

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | N Y | J A | F 1 | 2 | 0 0 - | 0 0 | 0 0 | - 0 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | _____ | 5

CON'T
0 1 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 3 | 3 | 7 | 0 | 2 | 0 | 2 | 8 | 3 | 8 | 0 | 2 | 0 | 9 | 8 | 3 | 9

0 2 | During steady-state operation, the licensee was informed that an error was made in the
0 3 | ODYN base deck for Reload 4 analysis. This error led to delta CPR values for pressur-
0 4 | ization transients non-conservative with respect to Supplemental Reload Licensing
0 5 | submittal (Y1003J01A25) values. This causes TS 3.1.B.1 and TS Figures 3.1-2a, 3.1-2b
0 6 | and 3.1-2c values to be in error. The limiting value of CPR used throughout the cycle
0 7 | has been correct since a non-pressurization event has been most limiting. The health
0 8 | and safety of the public was not degraded by the event.

0 9 | SYSTEM CODE | Z Z | 11 | CAUSE CODE | B | 12 | CAUSE SUBCODE | A | 13 | COMPONENT CODE | Z Z Z Z Z Z | 14 | COMP. SUBCODE | Z | 15 | VALVE SUBCODE | Z | 16 |
17 | LER/RO REPORT NUMBER | 8 3 | 21 | EVENT YEAR | 8 3 | 22 | SEQUENTIAL REPORT NO. | 0 0 8 | 26 | OCCURRENCE CODE | / | 27 | REPORT TYPE | T | 30 | REVISION NO. | 0 | 32 |
18 | ACTION TAKEN | X | 33 | FUTURE ACTION | X | 34 | EFFECT ON PLANT | Z | 35 | SHUTDOWN METHOD | Z | 36 | HOURS | 0 0 0 0 | 37 | ATTACHMENT SUBMITTED | Y | 41 | NPRD-4 FORM SUB. | N | 42 | PRIME COMP. SUPPLIER | Z | 43 | COMPONENT MANUFACTURER | Z 9 9 9 | 47

1 0 | CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
1 1 | Personnel error led to improper input data for Reload 4 transient analysis. Review
1 2 | of cycle history showed that limiting CPR values consistent with the new values were
1 3 | used throughout. A Technical Specification change incorporating the revised values
1 4 | will be submitted.

1 5 | FACILITY STATUS | E | 29 | % POWER | 1 0 0 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | D | 31 | DISCOVERY DESCRIPTION | Notification from NSSS Vendor | 32

1 6 | ACTIVITY CONTENT RELEASED OF RELEASE | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36

1 7 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39

1 8 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43

2 0 | PUBLICITY ISSUED DESCRIPTION | N | 44 | DESCRIPTION | NA | 45

8302230293 830209
PDR ADOCK 05000333
S PDR

David Burch

PHONE: 315-342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 83-008/01T-0

PAGE 1 of 2

On February 2, 1983, the licensee was notified by General Electric Company that they had discovered an error in the ODYN base deck for the James A. FitzPatrick Reload 4 Cycle 5 analysis. The error was that the steam carry under fraction used for the REDY code was not replaced by the value used to qualify the ODYN code. Comparative calculations indicate that delta CPR values given in the Supplemental Reload Licensing submittal for Reload 4 (Y1003J01A25) are non-conservative for pressurization transients.

Since the limiting CPR depends on the rod block monitor setting and the average scram time to position 38, an evaluation was performed to determine if the values used during the cycle to date were consistent with the new delta CPR values for pressurization transients transmitted to us by General Electric. This showed that the limiting transient to date is the Rod Withdrawal Error and the correct limiting CPR has been in use.

The corrective action for this event is to submit revisions to Technical Specification 3.1.B.1 and Technical Specification figures 3.1-2a, 3.1-2b and 3.2-2c to the NRC. Until such time as the revised Technical Specifications are approved, the plant commits to using the proposed limiting CPR values in calculations to verify compliance to thermal limits.

These values are:

3.1.B.1 MCPR Operating Limit for Incremental Cycle
 Core Average Exposure

Fuel Type	BOC to 1 GWD/t before EOC	EOC- 1 GWD/t to EOC
-----------	------------------------------	------------------------

At RBM trip level setting $S=0.66W + 39\%$

8 x 8	1-24	1-26
8 x 8R	1-24	1-26
P8 x 8R	1-25	1-28

At RBM trip level setting $S=0.66W + 40\%$

8 x 8	1-24	1-26
8 x 8R	1-24	1-26
P8 x 8R	1-25	1-28

At RBM trip level setting $S=0.66W + 41\%$

8 x 8	1-27	1-27
8 x 8R	1-27	1-27
P8 x 8R	1-27	1-28

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 83-008/01T-0

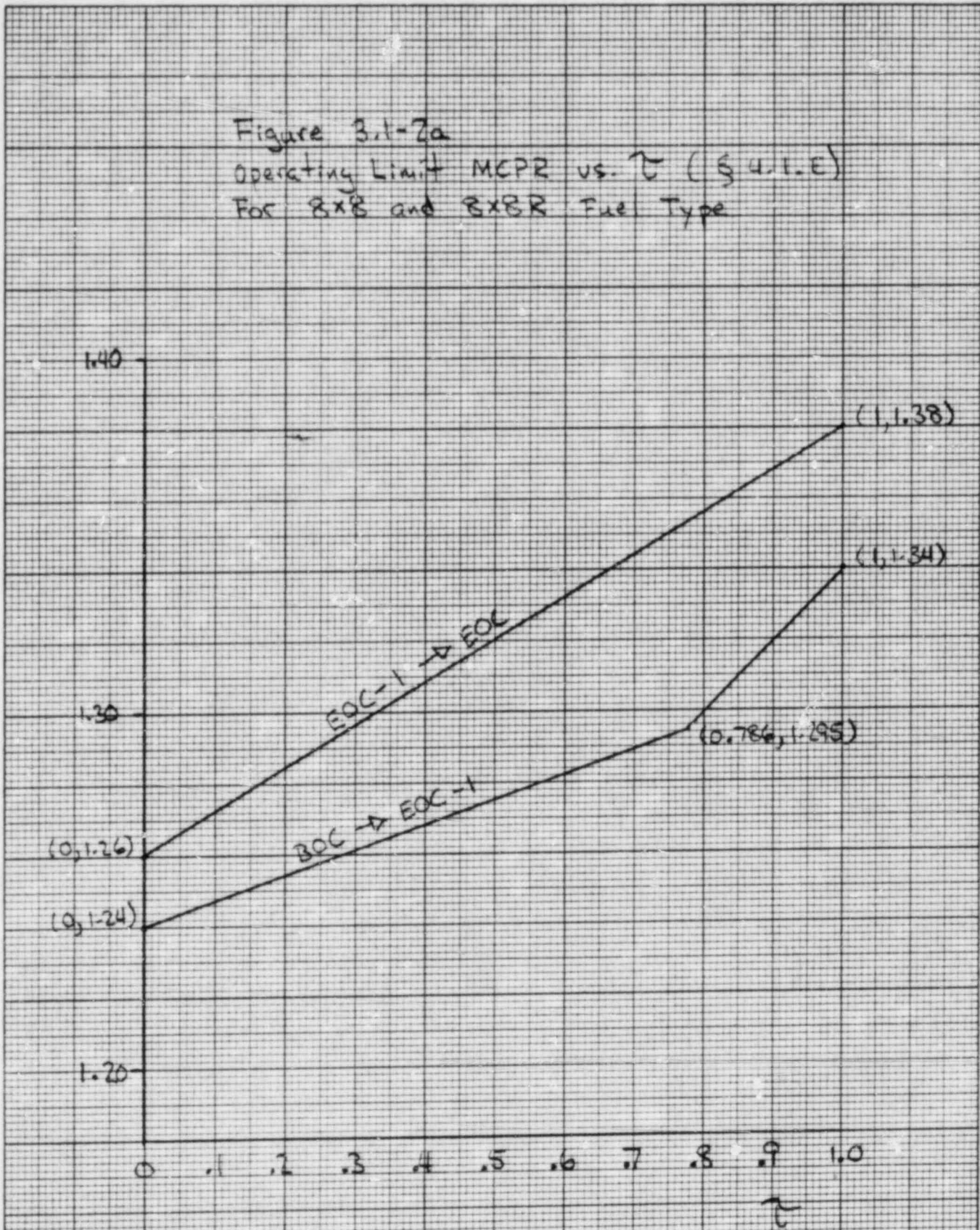
PAGE 2 of 2

At RBM trip level setting $S=0.66W + 42\%$

8 x 8	1-31	1-31
8 x 8R	1-31	1-31
P8 x 8R	1-31	1-31

(Technical Specification Figure 3.1-2c will be deleted)

Figure 3.1-2a
 Operating Limit MCPR vs. τ ($\leq 4.1.E$)
 For 8x8 and 8x8R Fuel Type

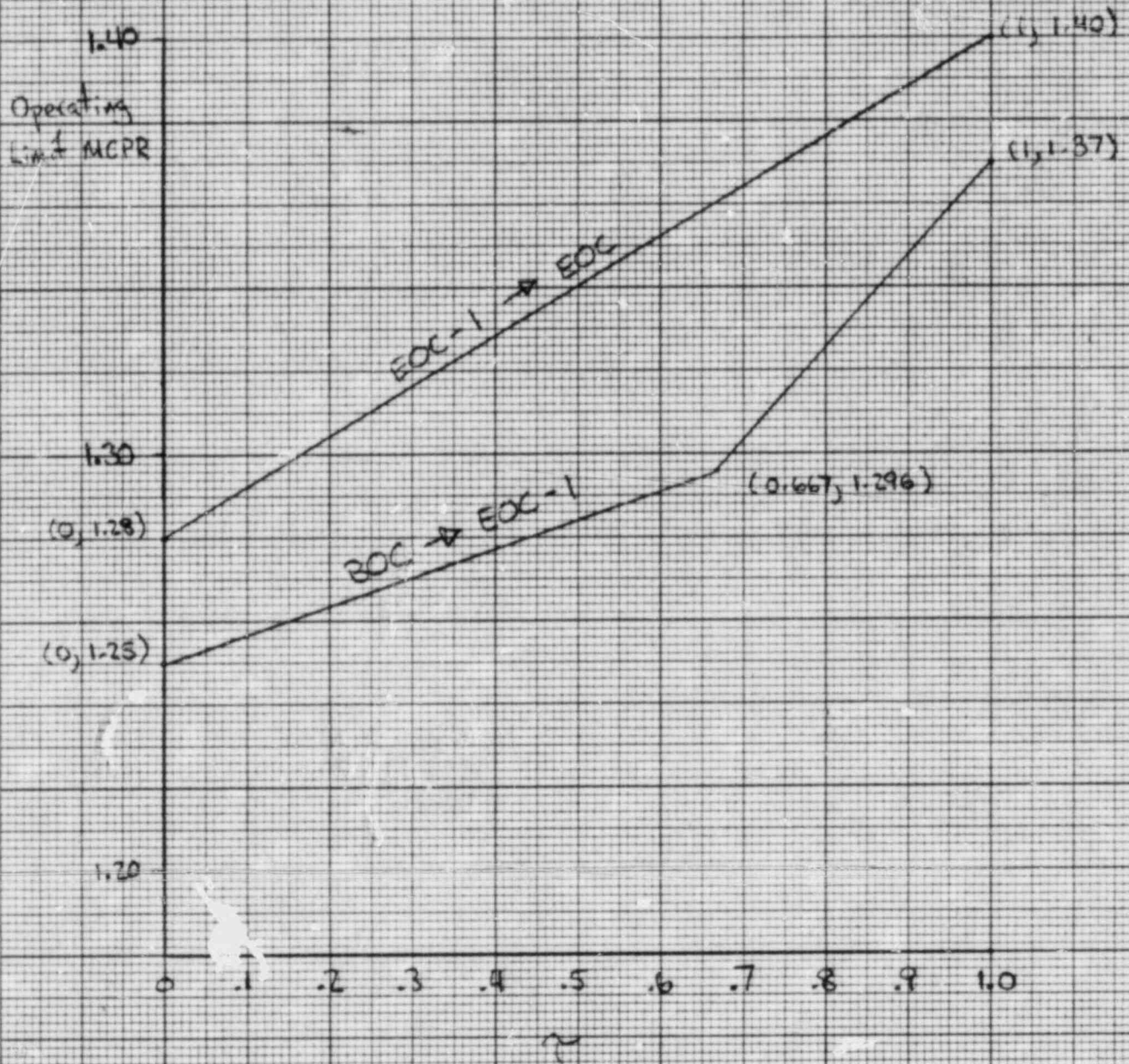


Option B $\tau=0$
 Option A $\tau=1$

461240

K·E
 20 X 20 TO THE INCH • 7 X 10 INCHES
 GEUPPEL & EBER CO. MADE IN U.S.A.

Figure 3.1-2b
Operating Limit MCPR vs. τ (§ 4.1-E)
FOR PBXER Fuel Type



Option B $\tau=0$
Option A $\tau=1$