

September 02, 1982

Docket No. 50-155  
LS05-32-09-015

Mr. David J. Vandewalle  
Nuclear Licensing Administrator  
Consumers Power Company  
1945 W. Parnall Road  
Jackson, Michigan 49201

Dear Mr. Vandewalle:

SUBJECT: SEP TOPIC VII-1.A, ISOLATION OF REACTOR PROTECTION SYSTEM  
FROM NON-SAFETY SYSTEMS, INCLUDING QUALIFICATION OF  
ISOLATION DEVICES - FINAL SAFETY EVALUATION REPORT FOR  
BIG ROCK POINT NUCLEAR POWER PLANT

Our draft safety evaluation report (SER) for this topic was forwarded by a letter from D. M. Crutchfield to D. J. Vandewalle dated March 5, 1982. Our evaluation identified a generic issue that has been resolved for all General Electric Boiling Water Reactors with the exception of Big Rock Point. The essence of this concern is that a sustained voltage or frequency transient in the RPS power supply (motor generator or alternate feed) could overheat half of the scram valves and prevent a scram. You were requested to provide (1) your commitment to install a Class 1E system, (2) your schedule for completion of the modification and (3) your schedule for submission of design information and proposed Technical Specifications.

Your letter of June 22, 1982, which provided comments on our draft SER, has been reviewed. Enclosure 1 is our contractor's final technical evaluation of this topic for Big Rock Point. Enclosure 2 is the staff's final SER that is based on Enclosure 1.

The staff concludes that suitable isolation devices exist in the Reactor Protection System with the exception of possible effects from the motor generator sets and alternate feed.

The staff's position is that suitably qualified isolators should be provided. The isolators should protect Class 1E loads against voltage regulator failure in both high and low voltage mode and frequency degradation resulting from motor generator set bearing and prime mover drive frequency degradation.

SE04  
DSU USE (18)

ADD:  
R. Scholl

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PDR ADOCK 05000155  
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OFFICE							
SURNAME							
DATE							

D. J. Vandewalle

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This safety evaluation may be revised in the future if your facility design is changed or if NRC criteria relating to this topic are modified before the Integrated Assessment is completed.

Sincerely,

*Original signed by:*

Dennis M. Crutchfield, Chief  
Operating Reactors Branch No. 5  
Division of Licensing

Enclosures:  
As stated

cc w/enclosures:  
See next page

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DATE	8/25/82	8/26/82	8/27/82		8/31/82	8/31/82	8/17/82

Mr. David J. Vandewalle

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