

ATTACHMENT 3 to TXX-94057

AFFECTED TECHNICAL SPECIFICATION PAGE  
(NUREG - 1468)

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

- 5). WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL  $F_0$  SURVEILLANCE TECHNICAL SPECIFICATION," June 1983 (W Proprietary).  
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor (W(z) surveillance requirements for  $F_0$  Methodology).)
- 6). WCAP-10079-P-A, "NOTRUMP, A NODAL TRANSIENT SMALL BREAK AND GENERAL NETWORK CODE," August 1985, (W Proprietary).
- 7). WCAP-10054-P-A, "WESTINGHOUSE SMALL BREAK ECCS EVALUATION MODEL USING THE NOTRUMP CODE", August 1985, (W Proprietary).
- 8). WCAP-11145-P-A, "WESTINGHOUSE SMALL BREAK LOCA ECCS EVALUATION MODEL GENERIC STUDY WITH THE NOTRUMP CODE", October 1986, (W Proprietary).
- 3). RXE-90-006-P, "Power Distribution Control Analysis and Overtemperature N-16 and Overpower N-16 Trip Setpoint Methodology," February 1991.  
(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor.)
- 10). RXE-88-102-P, "TUE-1 Departure from Nucleate Boiling Correlation", January 1989.
- 11). RXE-88-102-P, Sup. 1, "TUE-1 DNB Correlation - Supplement 1", December 1990.
- 12). RXE-89-002, "VIPRE-01 Core Thermal-Hydraulic Analysis Methods for Comanche Peak Steam Electric Station Licensing Applications", June 1989.
- 13). RXE-91-001, "Transient Analysis Methods for Comanche Peak Steam Electric Station Licensing Applications", February 1991.
- 14). RXE-91-002, "Reactivity Anomaly Events Methodology", May 1991.  
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limit, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
- 15). RXE-90-007, "Large Break Loss of Coolant Accident Analysis Methodology", December 1990.
- 16). TXX-88306, "Steam Generator Tube Rupture Analysis", March 15, 1988.

Reference 17) is for Unit 1 only:

- 17). WCAP-9220-P-A, "WESTINGHOUSE ECCS EVALUATION MODEL, February 1978 Version," February 1978 (W Proprietary).

RXE-91-005, "Methodology for Reactor Core Response to Steamline Break Events," May, 1991.