

Mr. Jerry W. Yelverton

- 2 -

on integral attachment Weld 59-049W, respectively, are acceptable resolutions for these requests. Our related SE is enclosed.

This action closes TAC M88190.

Sincerely,

ORIGINAL SIGNED BY:

William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555-0001

March 25, 1994

Docket No. 50-368

Mr. Jerry W. Yelverton
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

Dear Mr. Yelverton:

SUBJECT: ADDITIONAL INFORMATION CONCERNING FIRST 10-YEAR INSERVICE INSPECTION RELIEF REQUESTS B-B/3.1, C-E-1/C2.5, AND PART OF B-J/B4.5, ARKANSAS NUCLEAR ONE, UNIT 2 (ANO-2)(TAC NO. M88190)

The NRC staff, with technical assistance from its contractor, the Idaho National Engineering Laboratory (INEL), has reviewed and evaluated additional information provided by Entergy Operations, Inc., in its November 3, 1993, letter, regarding three previously evaluated relief requests for the first 10-year inservice inspection (ISI) interval. These three relief requests, from the requirements of American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, were originally submitted (along with other relief requests) by letter dated August 31, 1989. The August 31, 1989, submittal, and additional information submitted on April 30 and August 20, 1992, was evaluated in an NRC Safety Evaluation (SE) dated December 30, 1992. In the December 30, 1992, SE, the staff denied these three relief requests and stated that either the inspections be performed at the next scheduled outage of sufficient duration, or the relief requests be resubmitted with additional information.

Based on the information submitted, the staff concludes that for Relief Request B-B/B3.1, performance of the Code-required examinations for the first 10-year interval would result in a hardship without a compensating increase in quality and safety, and your proposed alternative is authorized pursuant to 10 CFR 50.55a(a)(3)(ii), provided that at least one steam generator stay cylinder weld is added to the ISI program and examined each 10-year ISI interval. For Relief Requests B-J/B4.5 and C-E-1/C2.5, the staff concludes that the Code-required examinations that will be performed on Weld 25-017 and

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Sincerely,



William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
See next page

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Entergy Operations, Inc.

Arkansas Nuclear One, Unit 2

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