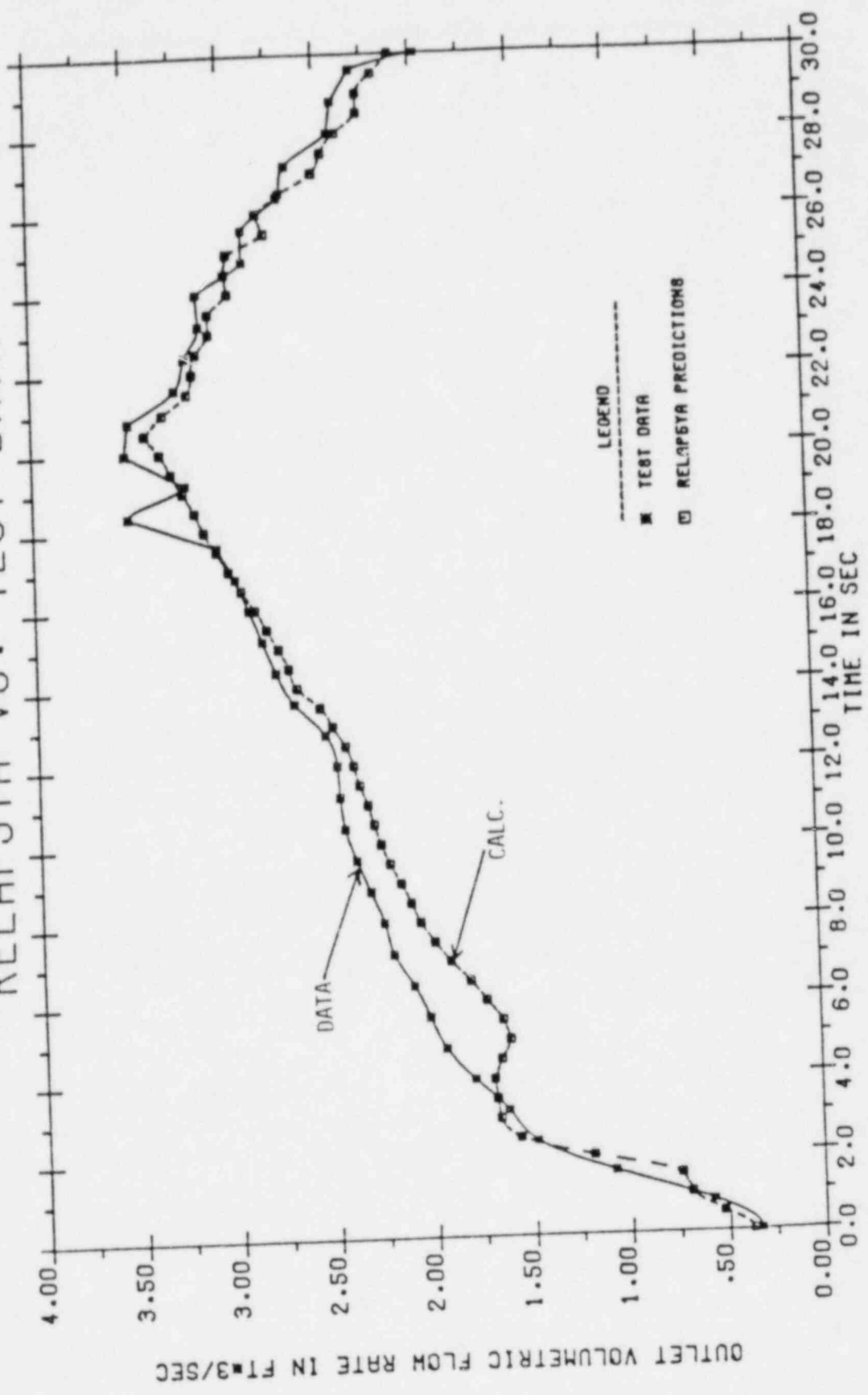
# TEST 3086C -FILM BOILING IN UPFLOW RELAP5YA VS. TEST DATA



# TEST 3086C - FILM BOILING IN UPFLOW RELAP5YA VS. TEST DATA



# TEST 3086C -FILM BOILING IN UPFLOW RELAP5YA VS. TEST DATA

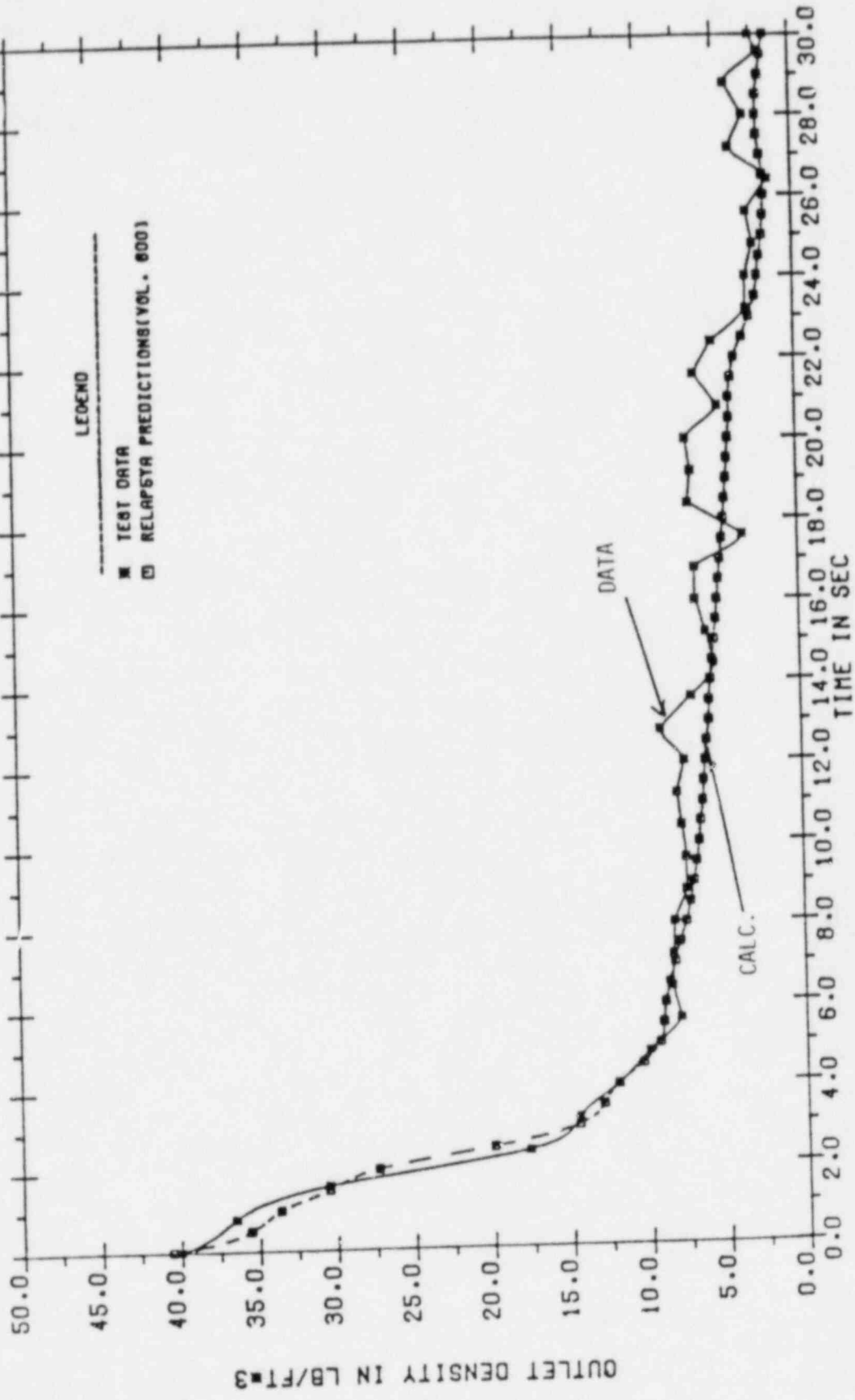


OUTLET VOLUMETRIC FLOW RATE IN FT³/SEC

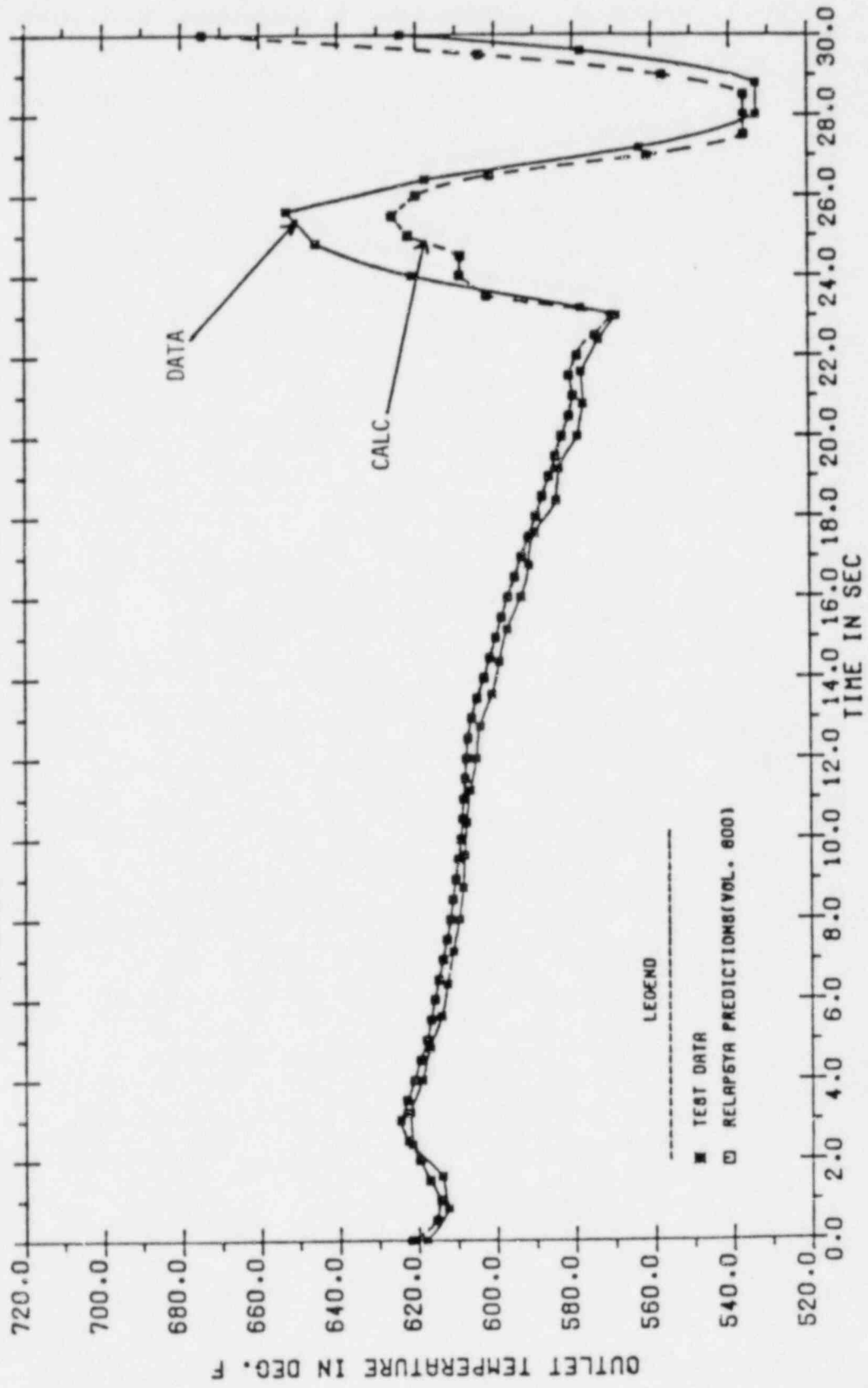
LEGEND  
 ■ TEST DATA  
 □ RELAP5YA PREDICTIONS

DATA  
 CALC.

# FILM BOILING TEST 3086C RELAP5YA VS. TEST DATA



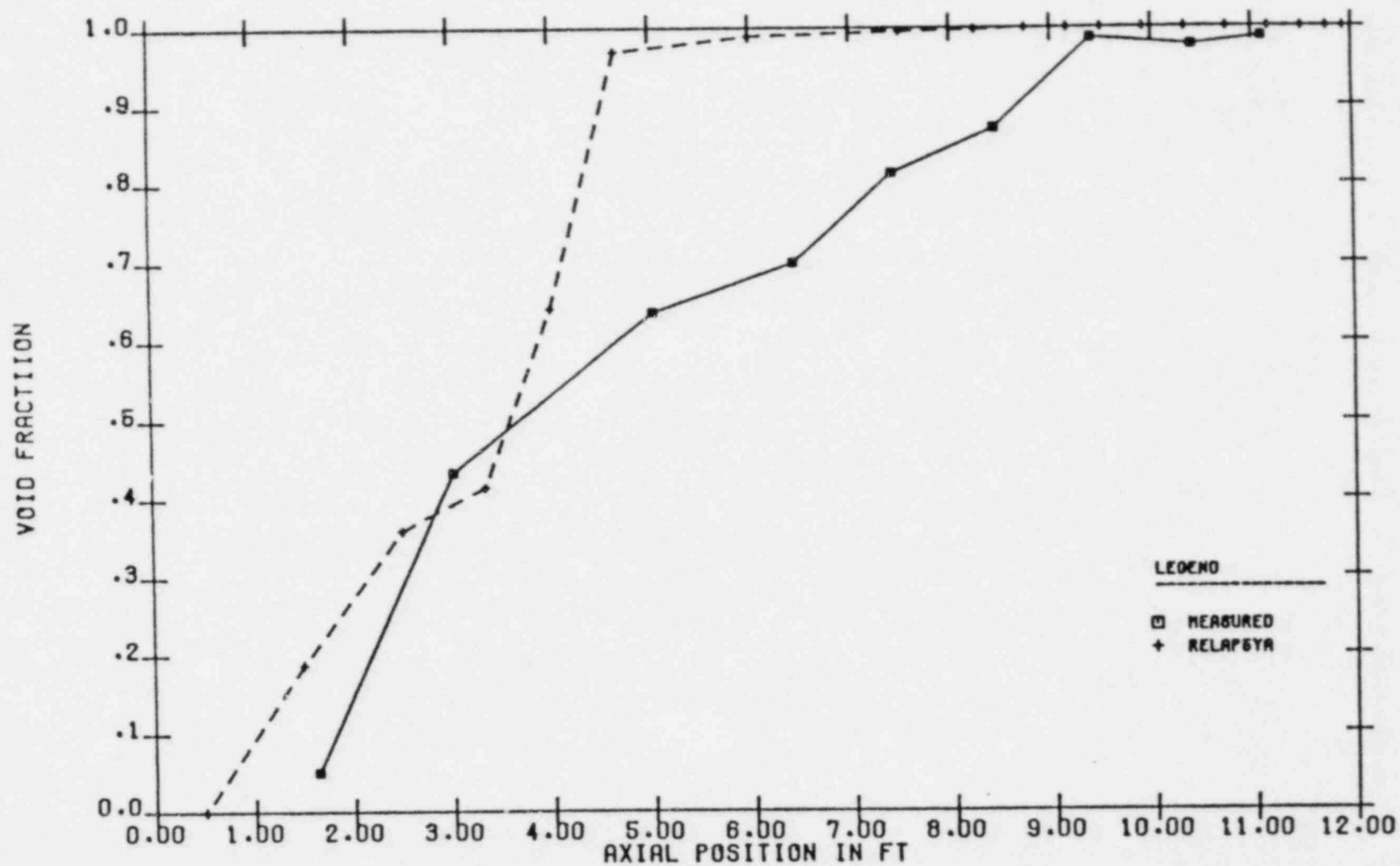
# FILM BOILING TEST 3086C RELAP5YA VS. TEST DATA



QUASI STEADY STATE BOILOFF TEST

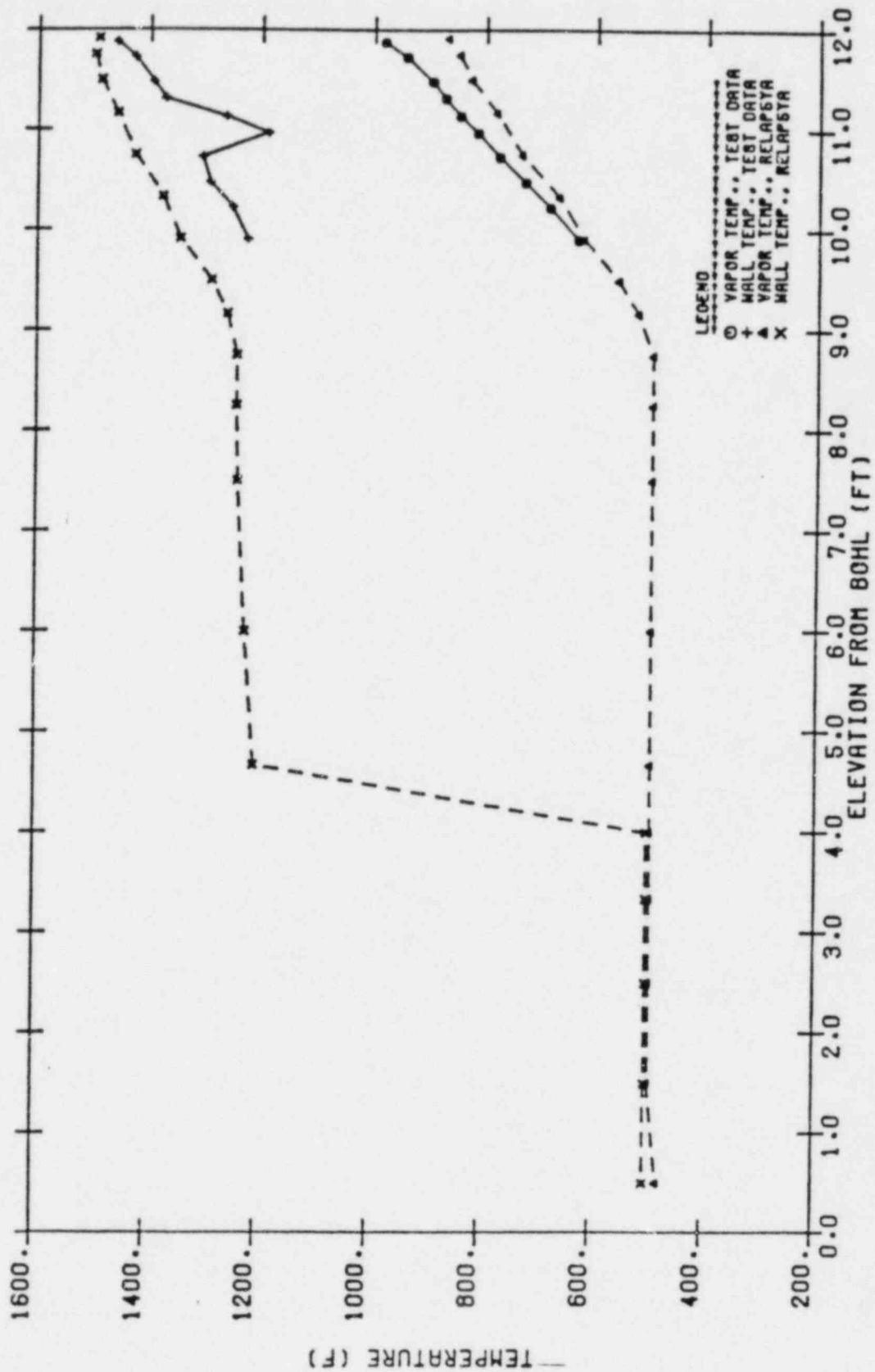
<u>Test No</u>	<u>Pressure (psia)</u>	<u>Mass Flux (lbm/hr-ft<sup>2</sup>)</u>	<u>Linear Power (kw/ft)</u>
3.09.10I	650	2.19.10 <sup>4</sup>	0.68

# VOID FRACTION PROFILE, TEST 3.09.10I





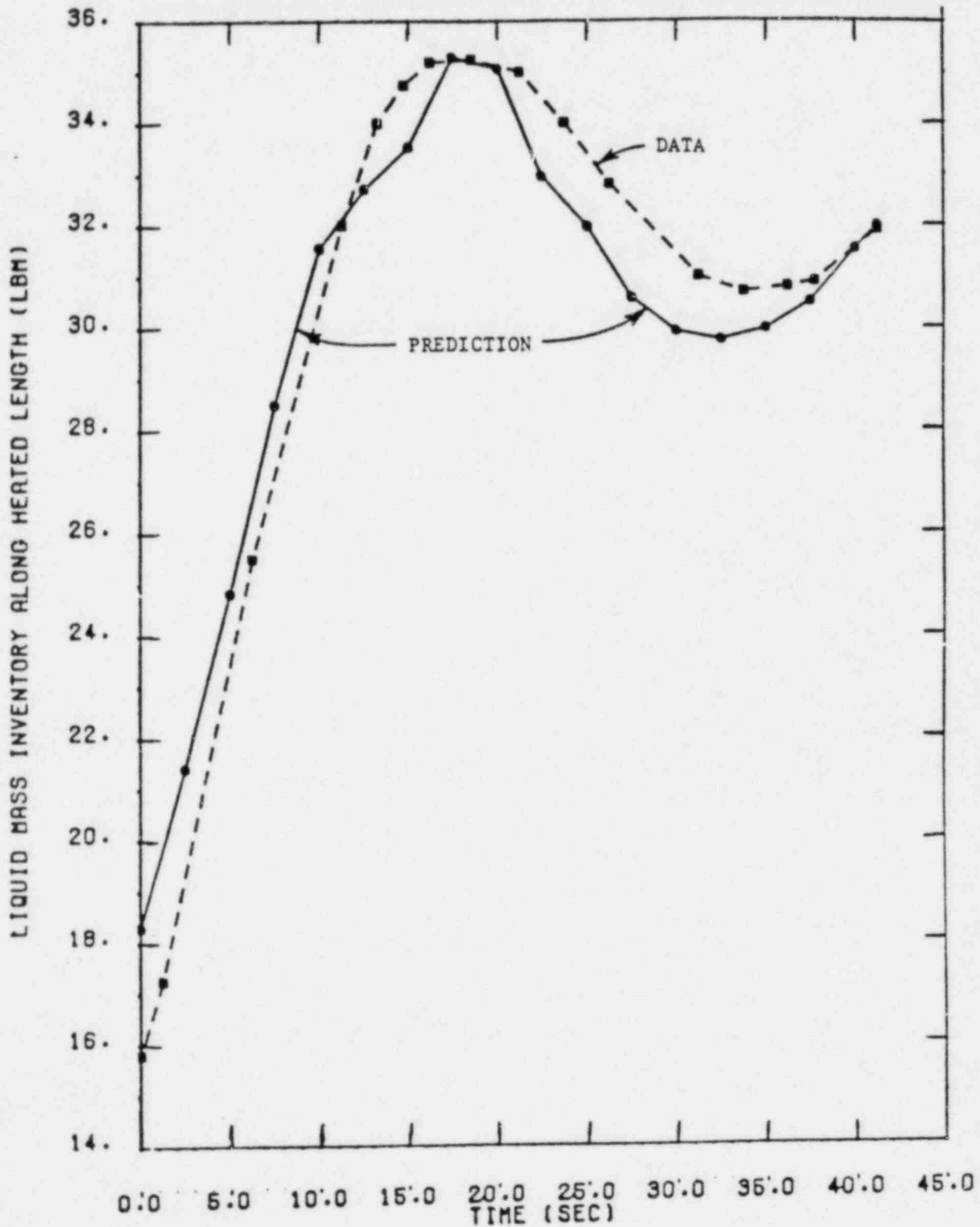
# TEMPERATURE PROFILE TEST 3.09.10I



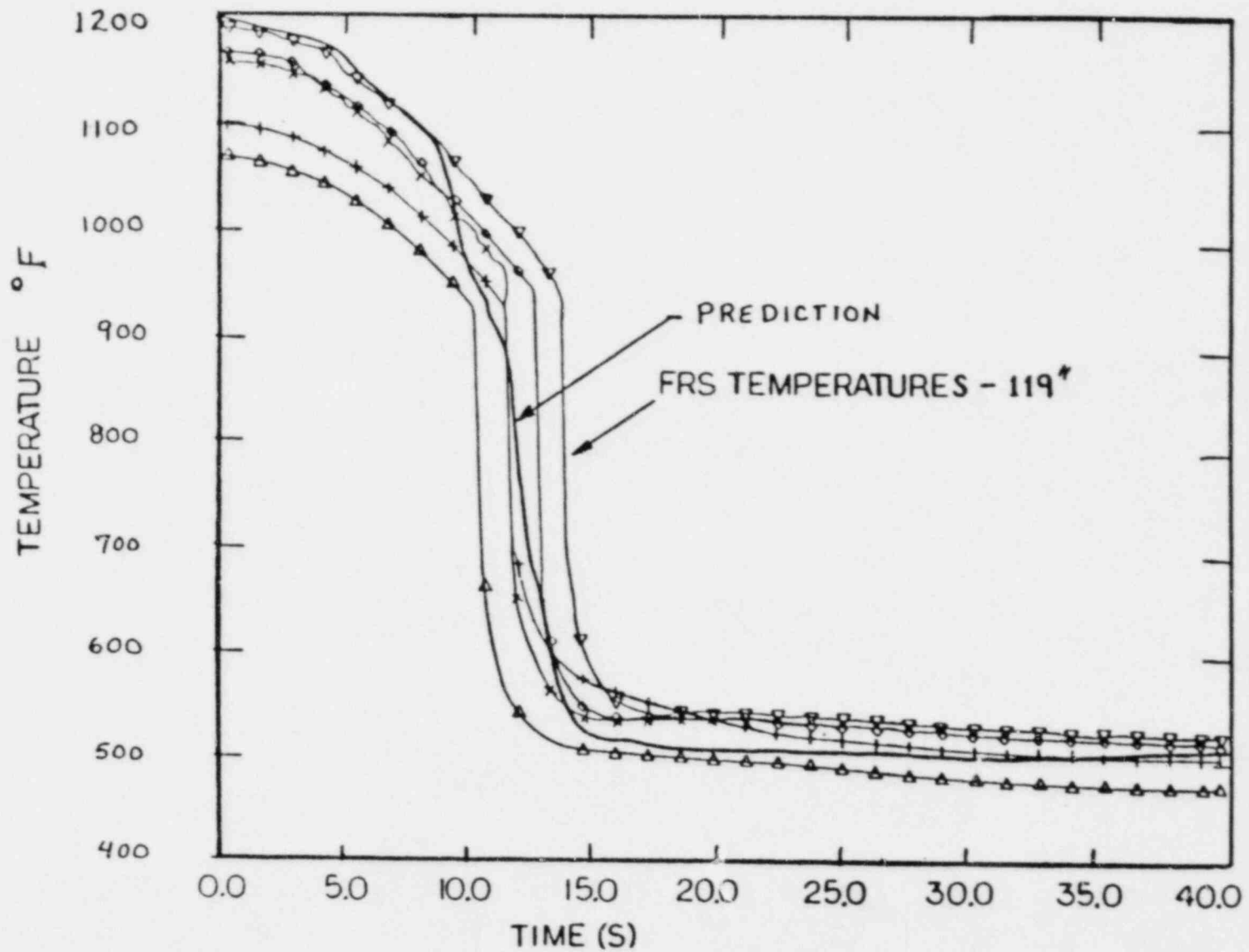
REFLOOD TESTS

<u>TEST NO.</u>	<u>INITIAL PRESSURE (psia)</u>	<u>FLOODING VELOCITY (in/sec)</u>	<u>LINEAR POWER (kw/ft)</u>
3.09.100	560	4.8	0.62
3.09.10Q	570	2.3	0.31

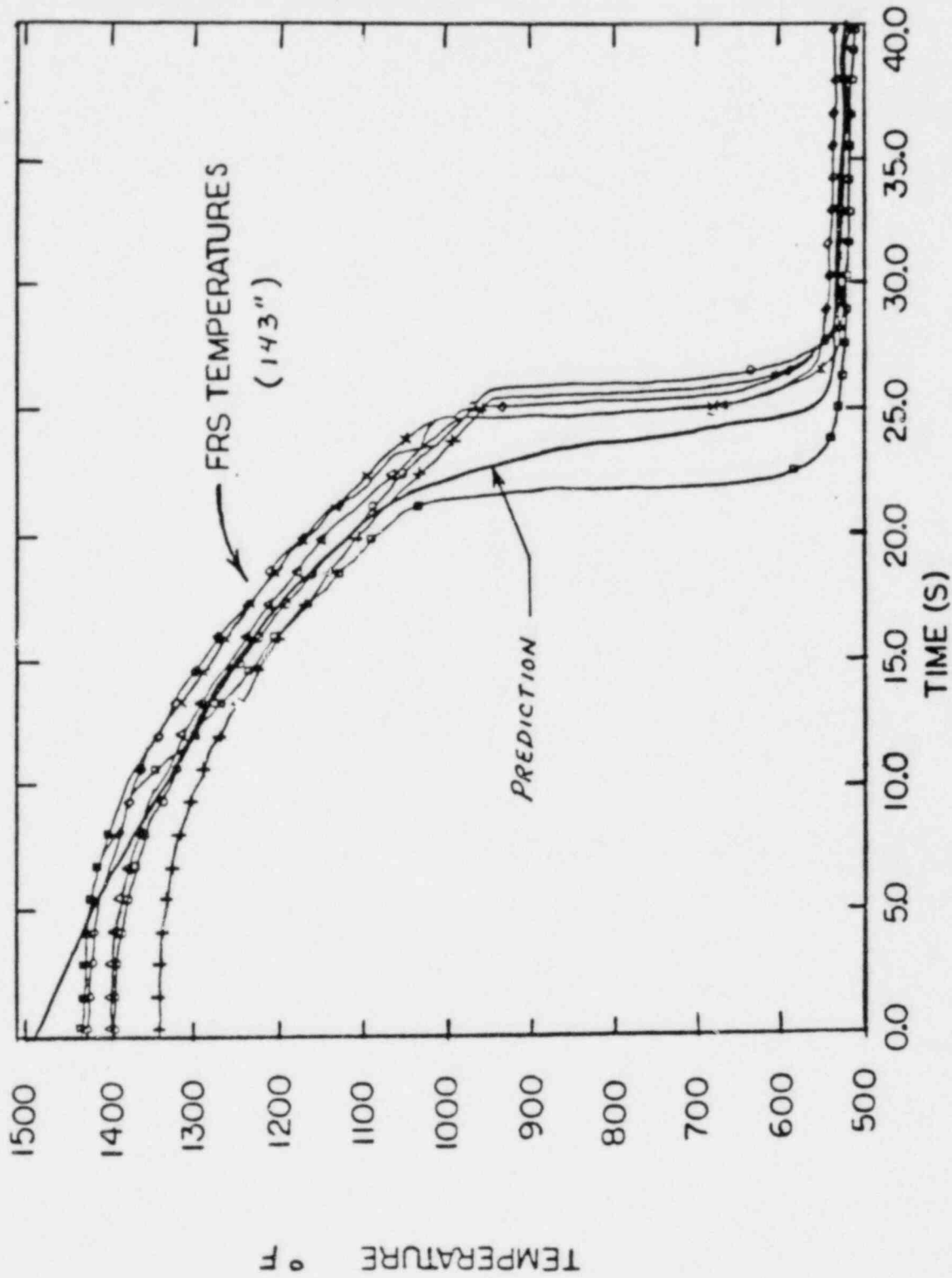
# THTF SBLOCA II REFLOOD TEST 3.09.100



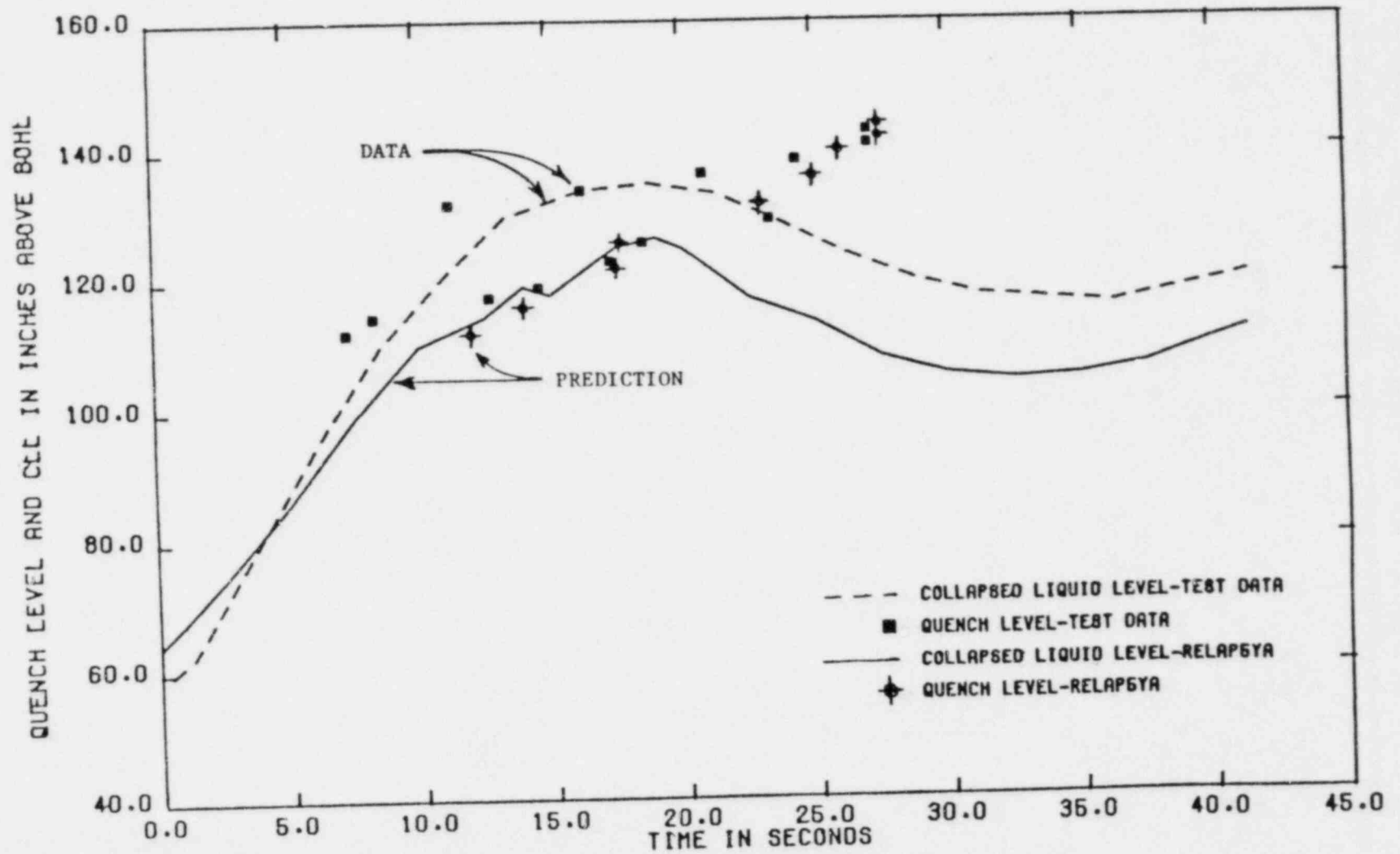
# THTF SBLOCA II: REFLOOD TEST 3.09.10φ



# THTF SBLOCA II REFLOOD TEST 3.09.10φ



# THTF SBLOCA II REFLOOD TEST 3.09.100









TLTA LARGE BREAK

TEST 6425/2

BE CALCULATION

(RUN L4-1)

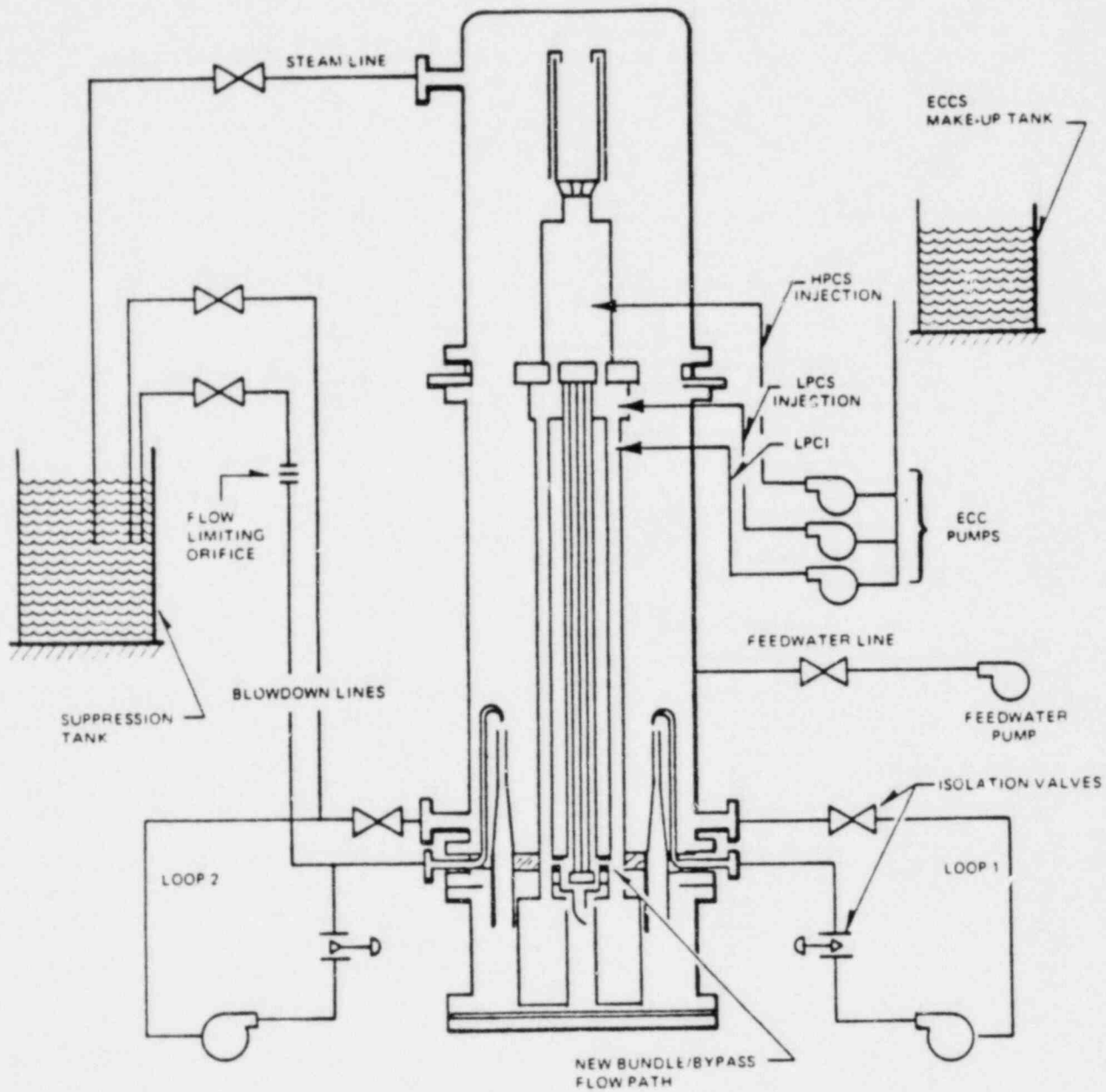


FIG. TLTA-5A CONFIGURATION

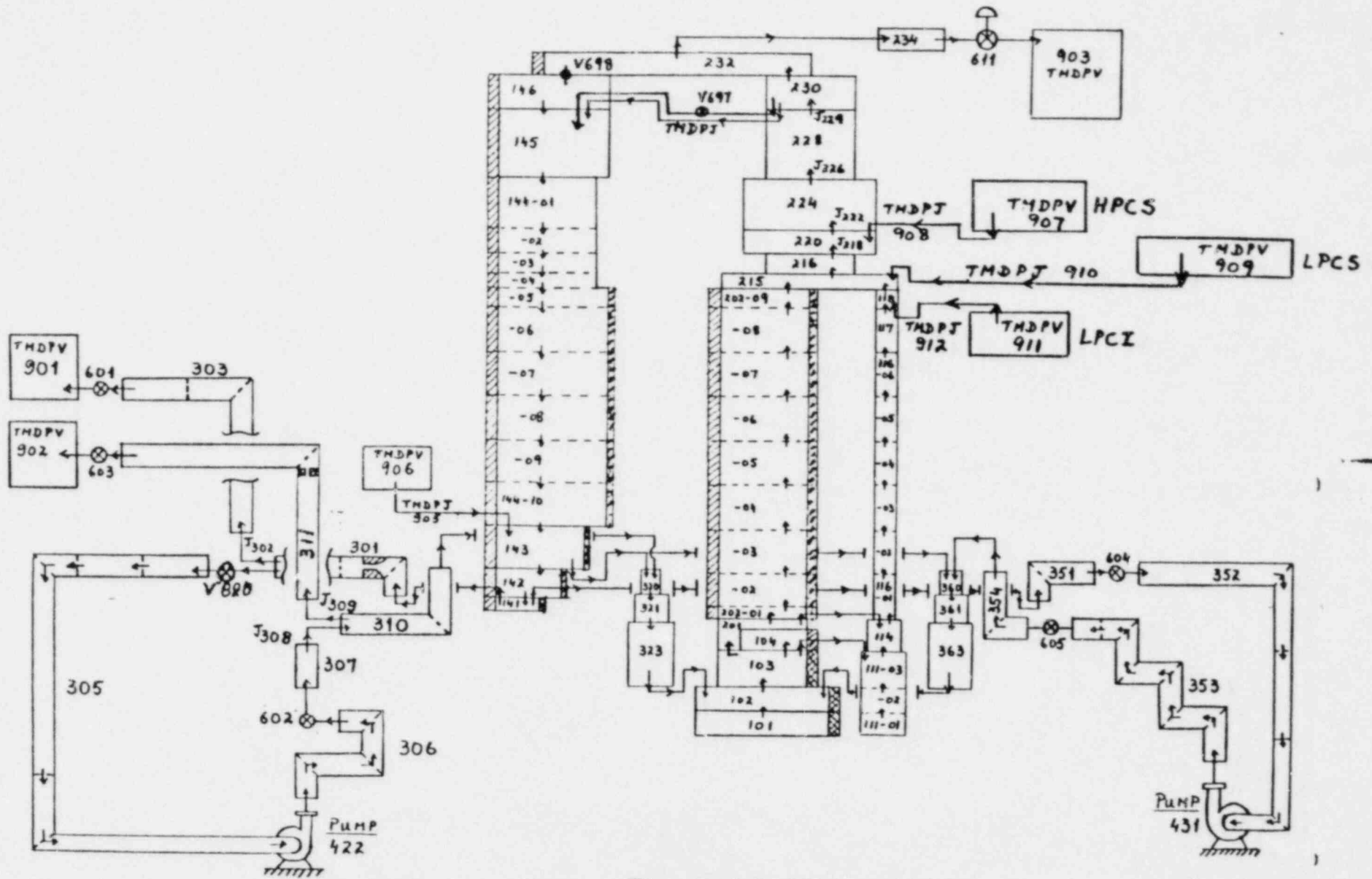
INITIAL CONDITIONS OF THE BD/ECC 1A REFERENCE TEST (6425 RUN 2)

<u>Initial Conditions</u>	<u>TLTA</u>	
Bundle power	5.05 <sup>a</sup> ± 0.03MW	
Steam dome pressure	1044 ± 5 psia	(7198 Pa)
Lower plenum pressure	1071 ± 5 psia	(7384 Pa)
Lower plenum enthalpy	528 ± 5 Btu/lbm	(1228 Kj/Kg)
Initial water level <sup>b</sup>	73 ± 6 in. E1	(1.85m)
Feed water enthalpy	41 ± 2 Btu/lbm	(95 Kj/Kg)
Bundle inlet to outlet DP	17 ± 2 psi	(117 Pa)
Steam flow	6 ± 1 lbm/sec	(2.7 Kg/s)
Feed water flow	1.4 ± 0.3 lbm/sec	(0.5 Kg/s)
Drive Pump 1 flow (INTACT)	9.1 ± 1 lbm/sec	(4.1 Kg/s)
Drive Pump 2 flow (BROKEN)	8.4 ± 1 lbm/sec	(3.8 Kg/s)
Jet Pump 1 flow (INTACT)	22 ± 2 lbm/sec	(10 Kg/s)
Jet Pump 2 flow (BROKEN)	20 ± 2 lbm/sec	(9 Kg/s)
Bundle inlet flow	39 ± 5 lbm/sec	(18 Kg/s)

All uncertainty bands are judged from the maximum of data fluctuation and/or absolute uncertainties of the measurements.

<sup>a</sup>NOTE: 5.05 MW is central average bundle power; core average power is 4.60 MW for BWR/6.

<sup>b</sup>NOTE: Relative to jet pump support plate.



TLTA-5A NODALIZATION

SEQUENCE OF EVENTS FOR 6425 RUN 2 (AVG. POWER, AVG. ECC)

<u>Events</u>	<u>Time (sec)</u>
Blowdown valves open	0.0
Bundle power decay initiated	0.5
Blowdown loop jet pump flow reverses	0.5
Feedwater flow stops	0.5
Bypass flow reverses	1.7
Jet pump suction uncovers	6.7
Steamline valve completely closed	9.0
Recirc. suction line begins to uncover	9.4
Lower plenum bulk flashing	11
Guide tube flashing	11.2
Core inlet uncovers (SEO center line)	20
Loop 1 isolated	20
HPCS injection begins	27
Lower plenum mixture level reaches jet pump exit plane	35
LPCS, LPCI activated	37
LPCS flow begins	64
LPCI flow begins	75
Bypass/guide tube region begins to refill	85
CCFL breaks down at bypass outlet	95
Bundle begins to refill	114
Bypass region refilled	125
Bundle reflood with two-phase mixture	130
CCFL breaks down at upper tieplate	125
Bundle quenched	150
End of test	400

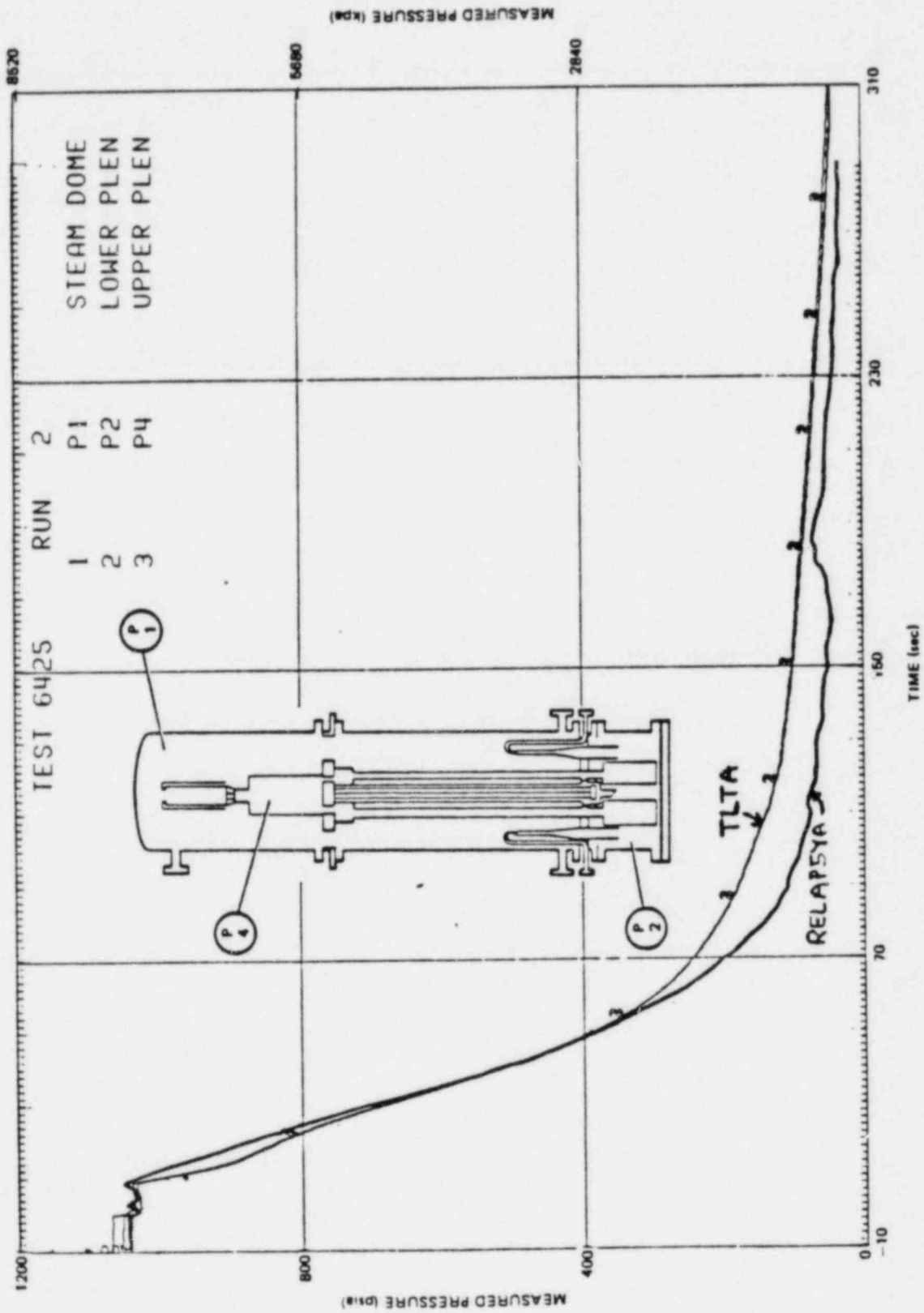


Figure J-5. System Pressures

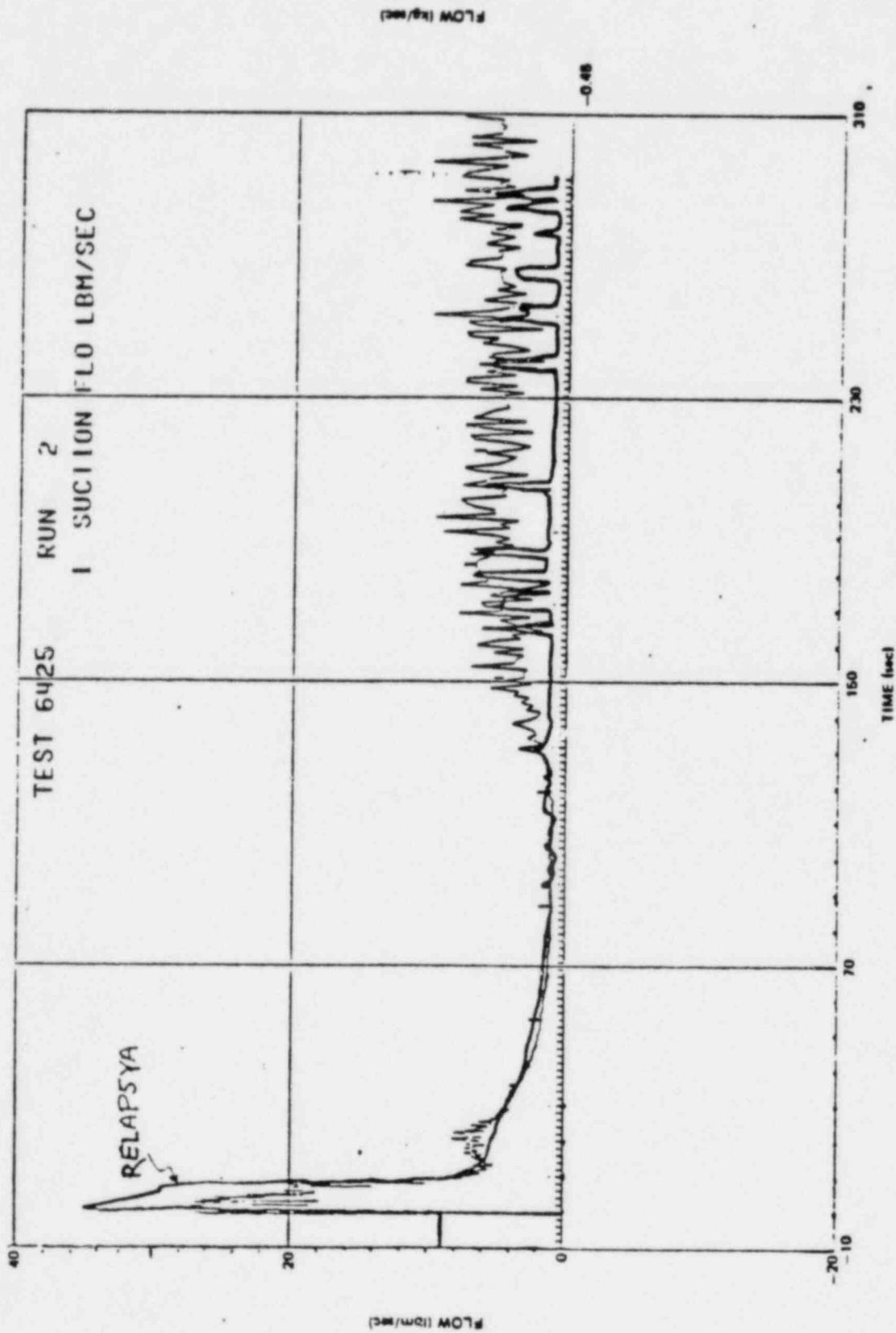


Figure J-65. Suction Line Break Mass Flow Rate

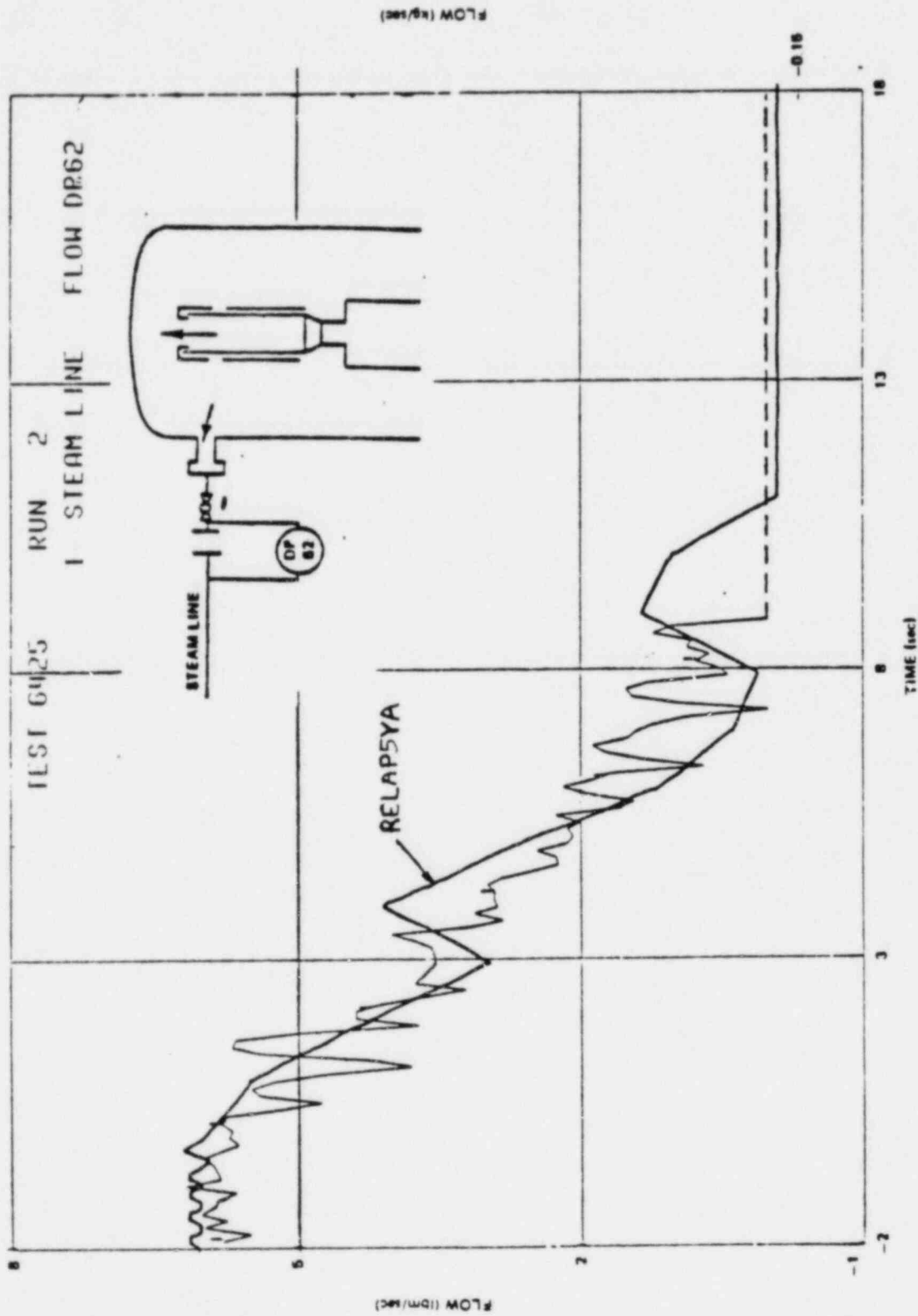


Figure J-72. Steam Line Mass Flow Rate



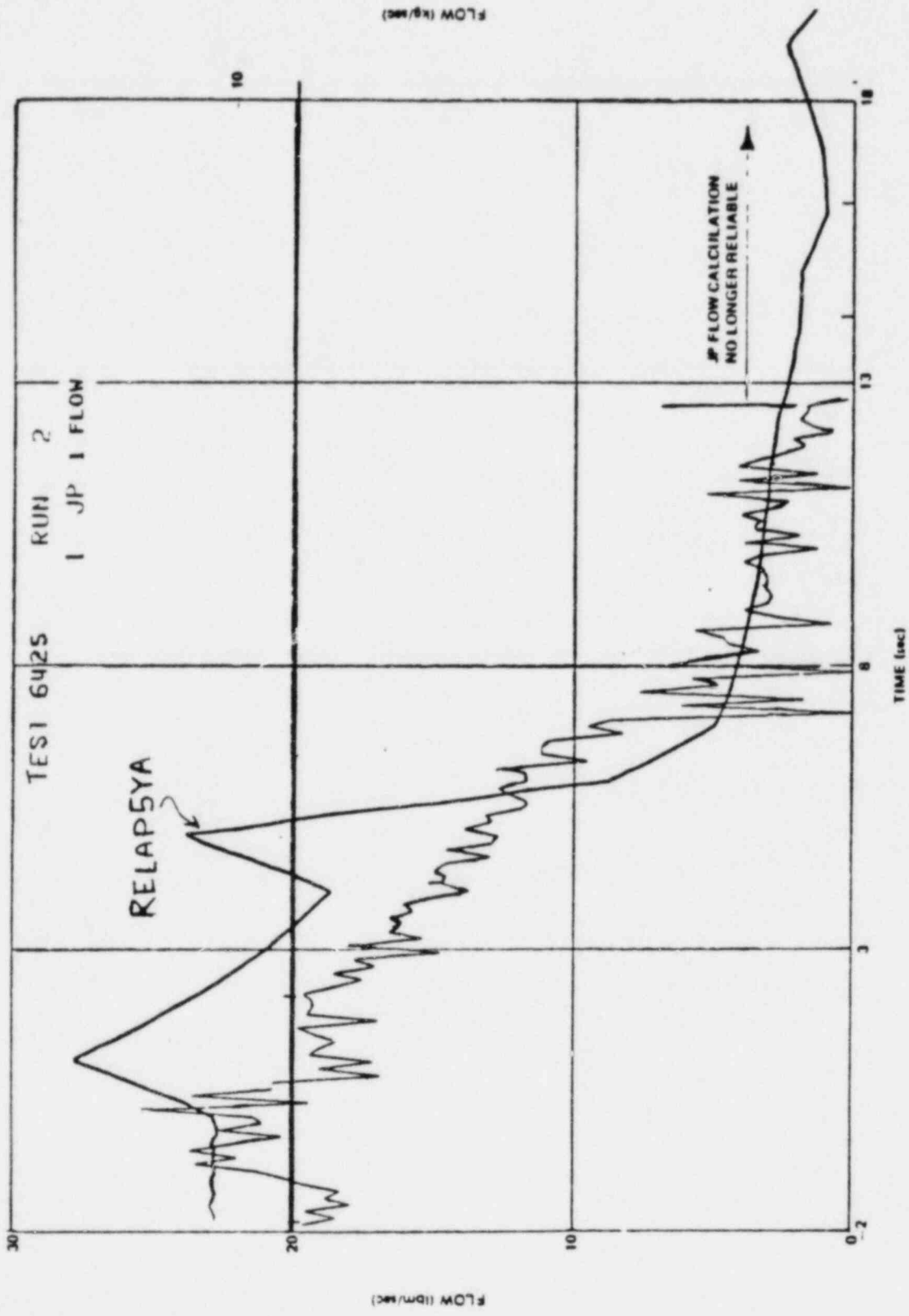


Figure J-75. Intact Loop Jet Pump Mass Flow Rate

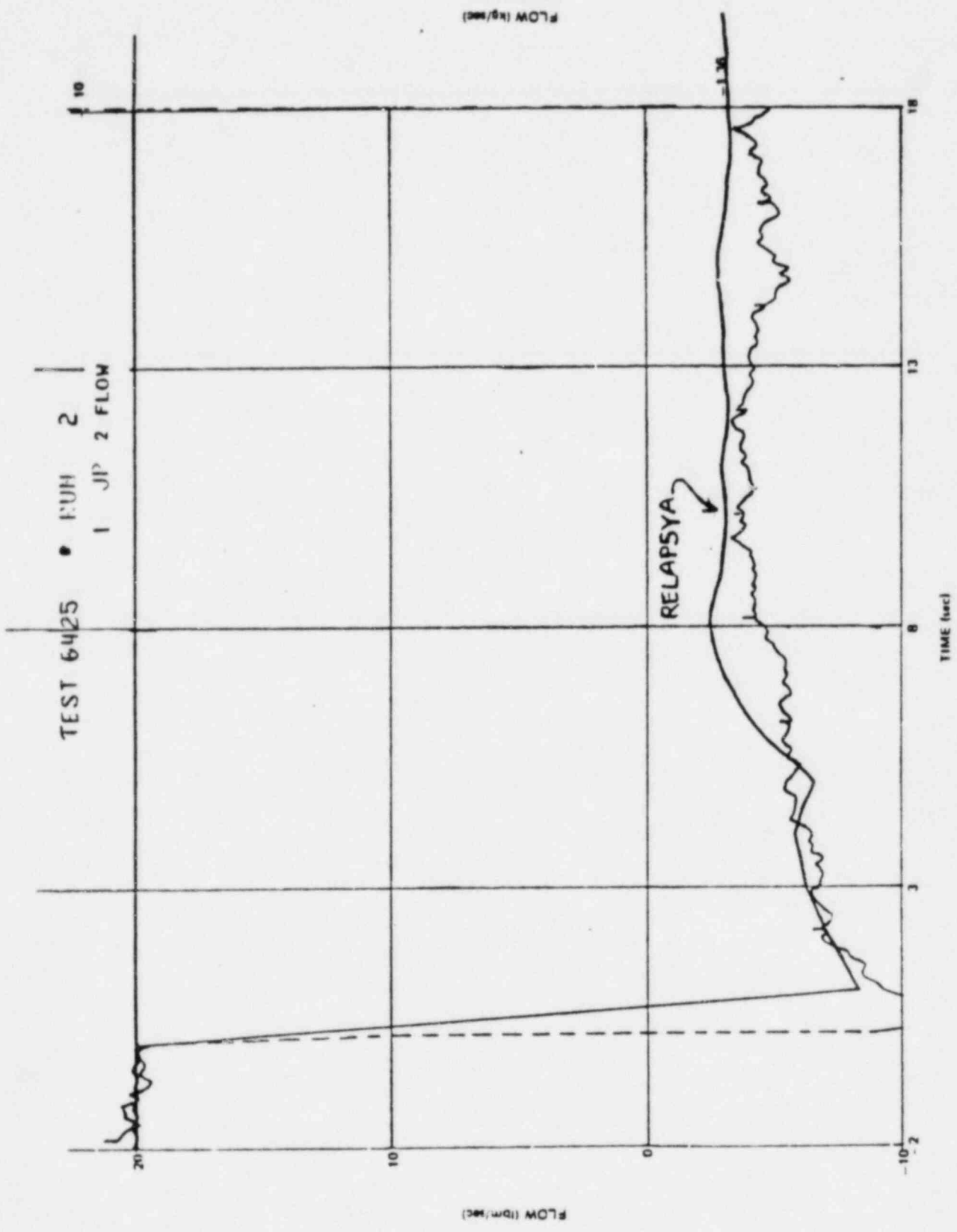


Figure J-76. Broken Loop Jet Pump Mass Flow Rate

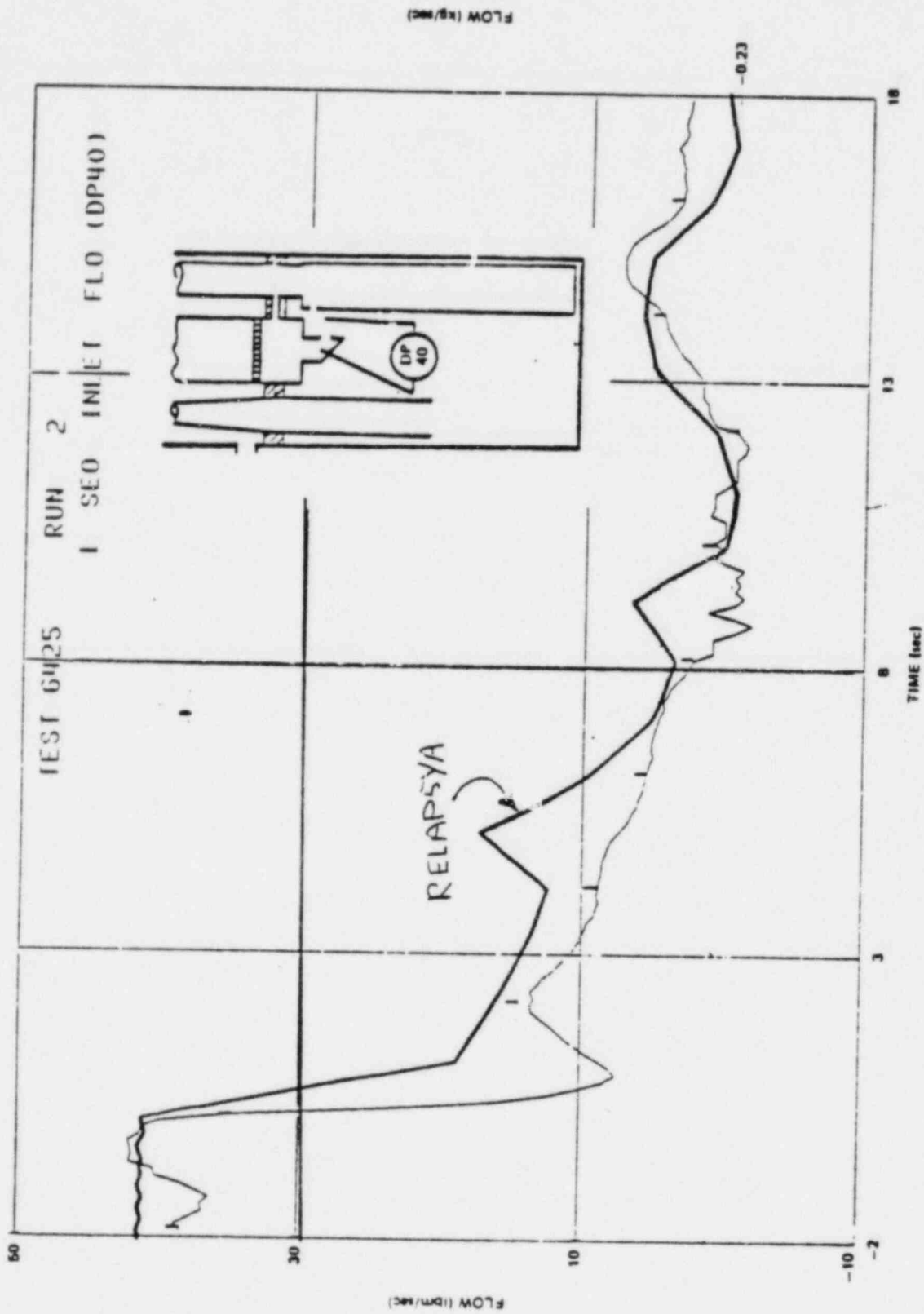


Figure J-68. Bundle Inlet Side Entry Orifice Mass Flow Rate

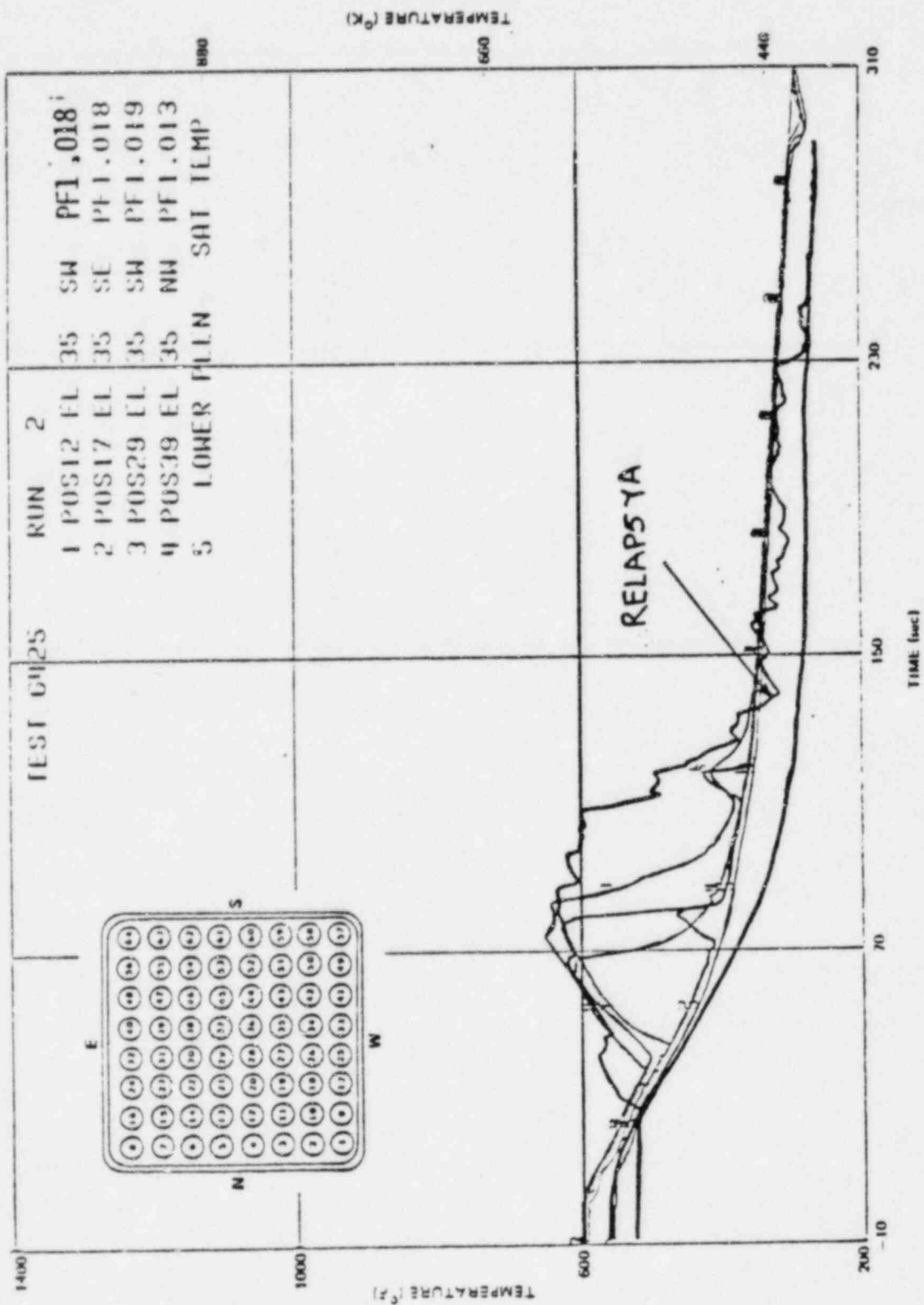


Figure J-95. Inside Clad Temperature-Elevation 35 in.

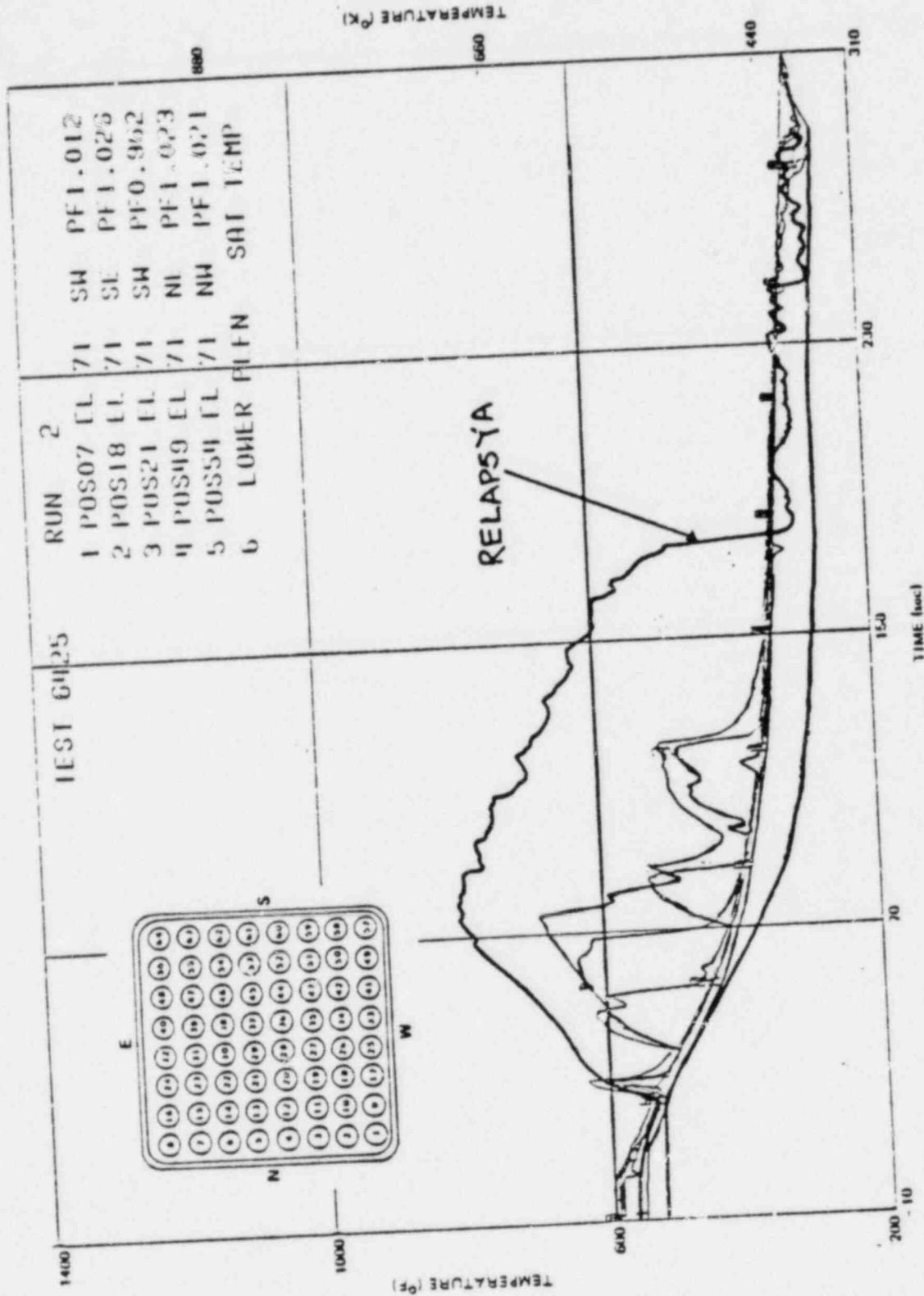


Figure J-91. Inside Clad Temperature-Elevation 71 in.

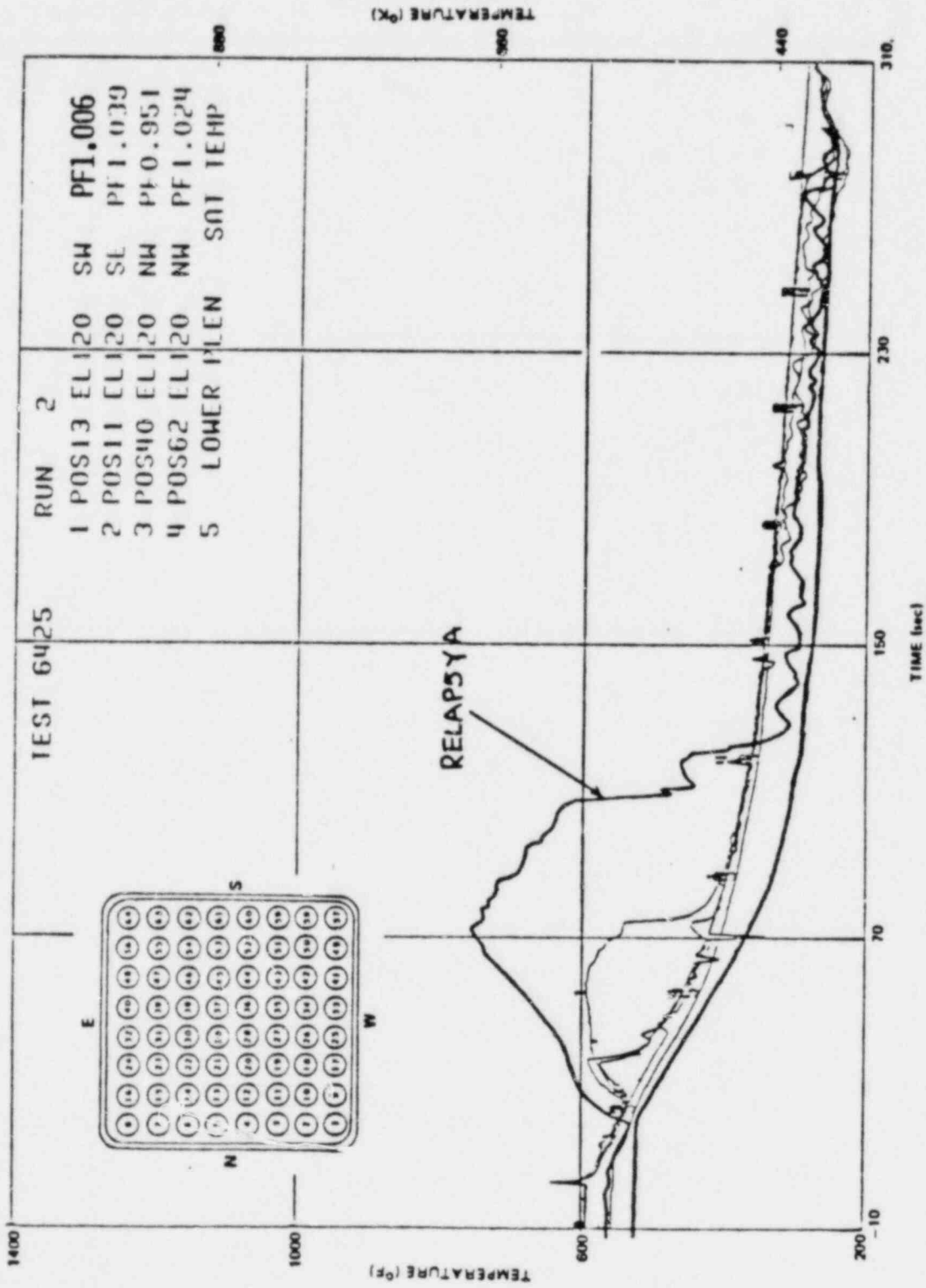


Figure J-83. Inside Clad Temperature-Elevation 120 in.

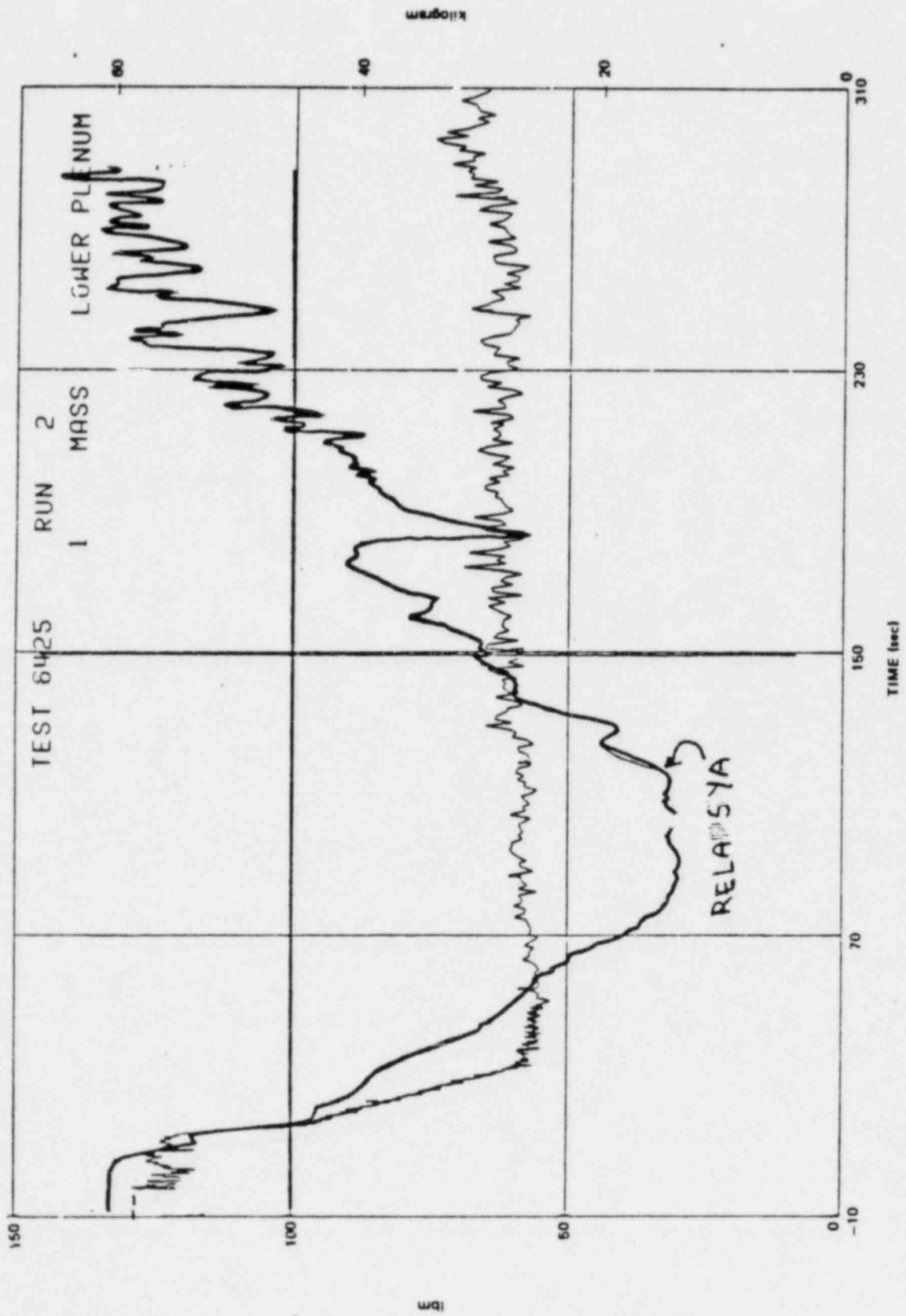


Figure J-49. Lower Plenum Fluid Mass

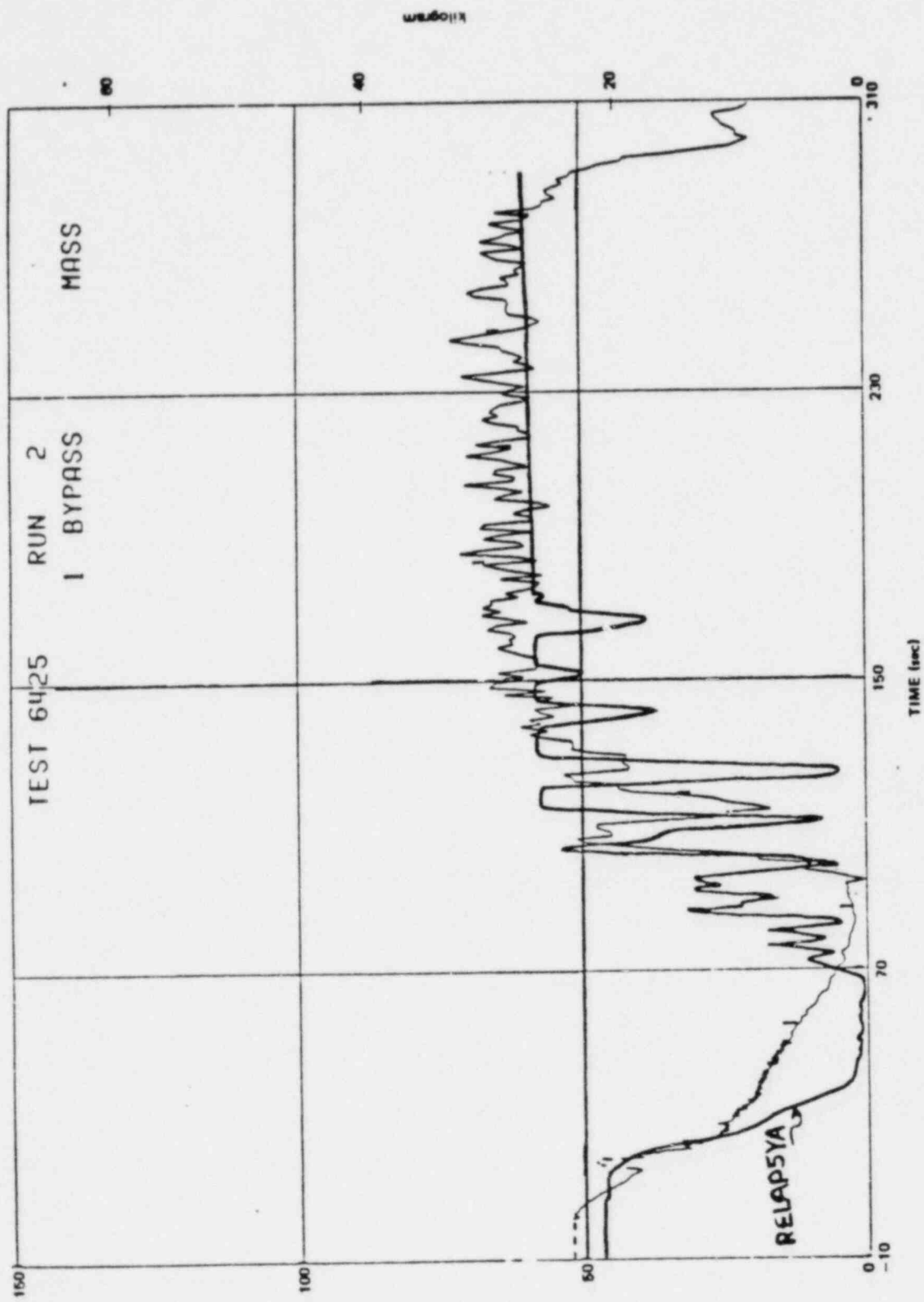


Figure J-53. Bypass Fluid Mass



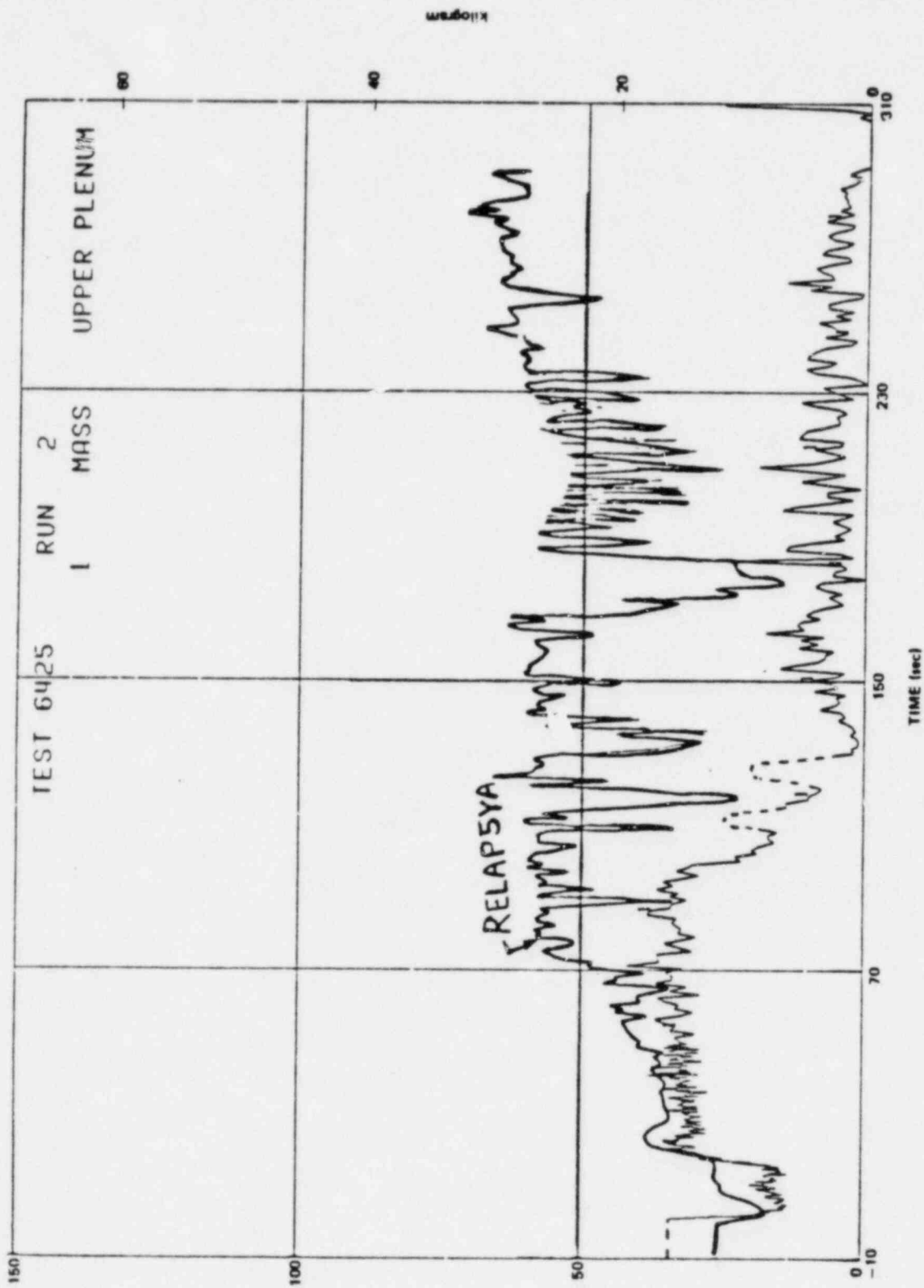


Figure J-54. Upper Plenum Fluid Mass

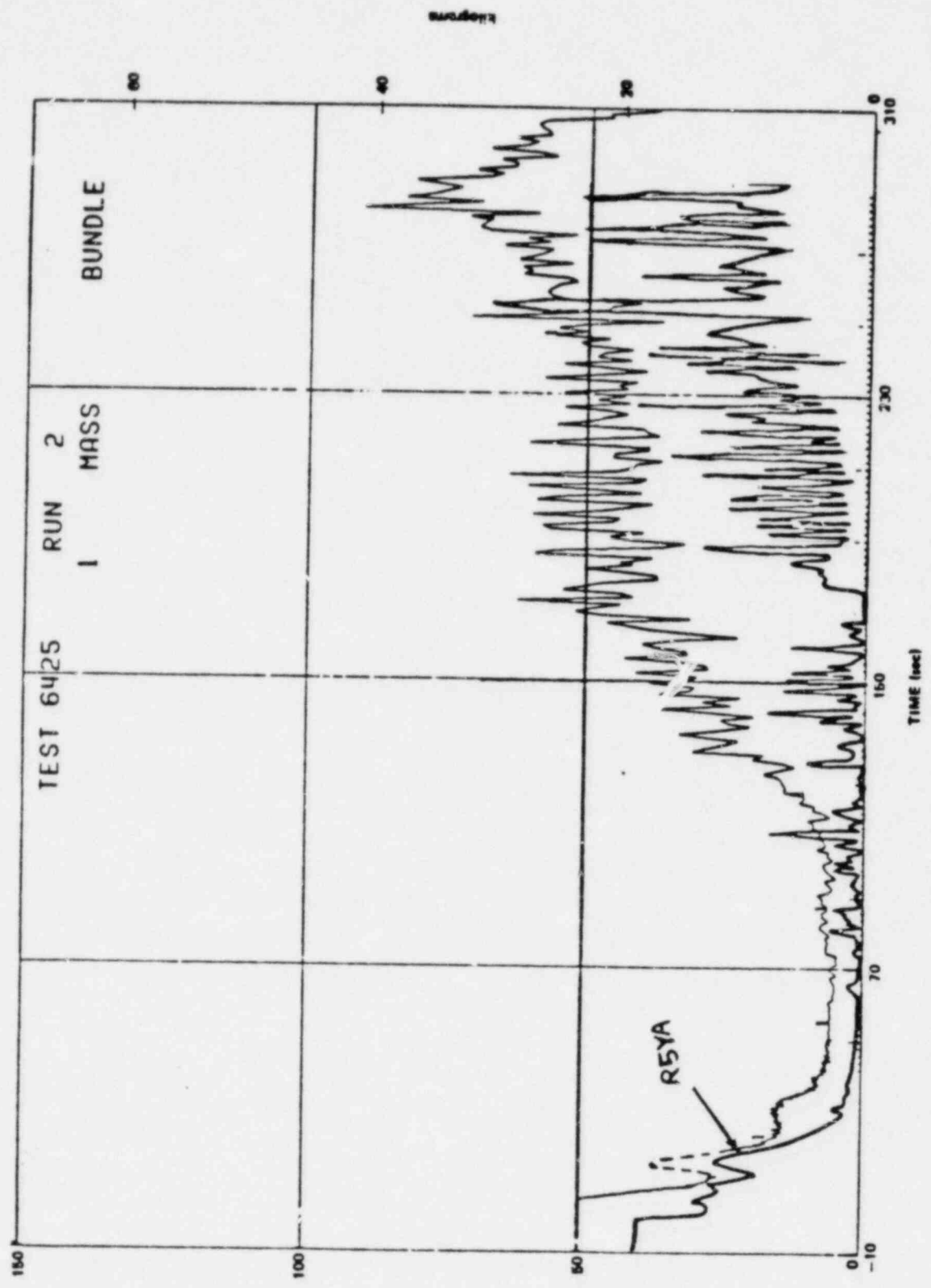


Figure J-50. Bundle Fluid Mass

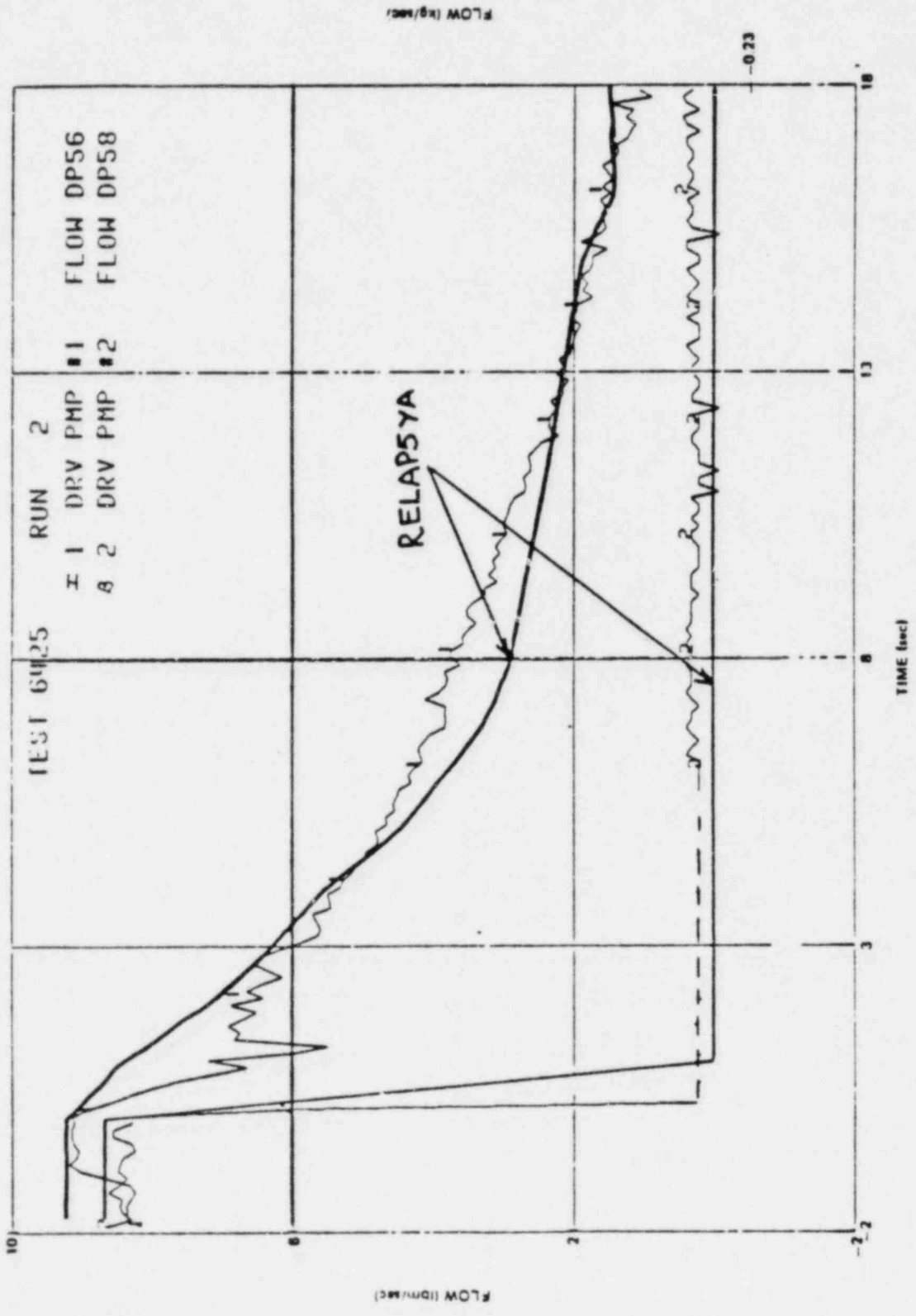


Figure J-74. Recirculation Pump Mass Flow Rates

# TLTA LARGE BREAK

TEST 6425/2  
(AVERAGE POWER, AVERAGE ECC)

EM CALCULATION

(RUN L3-6)

- EM HEAT TRANSFER FLAGS ON.
- ANS + 20% POWER DECAY
- 2% EXCESS POWER.

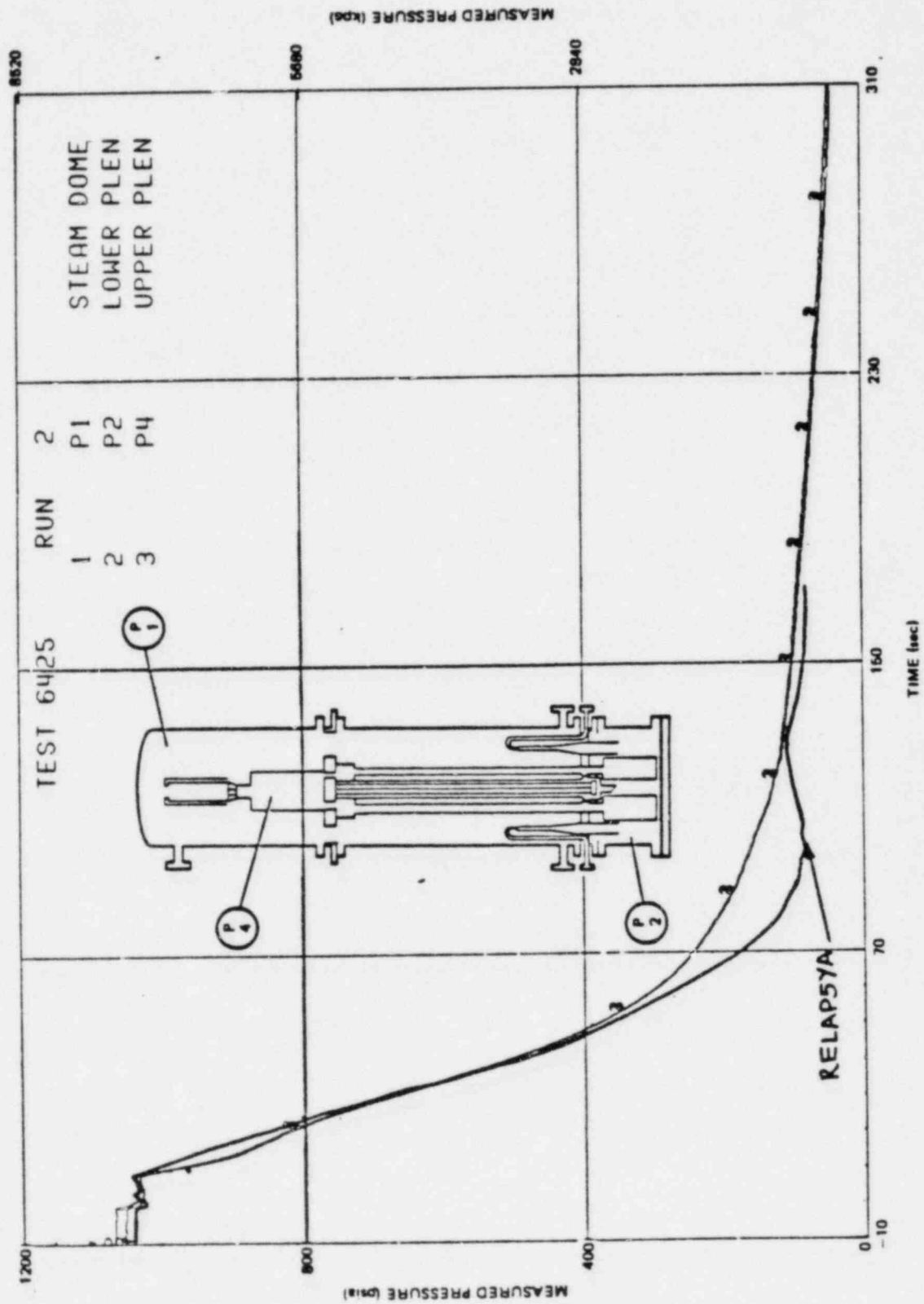


Figure J-5. System Pressures

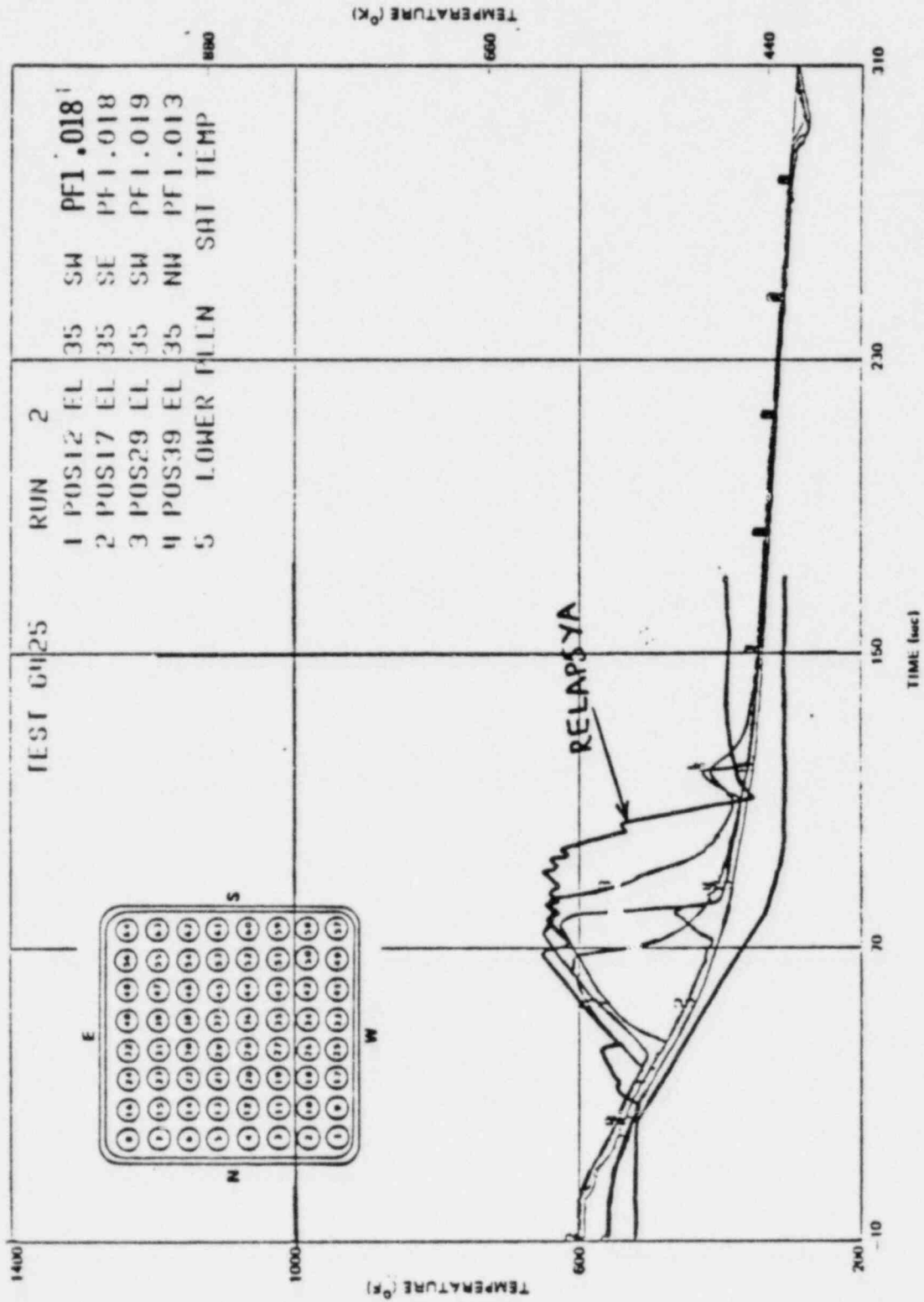


Figure J-95. Inside Clad Temperature-Elevation 35 in.

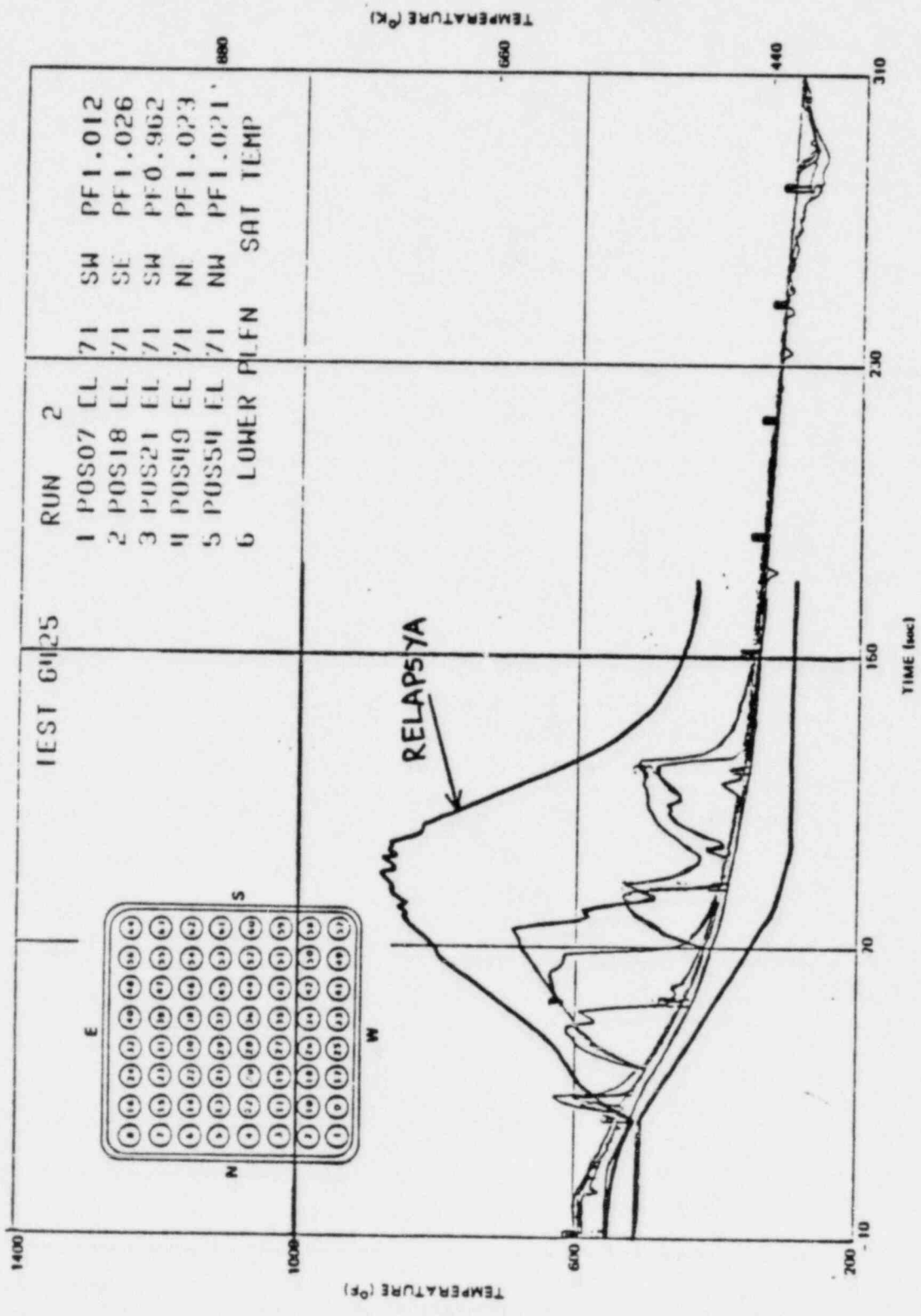


Figure J-91. Inside Clad Temperature-Elevation 71 in.

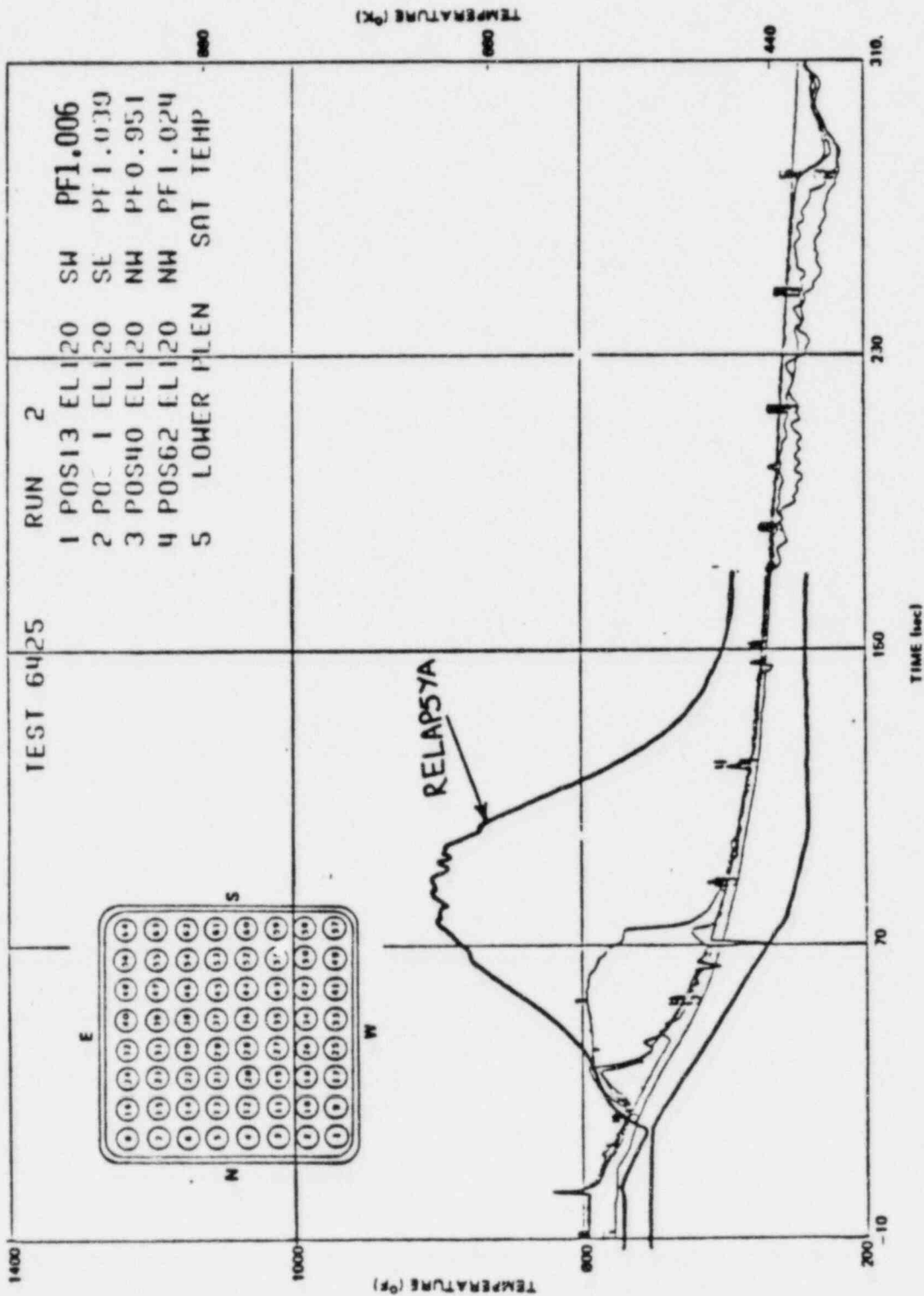


Figure J-83. Inside Clad Temperature-Elevation 120 in.



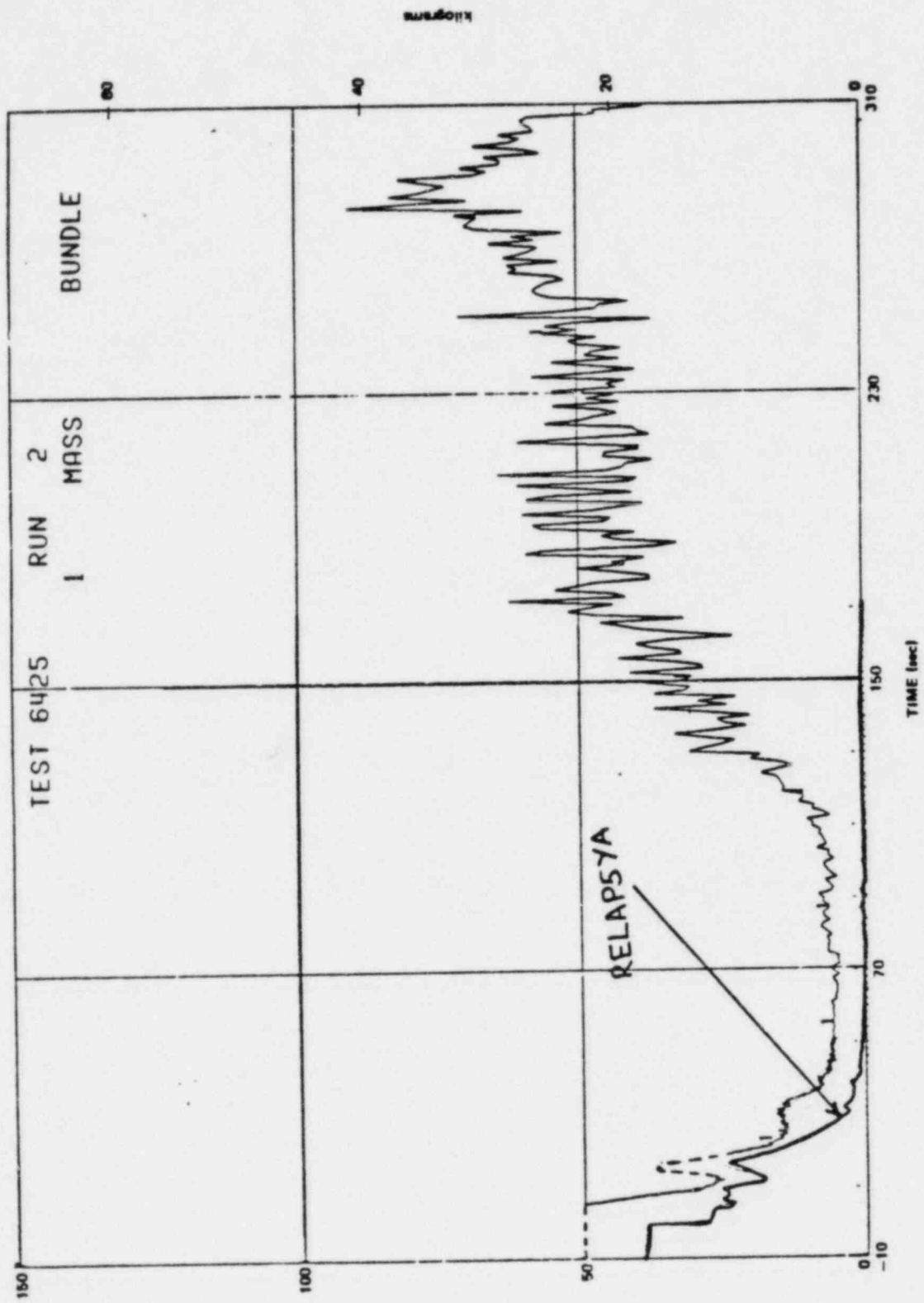


Figure J-50. Bundle Fluid Mass

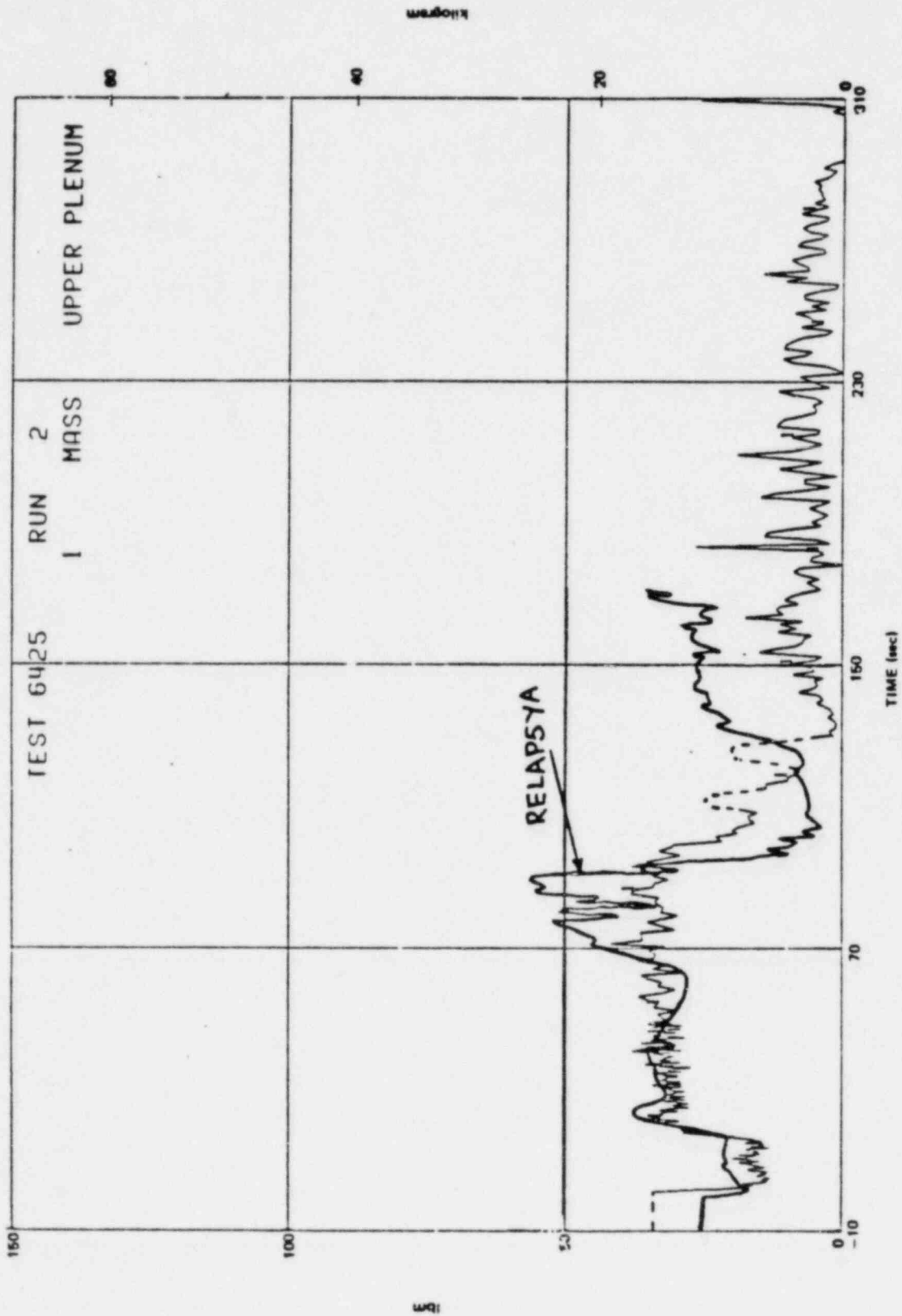


Figure J-54. Upper Plenum Fluid Mass

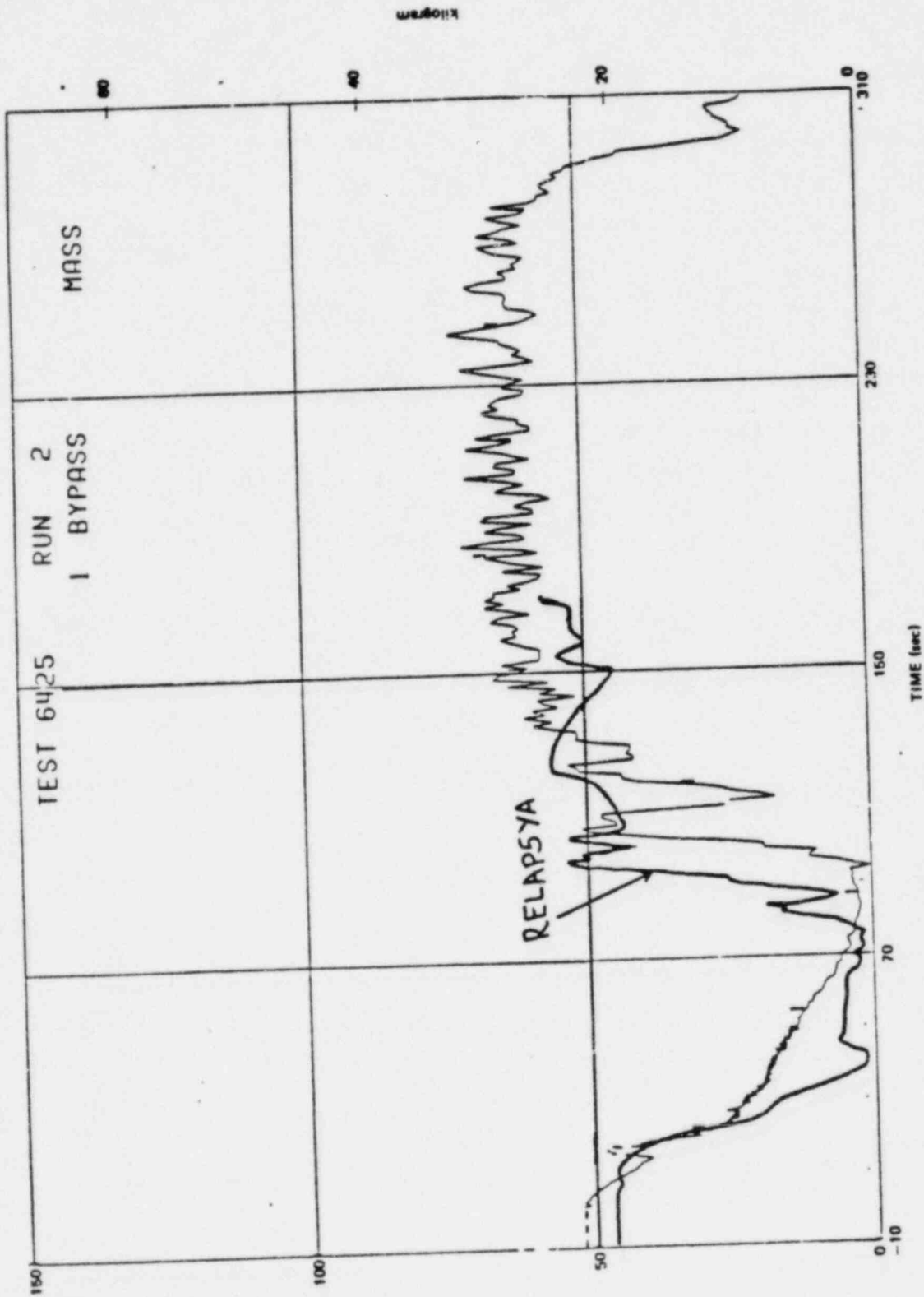


Figure J-53. Bypass Fluid Mass

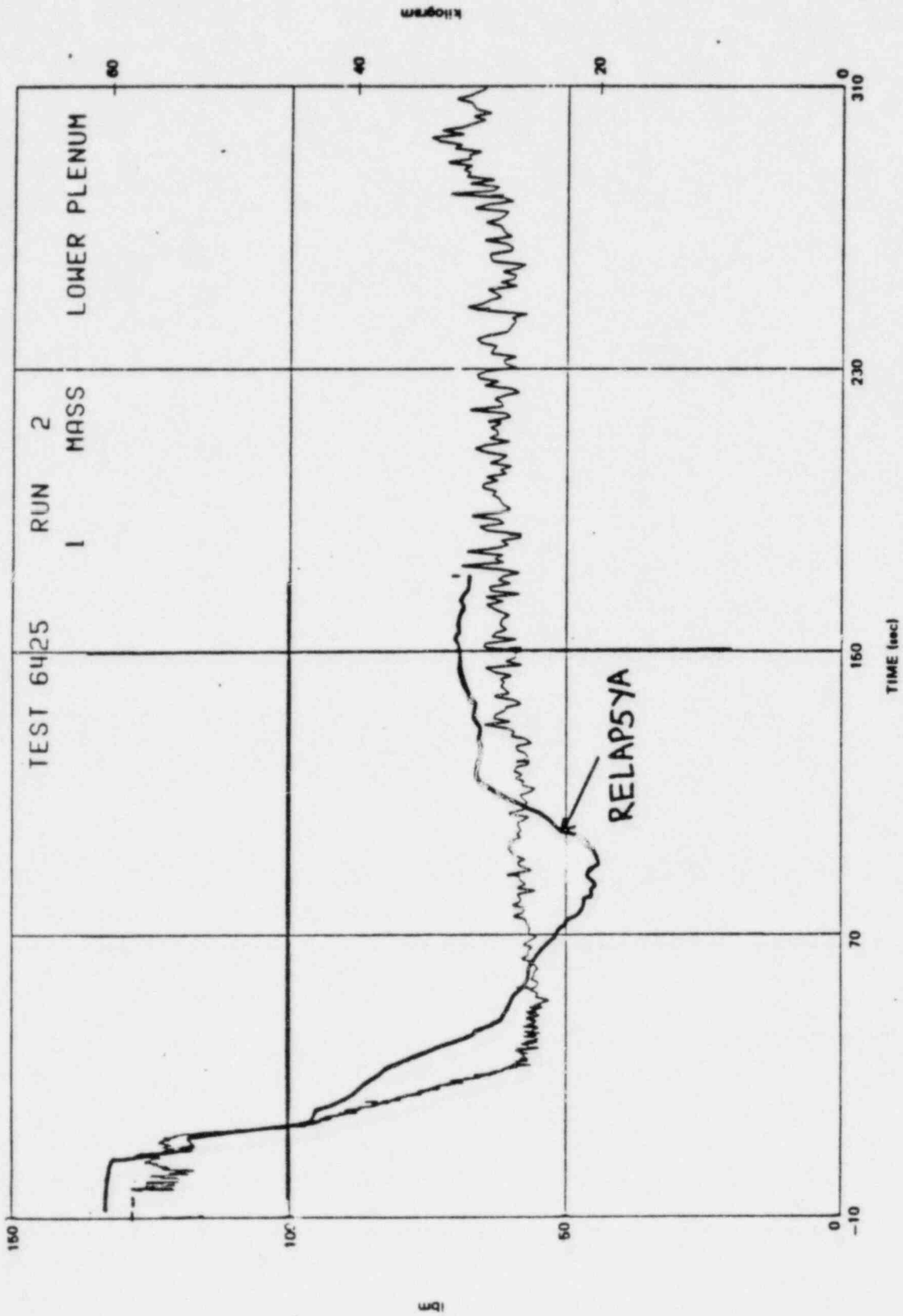


Figure J-49. Lower Plenum Fluid Mass

TLTA LARGE BREAK

TEST 6426/1

(AVG. POWER, NO ECC.)

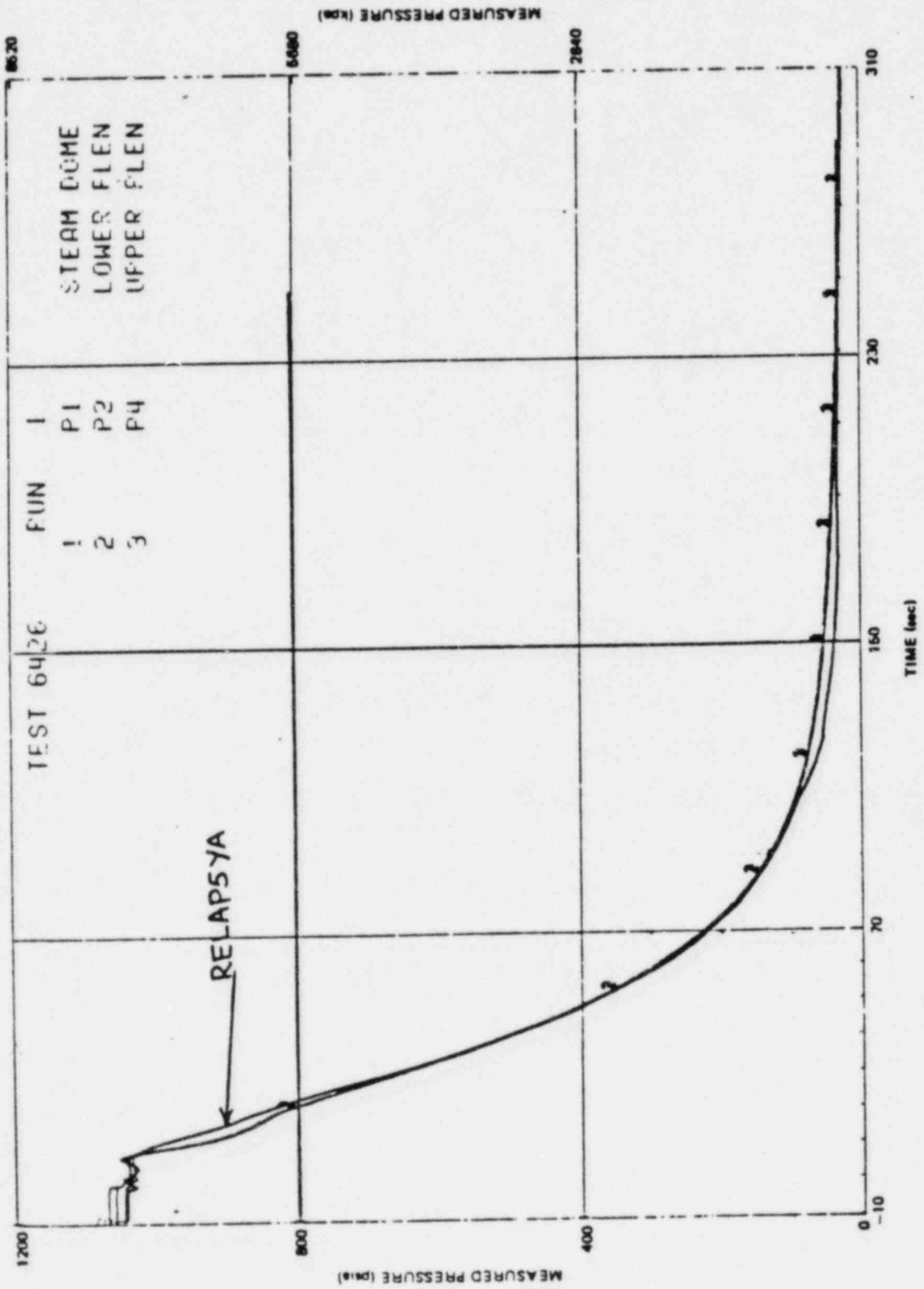


Figure L-5. System Pressures

L-55

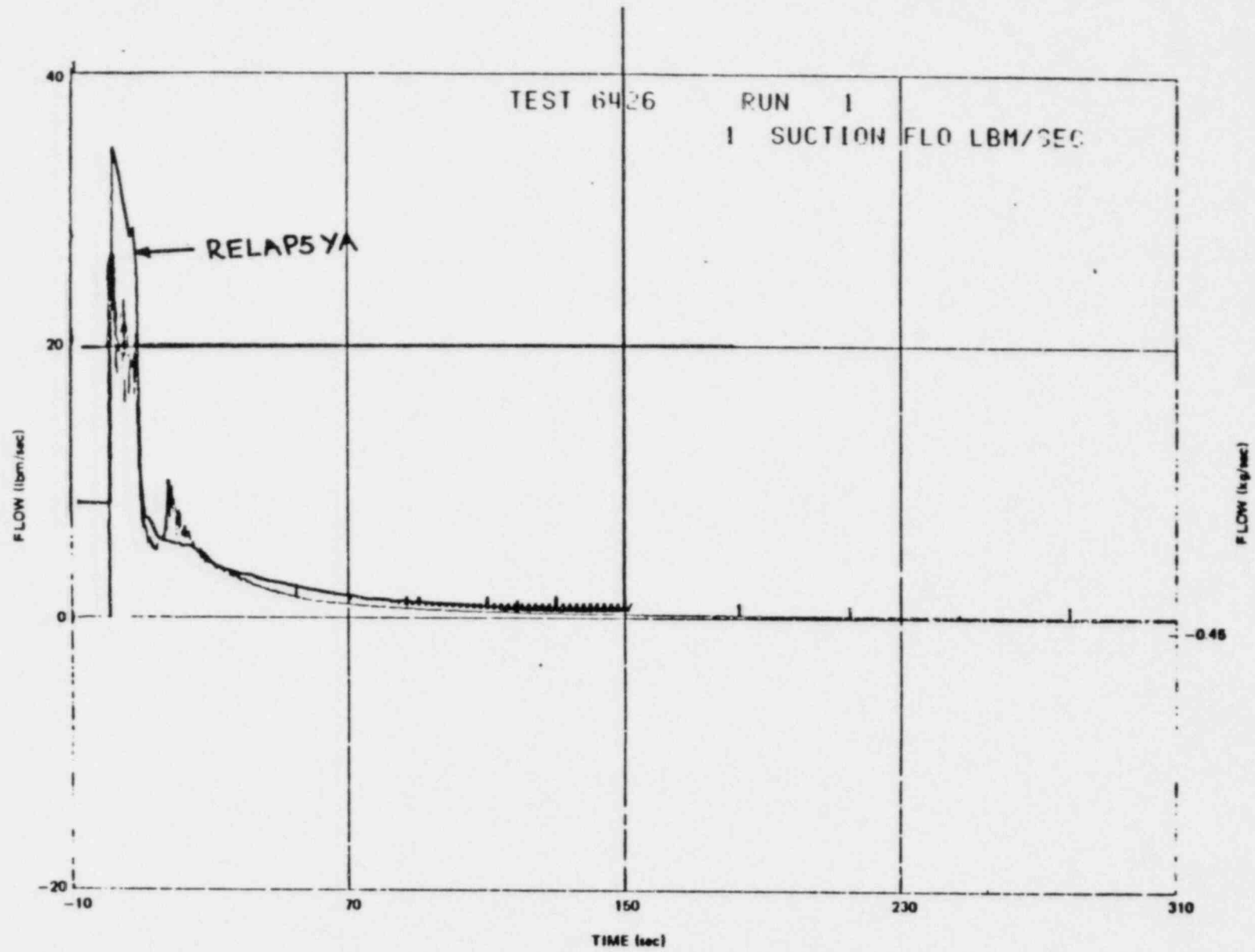


Figure L-50. Suction Line Break Mass Flow Rate

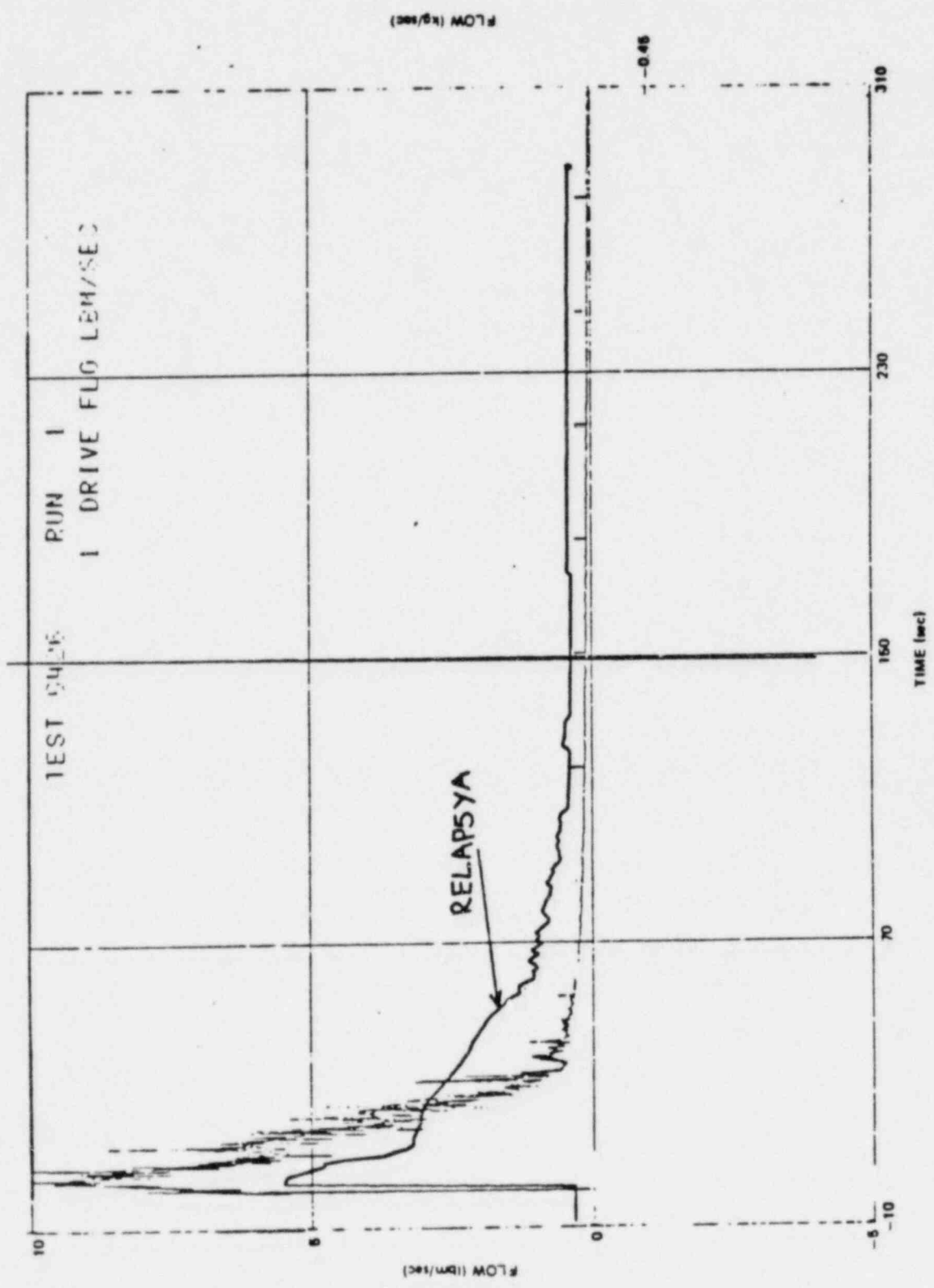


Figure L-49. Drive Line Break Mass Flow Rate



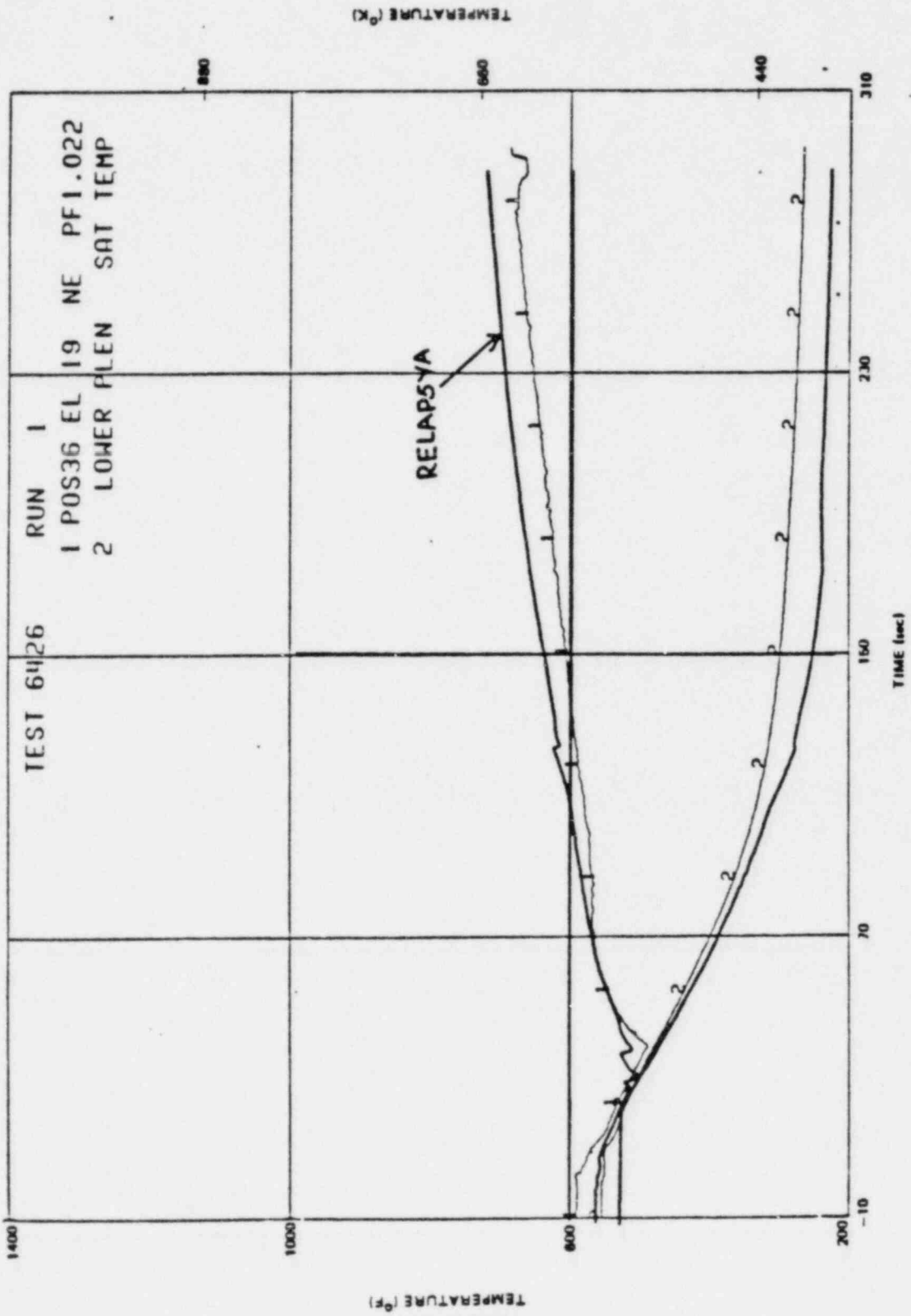


Figure L-76. Inside Clad Temperature - Elevation 19 in.

5

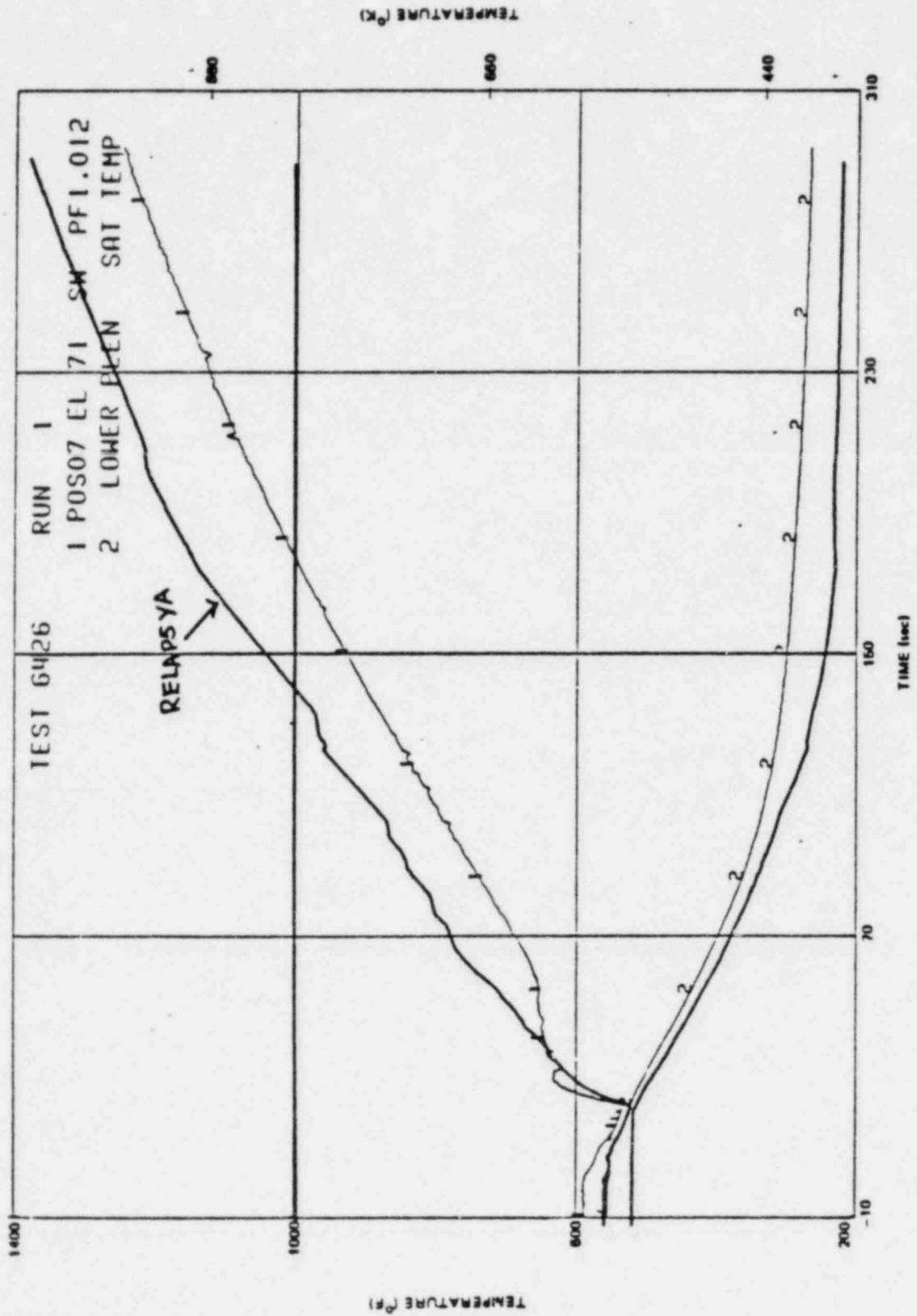


Figure L-74. Inside Clad Temperature - Elevation 71 in.

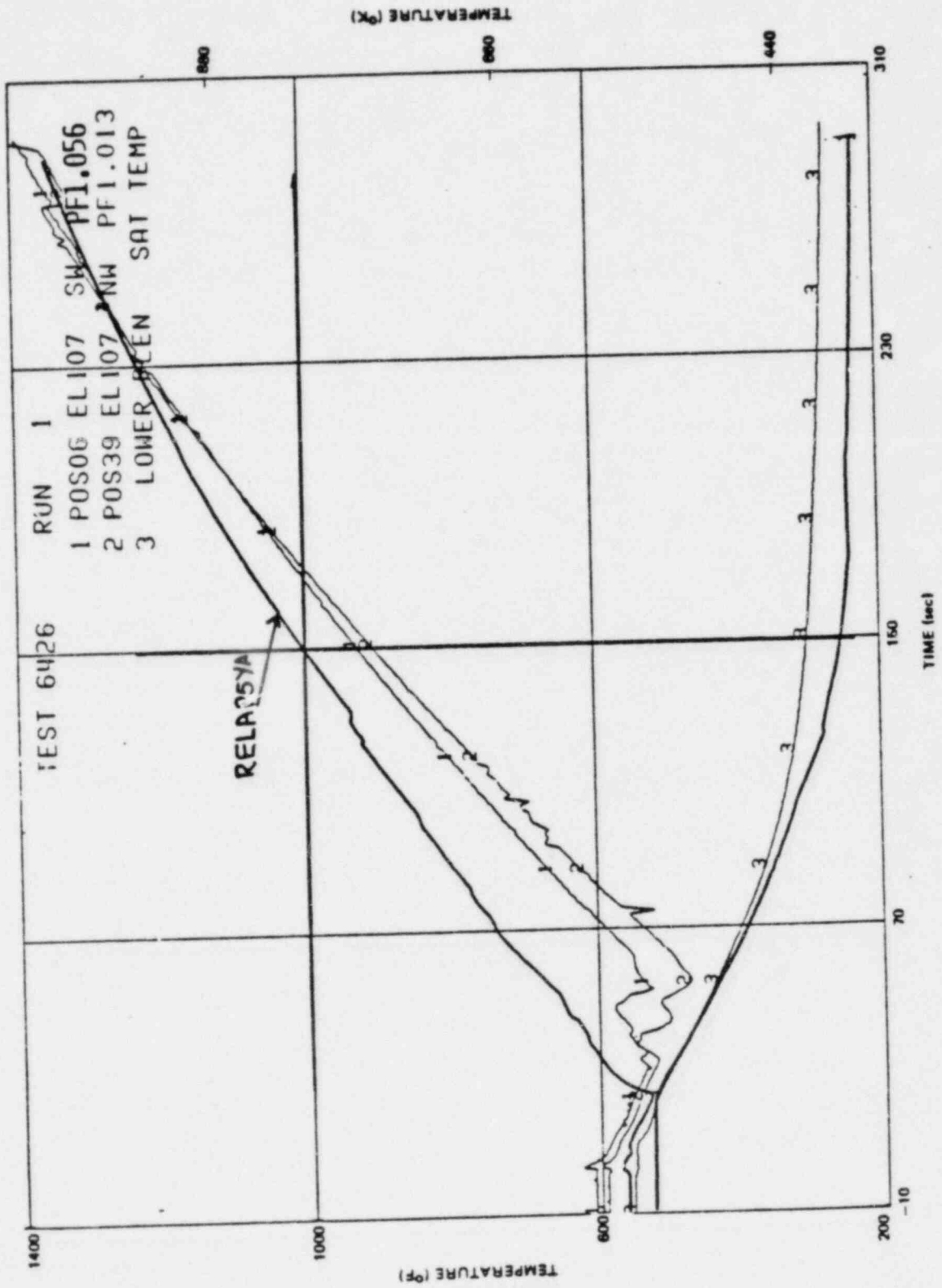


Figure L-69. Inside Clad Temperature - Elevation 107 in.

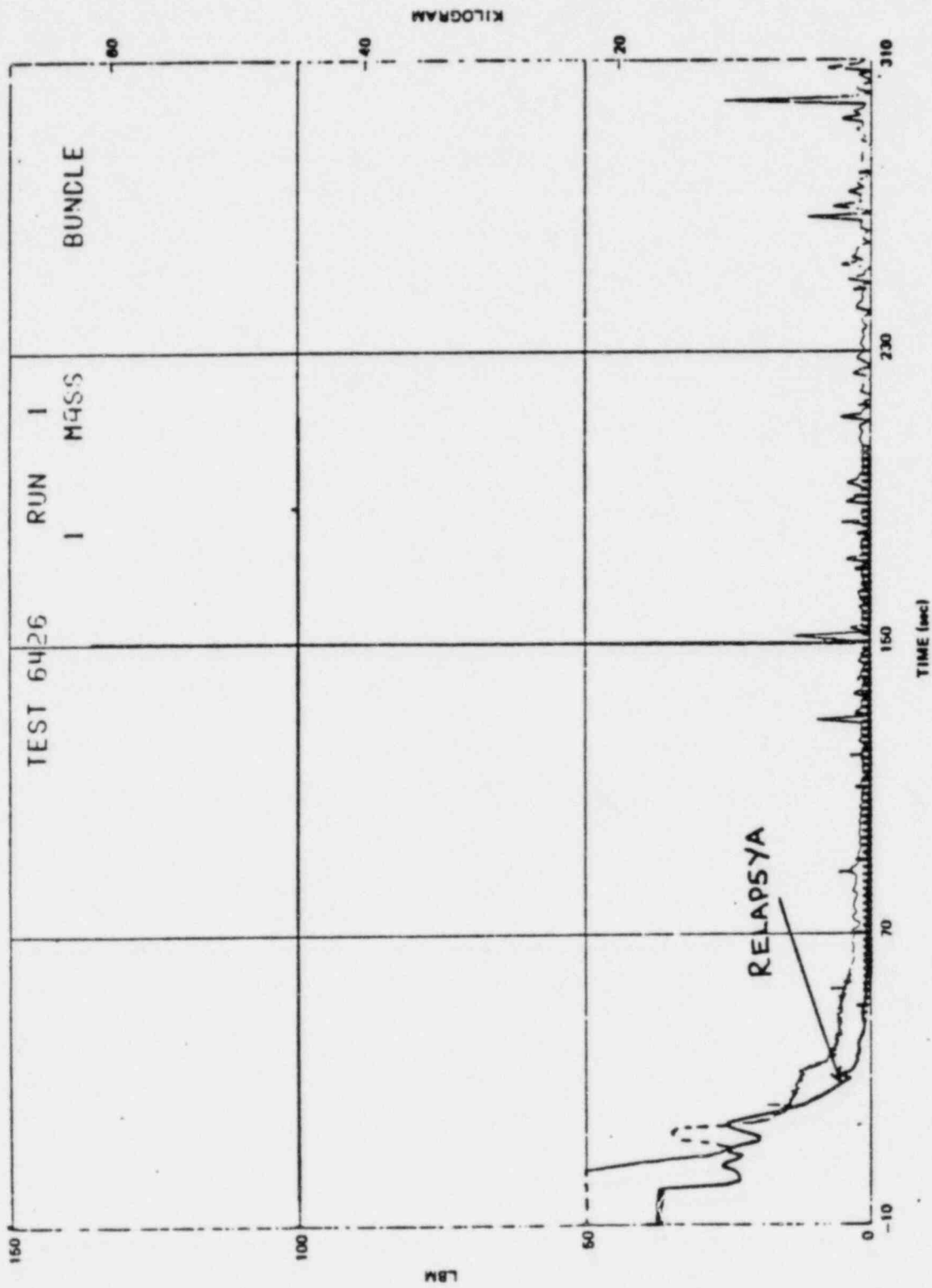


Figure L-39. Bundle Fluid Mass

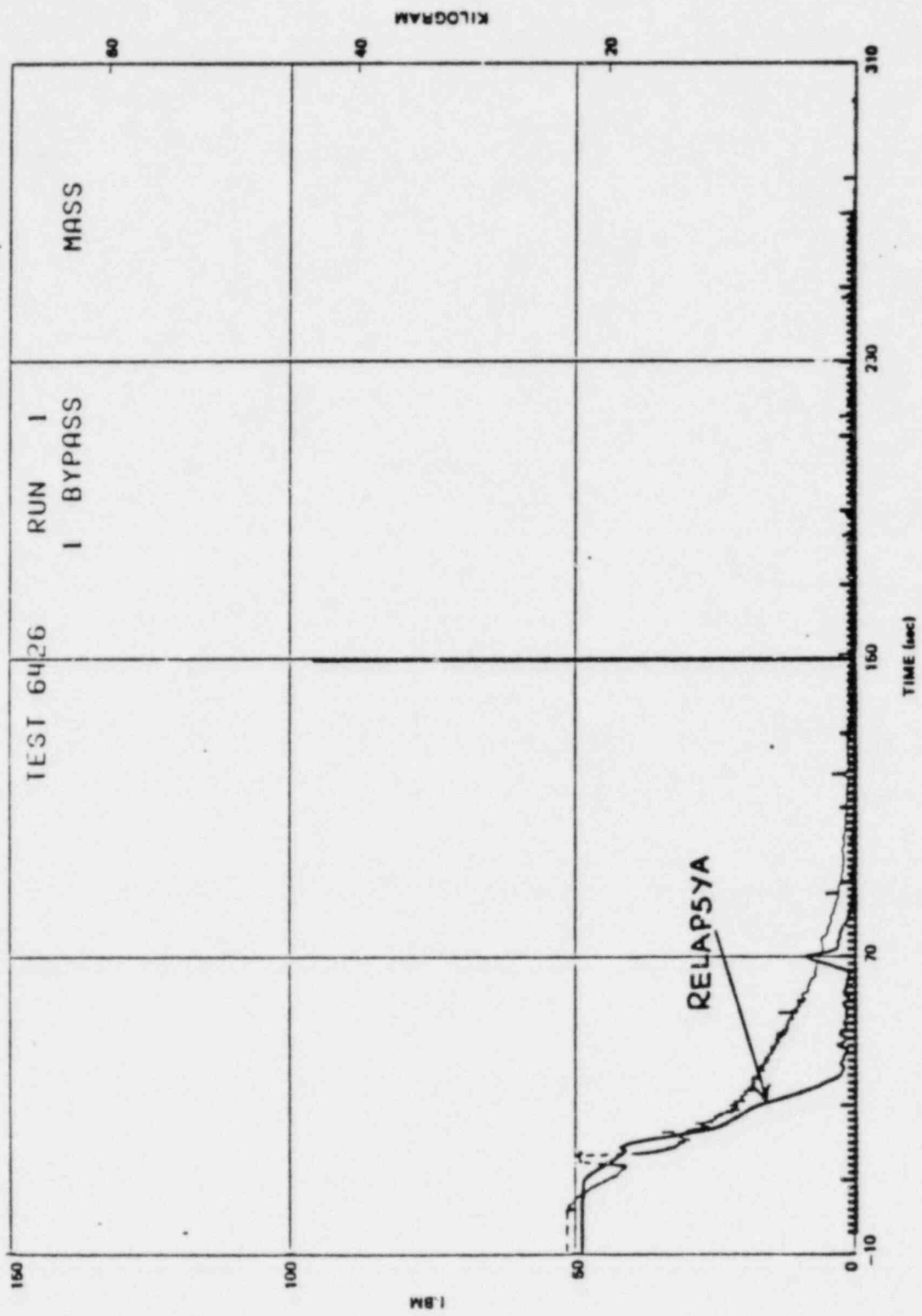


Figure L-42. Bypass Fluid Mass

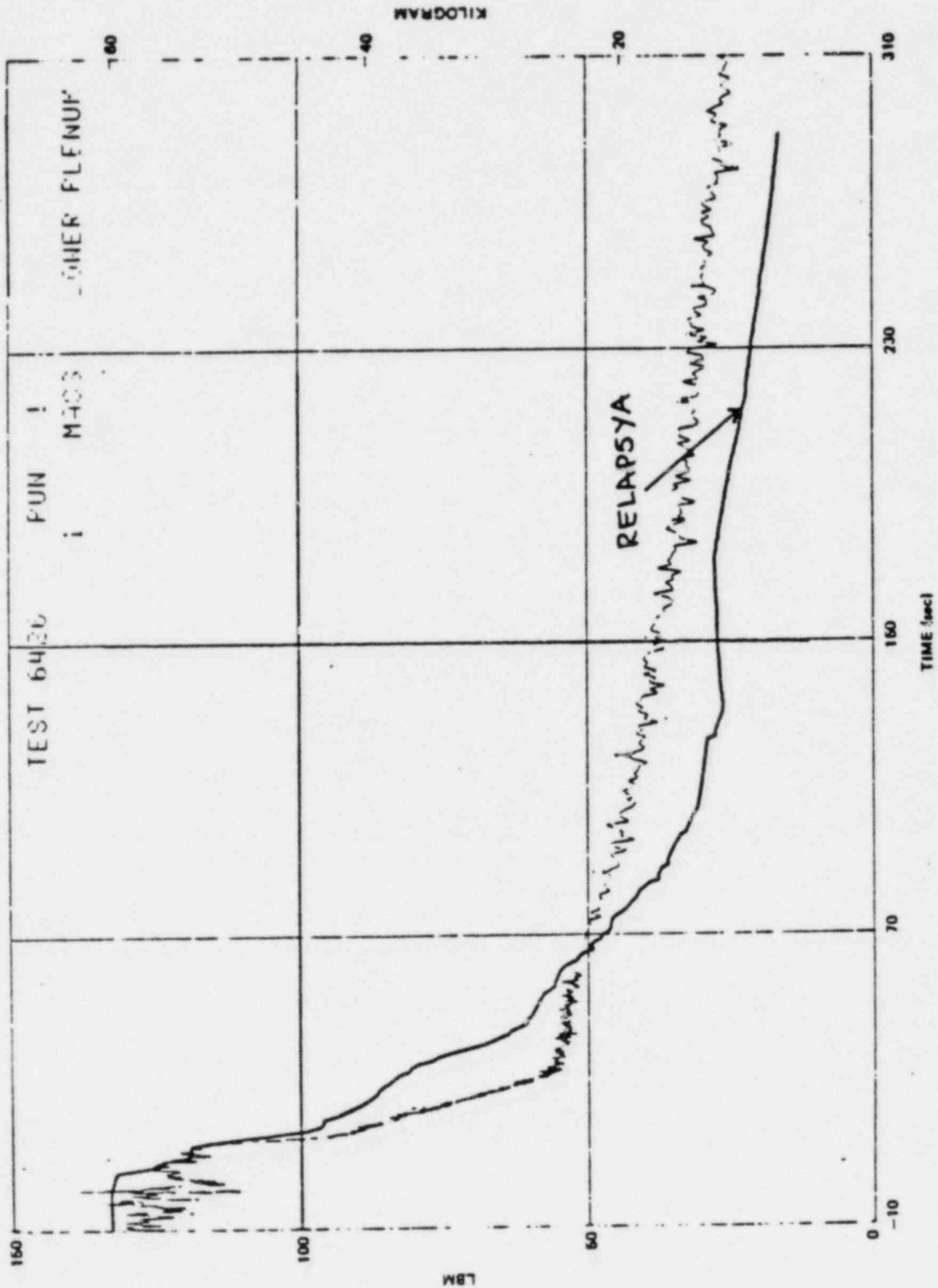


Figure L-38. Lower Plenum Fluid Mass

TLTA SMALL BREAK

TEST 6432/1

( DELAYED ADS  
DEGRADED ECC )

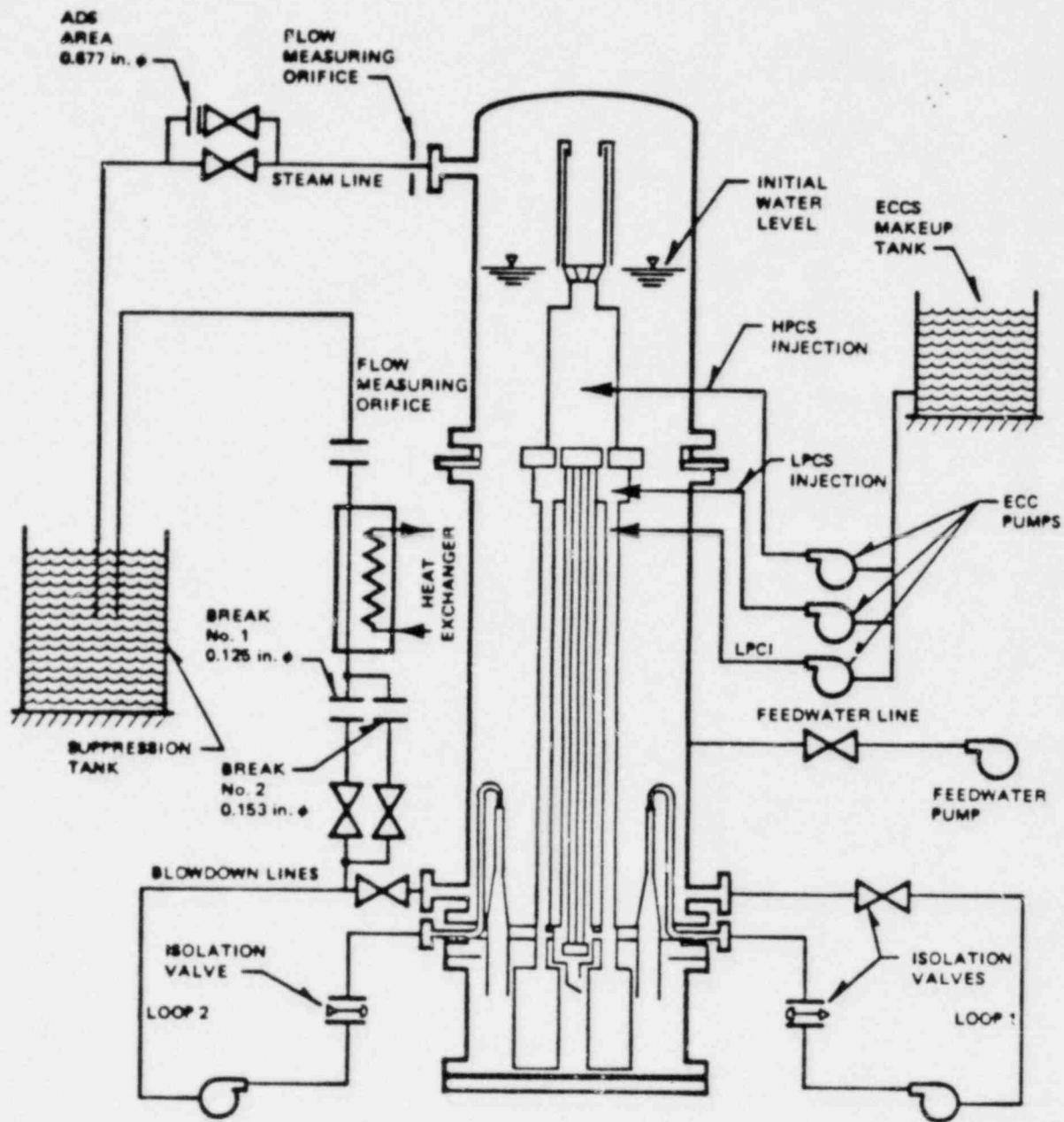


FIG: TLTA-5C CONFIGURATION FOR  
SMALL BREAK TEST II.



Table 4-2

## COMPARISON OF TEST CONDITIONS

(TLTA Small Break Test No. II, 6432/Run 1)

<u>Break Size</u>	<u>Specified</u>	<u>Measured</u>
Line No. 1	0.125±0.001 in. diameter	0.125±0.001 in. diameter
Line No. 2	0.153±0.001 in. diameter	0.153±0.001 in. diameter
ADS Orifice Size	0.677±0.001 in. diameter	0.677±0.001 in. diameter
ECCS		
Inlet Fluid Temperature	80±15°F	90±4°F
HPCS	HPCS deactivated	deactivated
LPCS (see Figure 2-6)	activated	activated
LPCI (see Figure 2-7)	activated	activated
Initial Condition		
Steam Dome Pressure	1050±20 psia	1048±5 psia
Water Level (Outside Shroud)	283±6 in. EL	283±3 in. EL
Bundle Flow (Core Flow)	34±5 lbm/sec.	34±5 lbm/sec.
Bypass Flow, Total	1.5±0.5 lbm/sec.	2.1±0.5 lbm/sec.
Steam Flow	1.4±0.5 lbm/sec.	1.6±0.5 lbm/sec.
Bundle Inlet Subcooling	23±5°F	21±4°F
Downcomer Temperature		
Above F.W. Sparger	T sat	553±4°F
Below F.W. Sparger	(T sat-23°F)±5°F	532±4°F
Timings		
Pump No. 1 Trip	0.0±0.2 sec.	0.0±0.1 sec.
Pump No. 2 Trip	4.0±1.0 sec.	4.0±0.2 sec.
Feed Water Trip	0.0±0.5 sec.	0.1±0.5 sec.
Break Open Line No. 1	t>140 sec.±1 sec.	t>138±1 sec.
Break Open Line No. 2	140<t<286 sec.	138±1<t<286±1 sec.
ADS Opening	286±2 sec.	286±1 sec.
MSIV (Steam Valve) Closure	166±2 sec.	165±1 sec.
ECCS Activated	37±1 sec.	37±1 sec.
Intact Recirculation Loop (No. 1 Isolated)	20±1 sec.	20±0.5 sec.

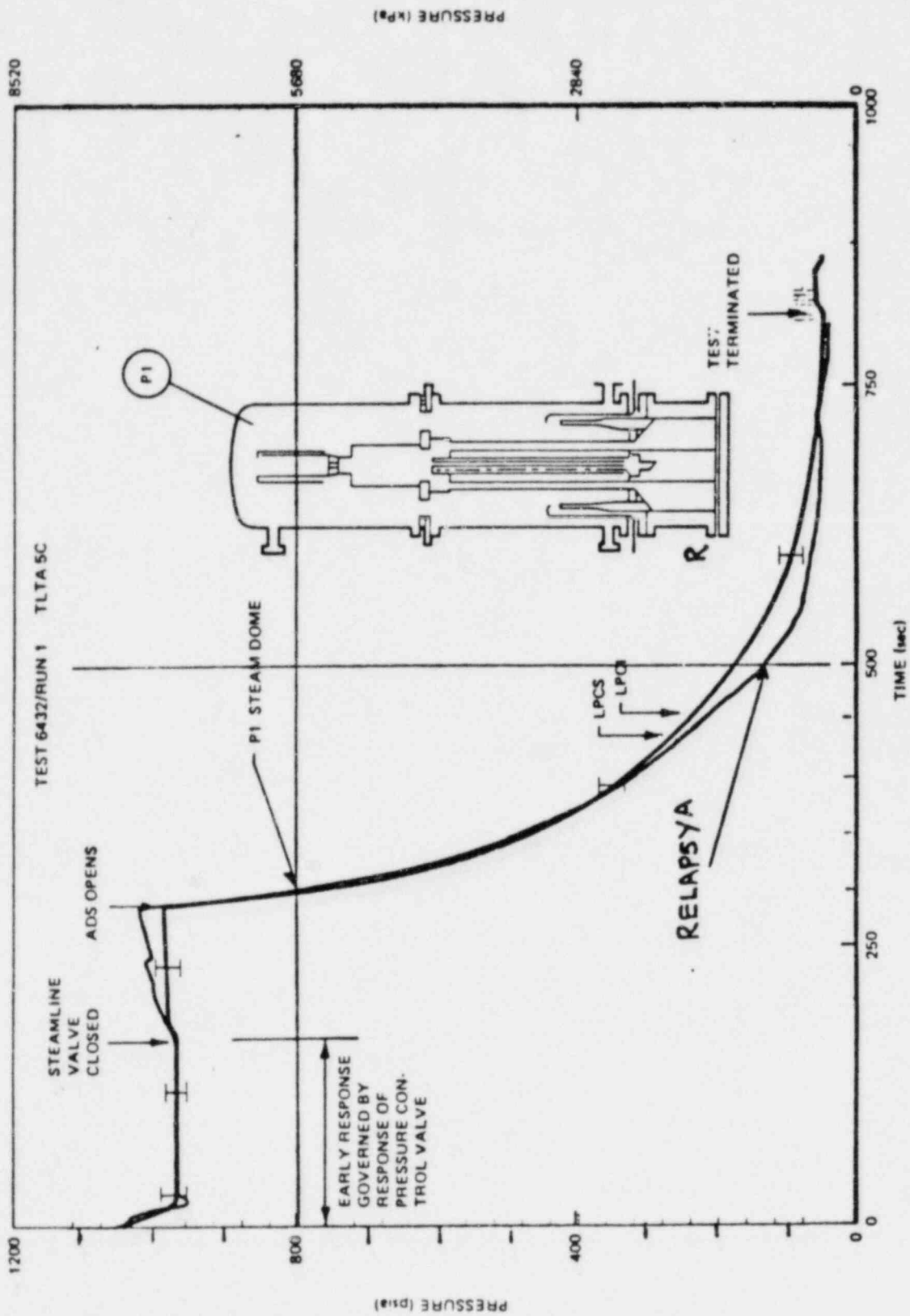


Figure D-2. System Pressure

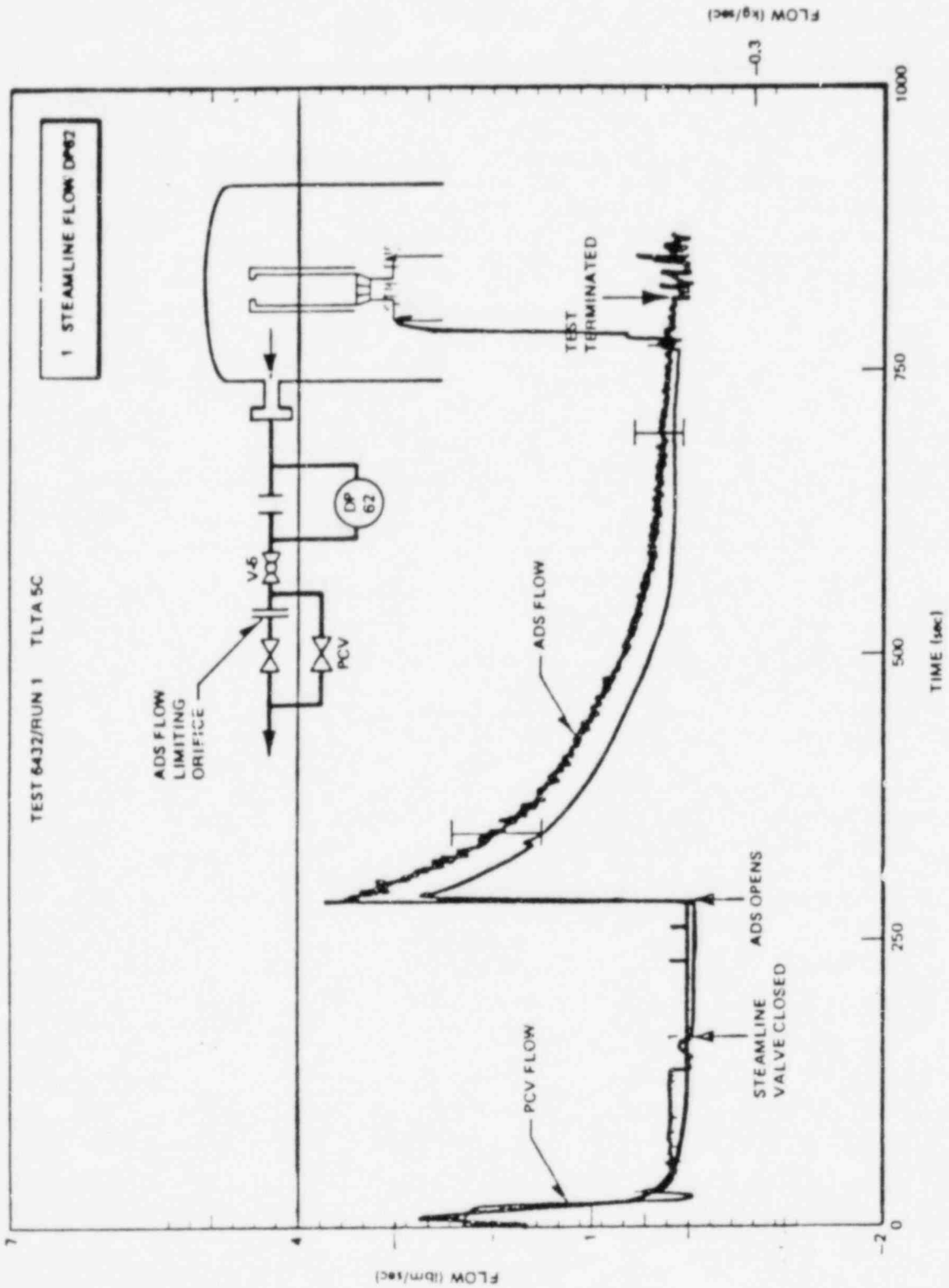


Figure D-5. Steam Line Flow

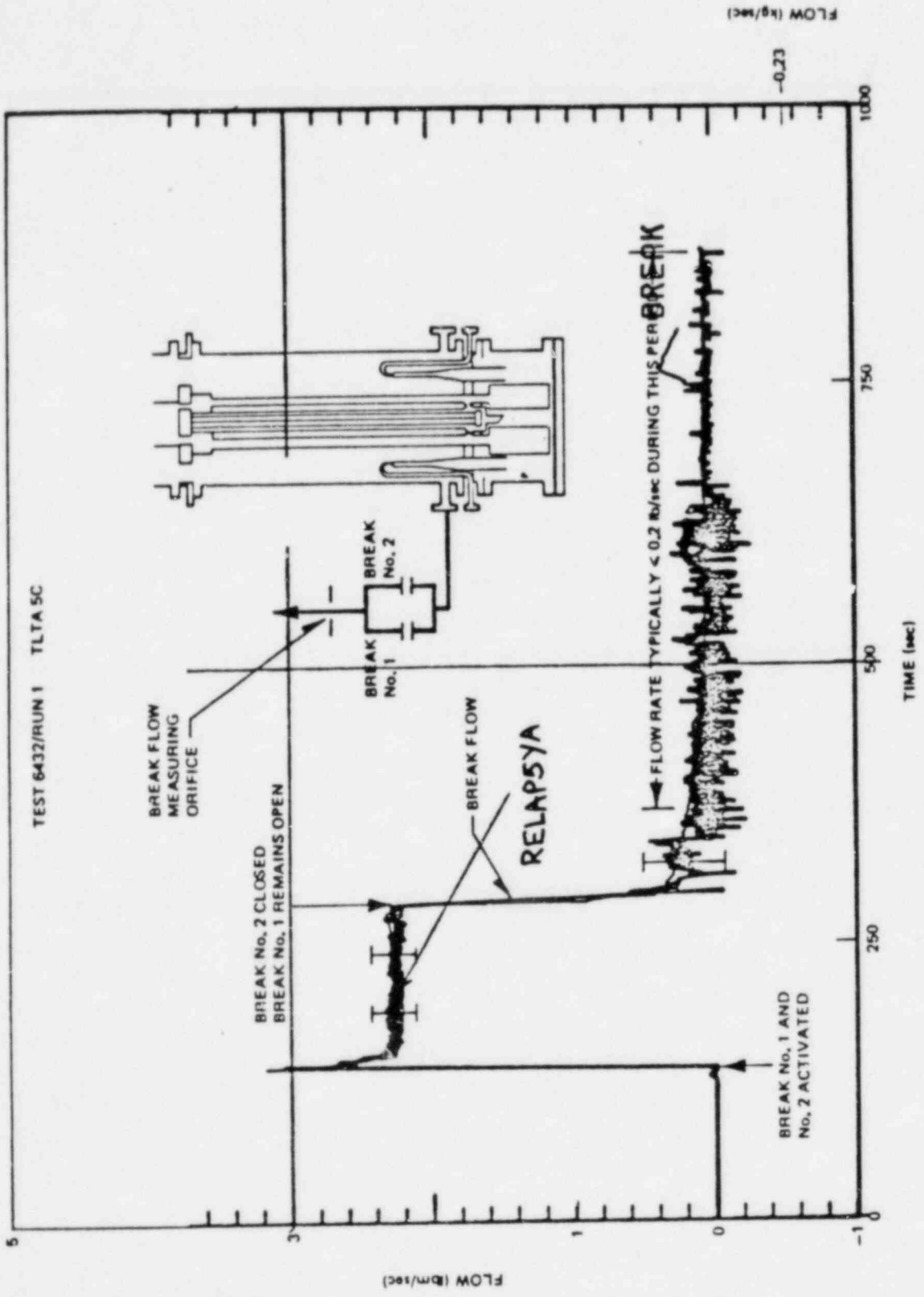


Figure D-8. Break Flow

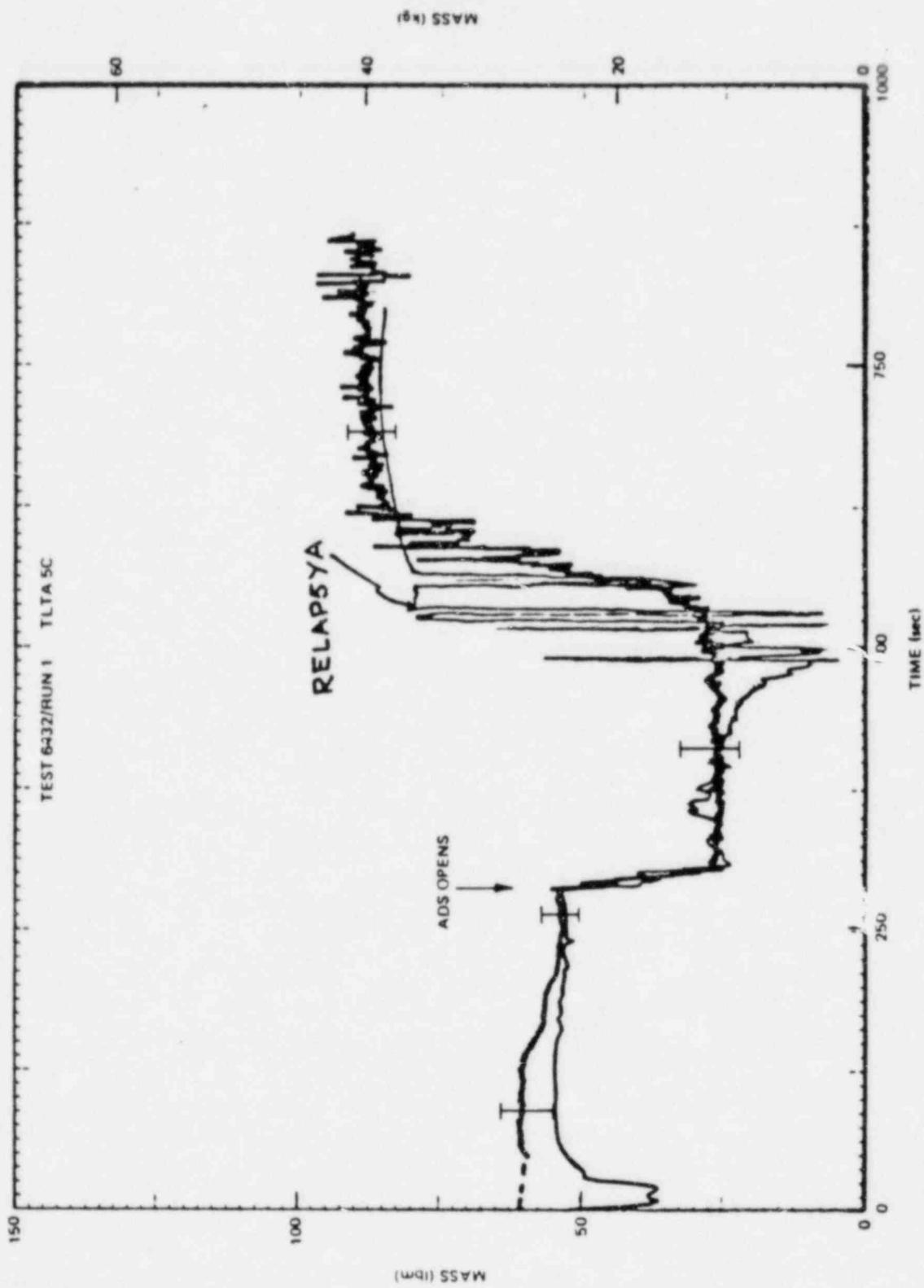


Figure D-58. Bundle Mass

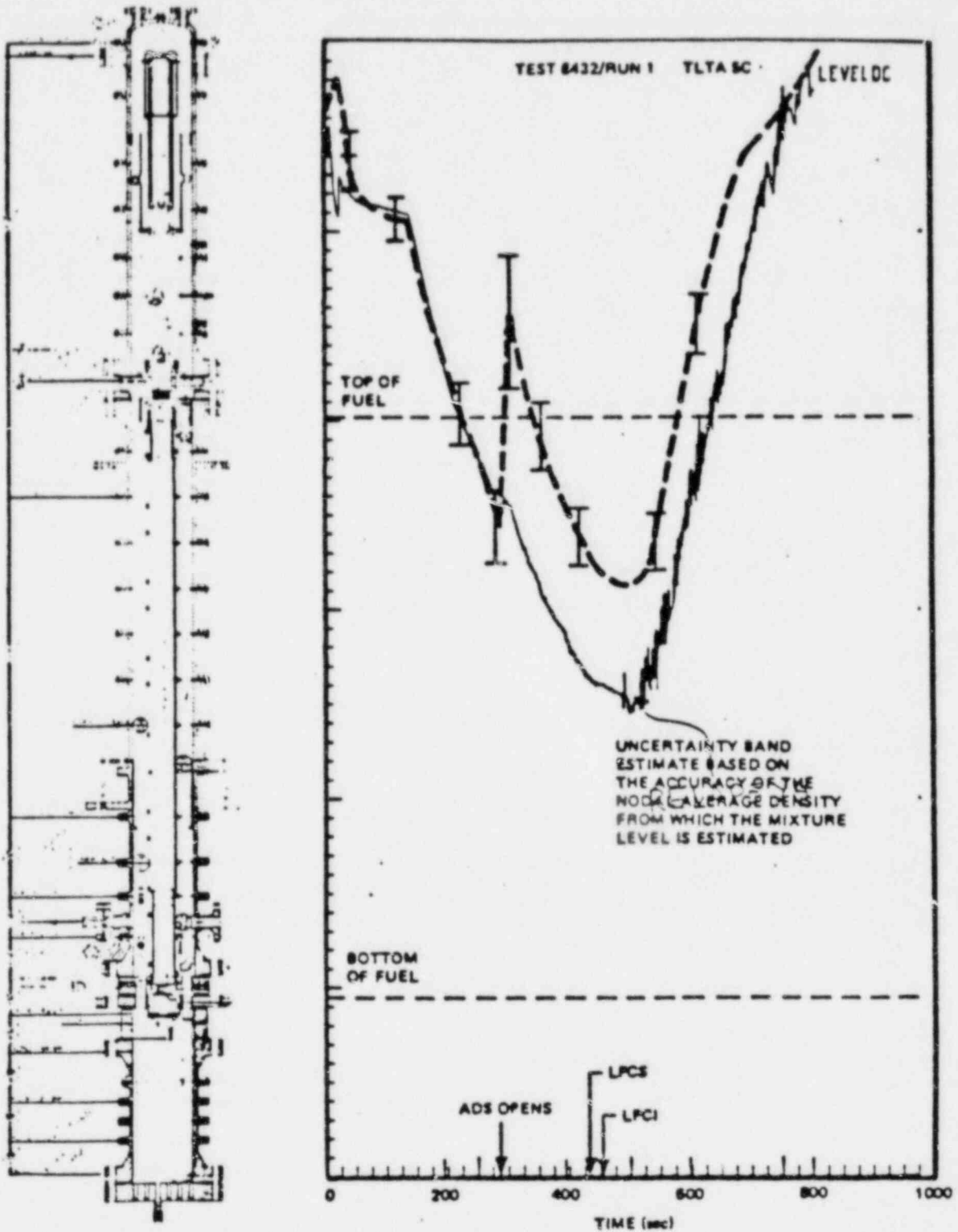


Figure D-10. Two-Phase Mixture Level - Outside the Shroud

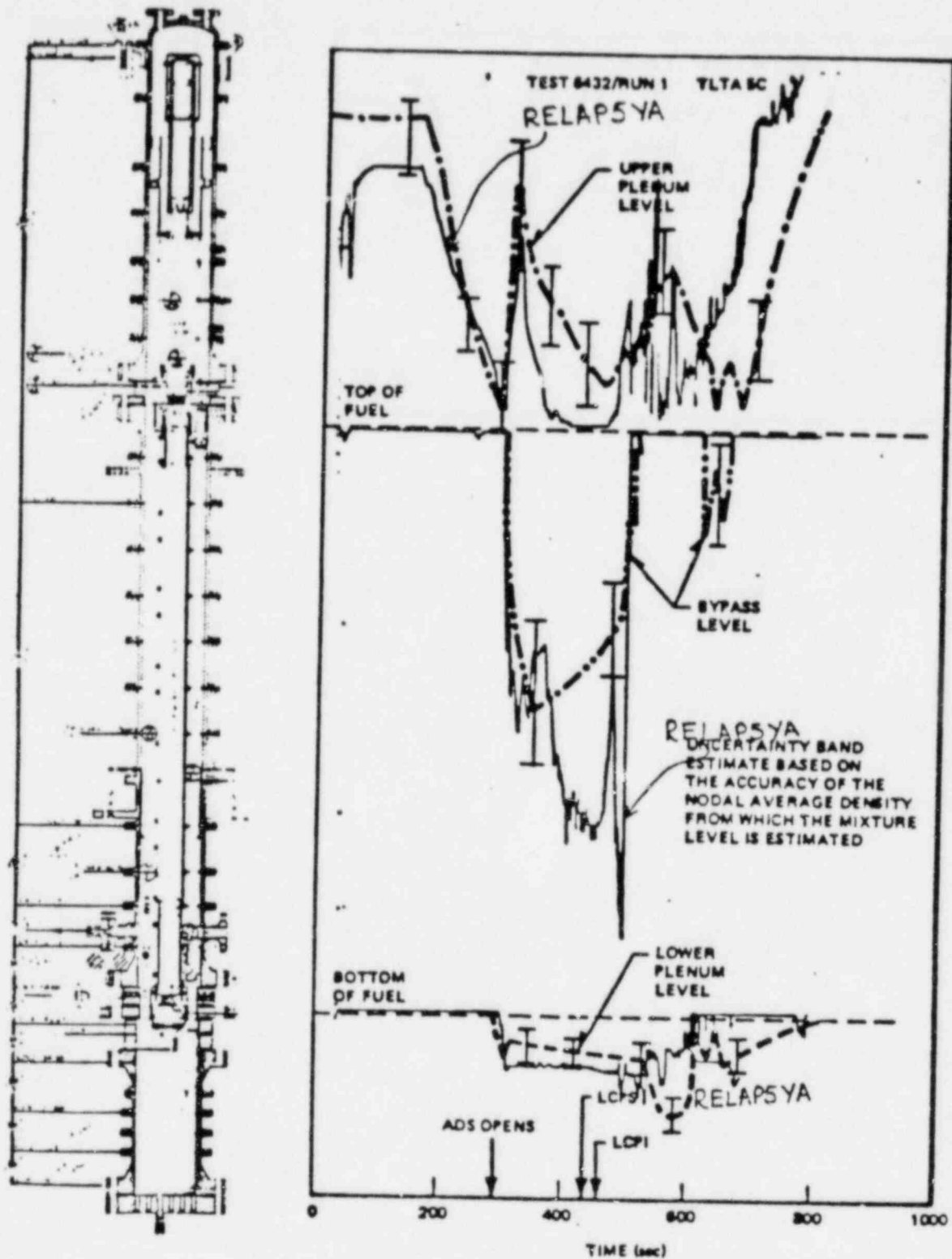


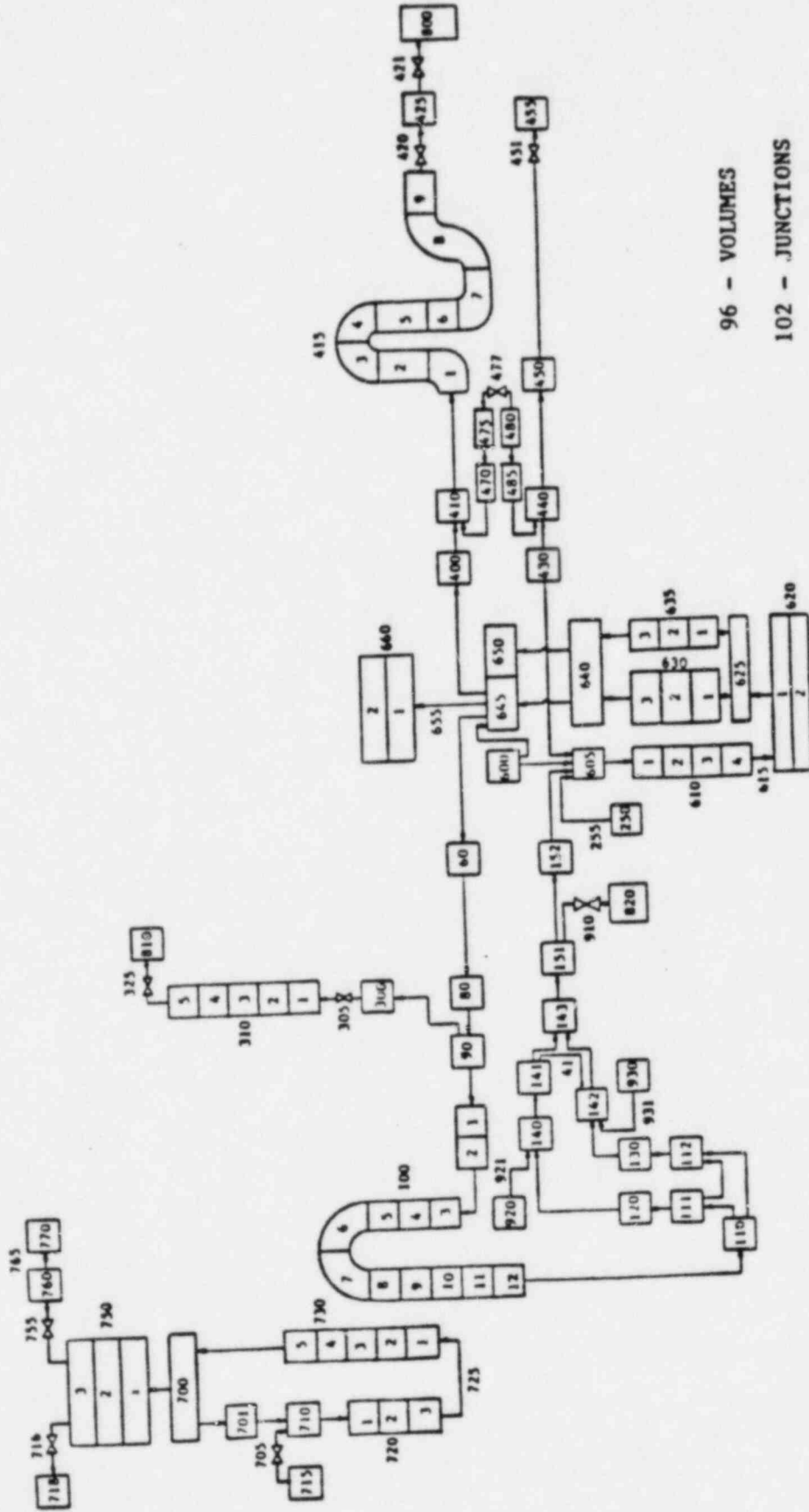
Figure D-9. Two-Phase Mixture Level - Inside the Shroud

# LOFT L3-6

- BACKGROUND
  
- MODEL DESCRIPTION
  
- COMPARISON TO DATA
  - PRESSURE
  - BREAK FLOW
  - DENSITY
  - SYSTEM INVENTORY
  
- SUMMARY



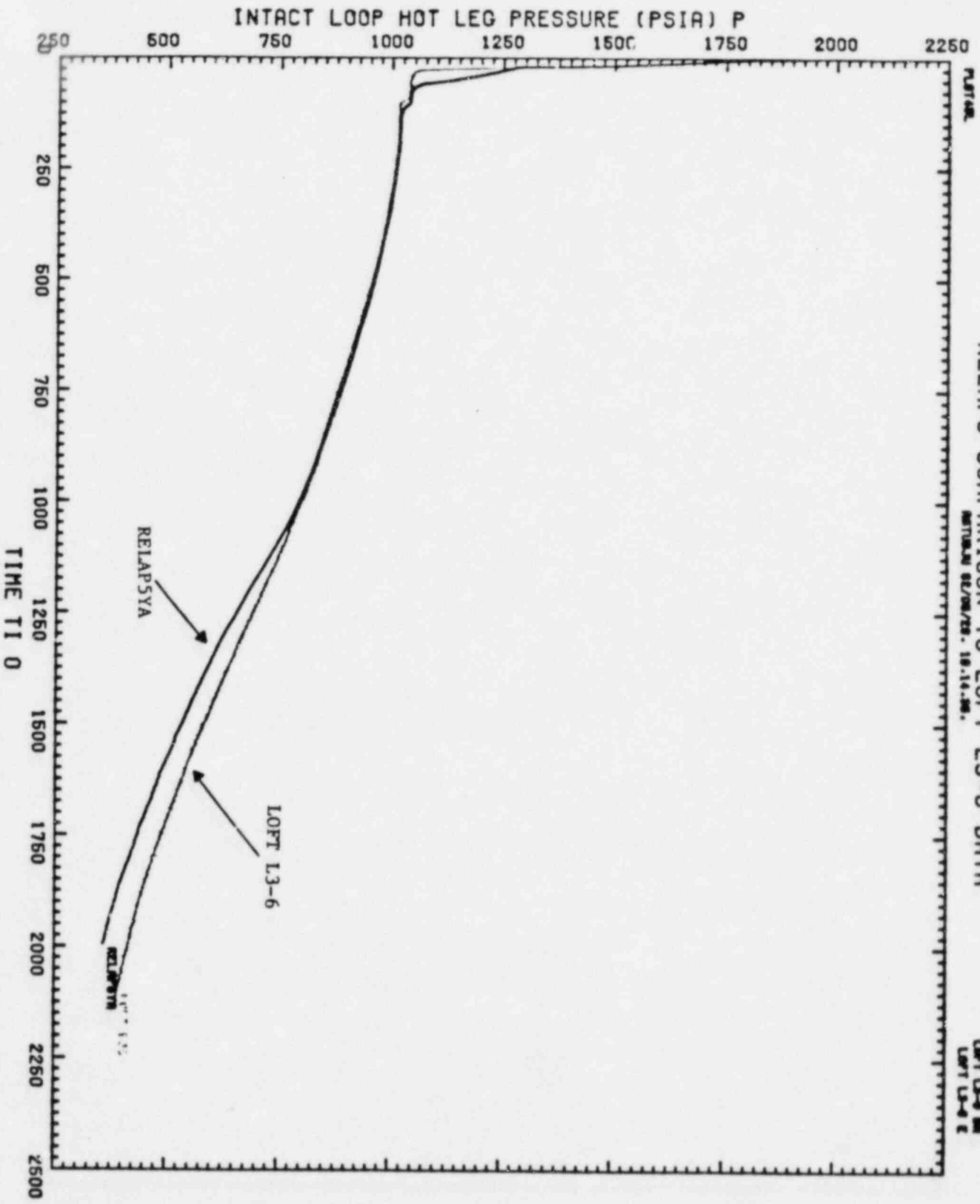
RELAPSYA LOFT L3-6 NODALIZATION



96 - VOLUMES

102 - JUNCTIONS

59 - HEAT SLABS



RELAPS COMPARISON TO LOFT L3-6 DATA

RETRIAL 02/08/78, 10:14 AM.

LOFT L3-6 88  
LOFT L3-6 2

INTACT LOOP HOT LEG PRESSURE (PSIA) P

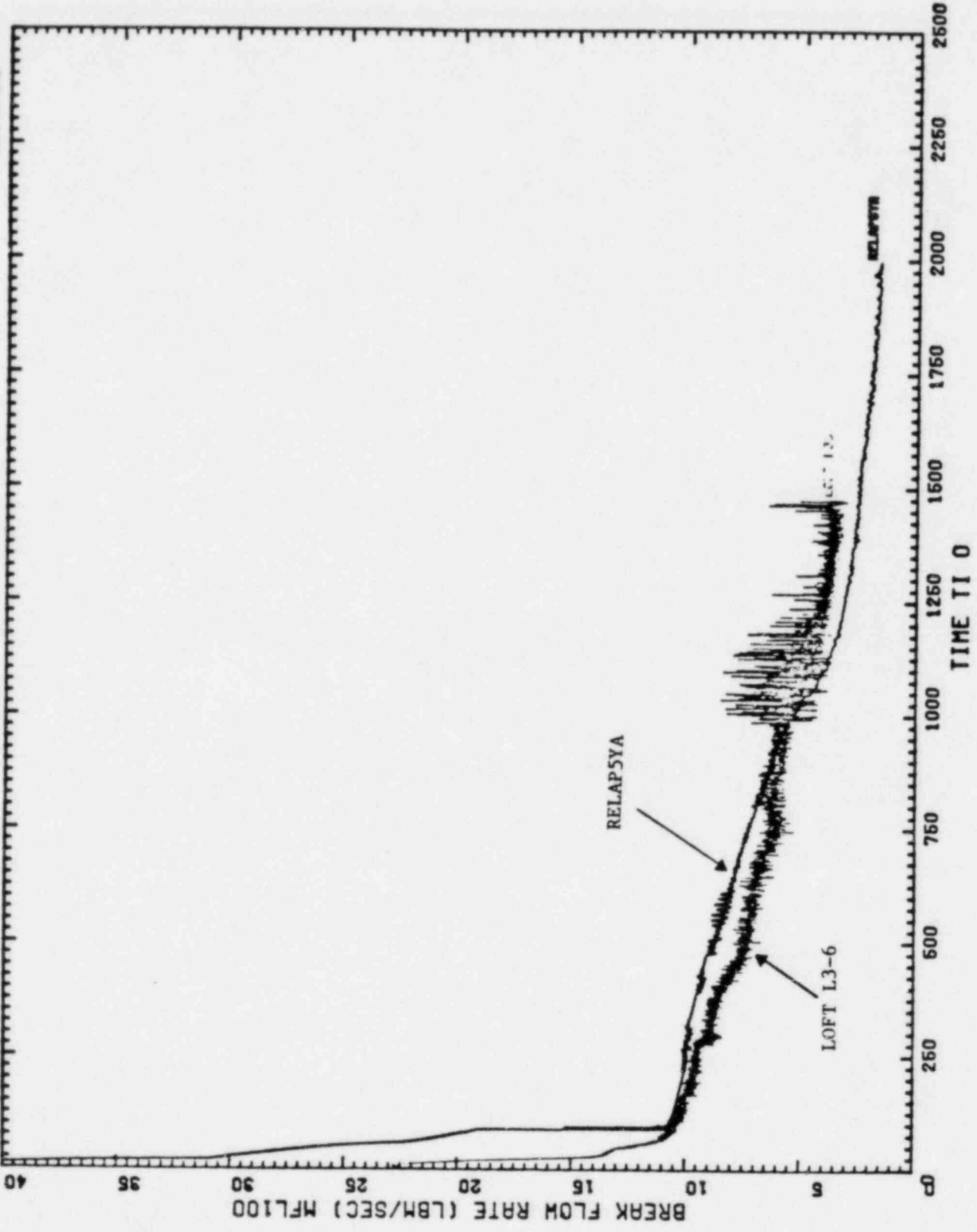
TIME TI 0

RELAPS COMPARISON TO LOFT L3-6 DATA

LOFT L3-6 DR  
LOFT L3-6 E

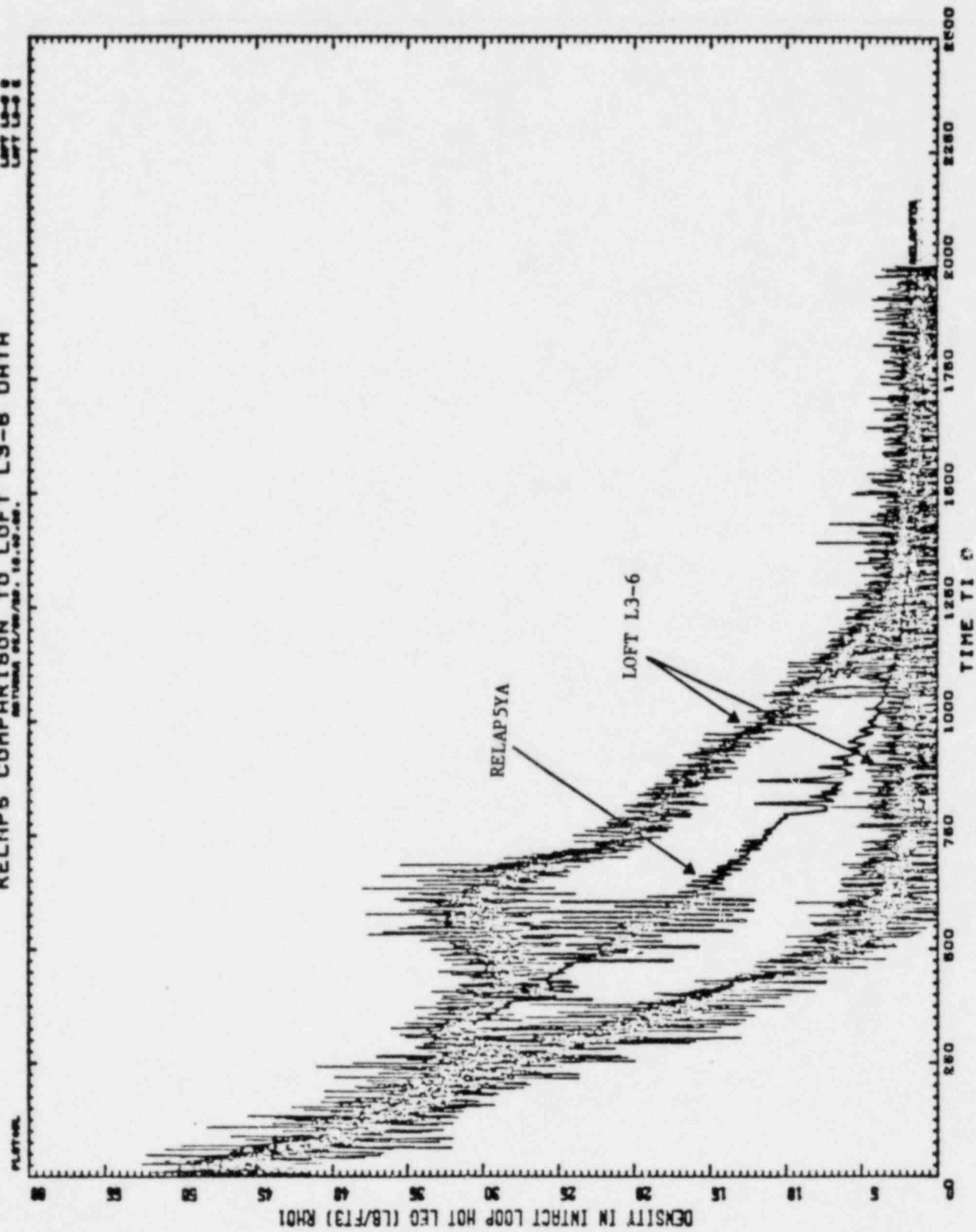
INSTRON 02/08/79, 19-1E-87.

PLUTAGL



RELAP5 COMPARISON TO LOFT L3-6 DATA

LOFT L3-6  
LOFT L3-6

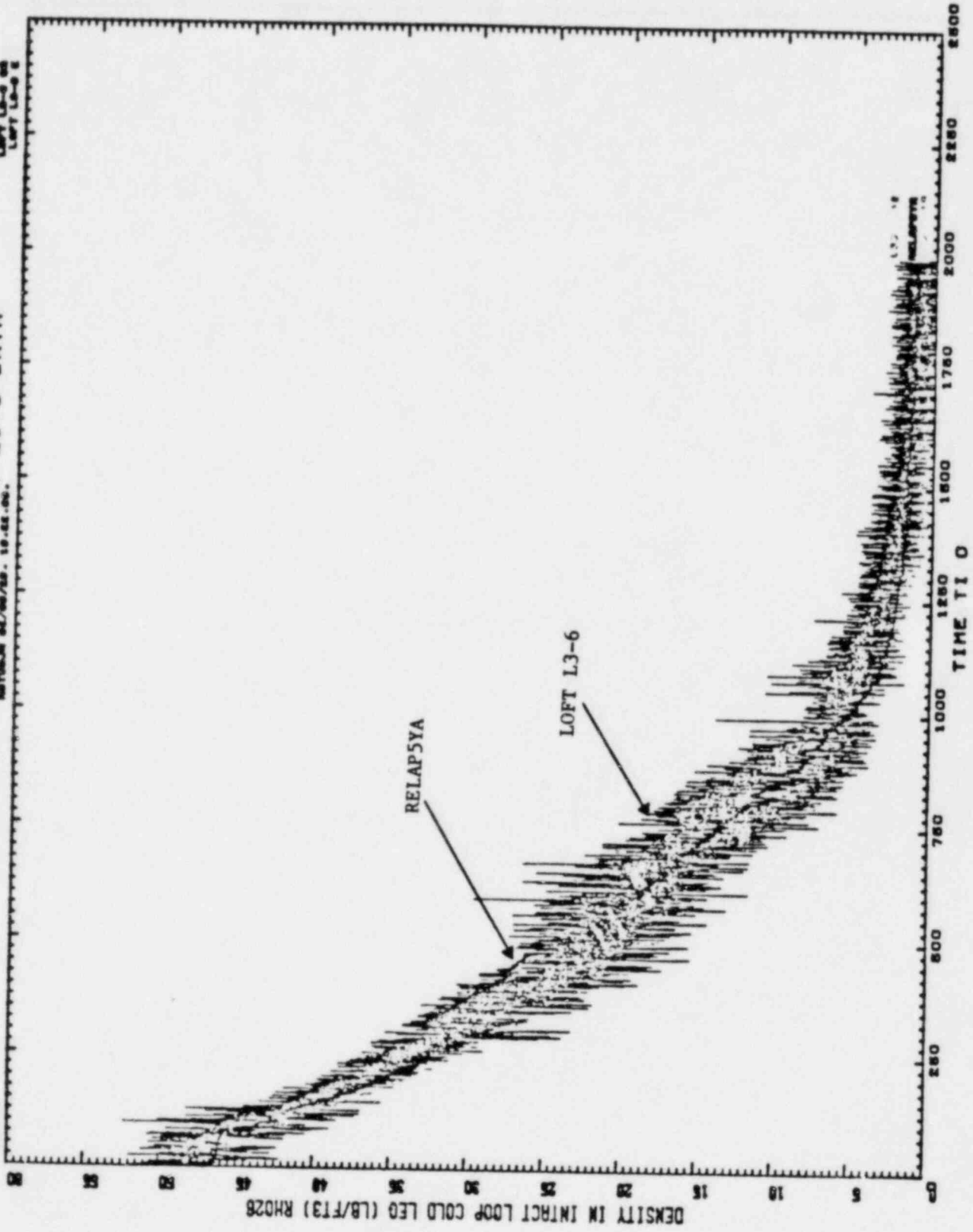


PLATT

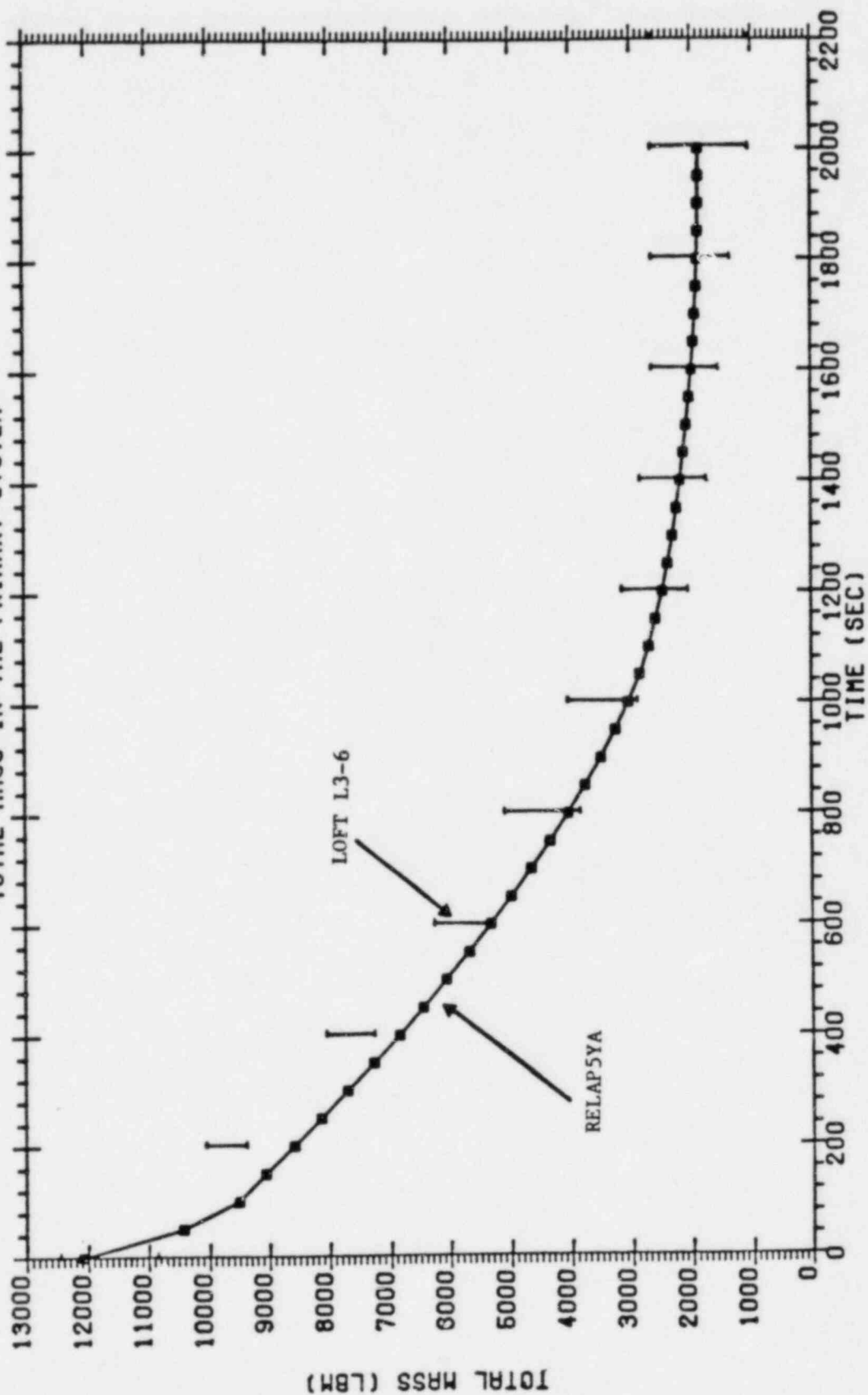
RELAP5 COMPARISON TO LOFT L3-6 DATA

LOFT L3-6 ON  
LOFT L3-6 R

PLAT 001



RELAPS COMPARISON TO LOFT L3-6 DATA  
TOTAL MASS IN THE PRIMARY SYSTEM



# SUMMARY

PARAMETER	DATA UNCERTAINTY	MAXIMUM DEVIATION BETWEEN RELAP5YA-DATA
PRIMARY PRESSURE	$\pm 37$ PSI	$\sim 50$ PSI
BREAK FLOW	unqualified ( 0 - 50 sec ) $\pm 15\%$ (50 - 1435 sec) $\pm 1.65$ LBM/SEC ( > 1435 sec)	$\sim 1$ LBM/SEC
DENSTIY	$\pm 5$ LBM/FT3	$< 5$ LBM/FT3
SYSTEM INVENTORY	$\pm 660$ LBM	$\sim 1000$ LBM early on later on very small