

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

AUGUST 1982

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

- I. Introduction
- II. Summary of Operating Experience
 - A. Unit One
 - B. Unit Two
- III. Plant or Procedure Changes, Tests, Experiments, and Safety Related Maintenance
 - A. Amendments to Facility License or Technical Specifications
 - B. Facility or Procedure Changes Requiring NRC Approval
 - C. Tests and Experiments Requiring NRC Approval
 - D. Corrective Maintenance of Safety Related Equipment
- IV. Licensee Event Reports
- V. Data Tabulations
 - A. Operating Data Report
 - B. Average Daily Unit Power Level
 - C. Unit Shutdown and Power Reductions
- VI. Unique Reporting Requirements
 - A. Main Steam Relief Valve Operations
 - B. Control Rod Drive Scram Timing Data
- VII. Refueling Information
- VIII. Glossary

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2, respectively, were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973, for Unit 1 and March 10, 1973, for Unit 2.

This report was compiled by Becky Brown and Randall Buss, telephone number 309-654-2241, extensions 127 and 181.

II. SUMMARY OF OPERATING EXPERIENCE

A. UNIT ONE

August 1-8: The unit began the month derated due to end of cycle fuel depletion/coastdown. On August 1 and again on August 2, load was reduced to approximately 225 MWe at the request of the Load Dispatcher, due to low system demand. On August 3, the unit dropped load to 400 MWe due to low system demand. At all other times an average load of 440 MWe was maintained.

August 9-15: On six occasions, Unit One was requested to reduce load by about 200 MWe due to low system demand between Midnight and approximately 0700 hours. An average load of 450 MWe was maintained at all other times.

August 16-21: On August 16, unit load was reduced, due to low system demand, to 225 MWe beginning 0300 hours, at 200 MWe/hour for one hour. At 0630 hours, load was increased to 405 MWe at 100 MWe/hour. At 1045 hours, load dropped 200 MWe/hour to 325 MWe to enable the 1A Condensate Booster pump to be taken out of service to repair a leak. Load was started to increase at 1135 hours to 430 MWe. On three occasions, load was reduced between Midnight and 0530 hours to approximately 220 MWe due to low system demand. Normal load increases followed.

August 22-27: On August 22 at 0115 hours, the Load Dispatcher requested a load reduction to 200 MWe due to low system demand. At 0700 hours, load was increased to 445 MWe on August 23. On three occasions, load was reduced due to low system demand between Midnight and 0500 hours to approximately 220 MWe, followed by a normal increase.

August 28-31: A unit load reduction was requested five times during this period due to low system demand.

B. UNIT TWO

August 1-6: The unit began the month holding load at approximately 680 MWe. The unit is derated to 780 MWe due to high vibration in the 2A Recirculation pump Motor-Generator Set. At 0230 hours on August 1, due to low system demand, load was dropped to 480 MWe by 0430 hours. After two hours, load was increased to 680 MWe by 0200 hours on August 2. At 1900 hours, load was dropped to about 610 MWe to lower the cooling water temperature. Load was increased to 700 MWe on August 6.

August 7-12: On August 7, at 0045 hours, load was decreased at 100 MWe/hour to 170 MWe to take the 2A Circulating Water pump out of service to allow a diver to inspect the traveling screens. At 1810 hours the pump was returned to service and load was increased in accordance with normal preconditioning ramps to 744 MWe by 1800 hours on August 9.

B. UNIT TWO (Continued)

August 13-31: On ten occasions, due to low system demand, the Load Dispatcher requested load reductions on Unit Two. At all other times the unit maintained an average load of approximately 720 MWe. On August 15 and 28, load was reduced to perform weekly Turbine tests.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On June 9, 1982, Amendments 80 and 74 were added to licenses DPR-29 and DPR-30, respectively. This Amendment adds the requirement for a simulated actuation of the automatic pressure relief system which opens all pilot valves each operating cycle in lieu of every six months. This Amendment also changed the testing frequency of the Automatic Depressurization System to immediately, but not daily thereafter, provided the required number of Feedwater pumps are operating or available when HPCI is declared inoperable.

On July 9, 1982, Amendments 79 and 73 were added to licenses DPR-29 and DPR-30, respectively. This Amendment changes 5.5.B to read: The K_{eff} of the spent fuel storage pool shall be less than or equal to 0.95."

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure Changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the safety related maintenance performed on Unit One and Unit Two during the reporting period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W. R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q20606		CRD Module 42-19	The accumulator piston seals were worn.	Water was leaking past the piston.	The accumulator was replaced.
Q20674		1A 125V Battery Charger 1A-8300	The float voltage on the charger had drifted high.	The AC breaker will not stay closed.	The float voltage was adjusted to 130.8 volts.
Q21258	82-20/03L	1/2A SBT 1/2-7503A	The temperature switch had tripped.	The heater won't turn on when system is running.	The temperature switch was reset and the system was tested.
Q20873	82-21/03L	Room Cooler Valve 1-1001- 188D	The valve was binding.	The cooling water valve could not be closed.	The valve parts were lubricated.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q20592		2B RHR Loop Injection Valve 2-1001-29B	A lead had come loose from the torque switch.	The valve will not open using the control switch.	Both the lead and the torque switch were replaced.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>UNIT ONE</u>		
<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-21/03L	8-1-82	1D RHR Service Water Pump Inoperable
82-22/03L	8-9-82	1/2B Diesel Fire Pump
82-23/03L	8-24-82	1A RHR Service Water Pump Out of Service to Work on Room Cooler
82-24/03L	8-26-82	1D RHR Service Water Pump Out of Service to Work on Room Cooler
<u>UNIT TWO</u>		
82-14/03L	8-2-82	2C RHR Service Water Pump Packing Blown
82-15/03L	8-4-82	Diesel Generator #2 Inoperable Greater Than 90 Minutes
82-16/03L	7-29-82	2B RHR Service Water Pump Inoperable

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE September 1 1982

COMPLETED BY Randall Buss

TELEPHONE 309-654-2241x181

OPERATING STATUS

0000 080182

1. Reporting period: 2400 083182 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>5712.4</u>	<u>74811.5</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>744.0</u>	<u>5656.9</u>	<u>71788.4</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>966928</u>	<u>10858341</u>	<u>145916700</u>
10. Gross electrical energy generated (MWH)	<u>290794</u>	<u>3489624</u>	<u>47018557</u>
11. Net electrical energy generated (MWH)	<u>254117</u>	<u>3185345</u>	<u>43769429</u>
12. Reactor service factor	<u>100.0</u>	<u>98.0</u>	<u>82.8</u>
13. Reactor availability factor	<u>100.0</u>	<u>98.0</u>	<u>86.6</u>
14. Unit service factor	<u>100.0</u>	<u>97.0</u>	<u>79.4</u>
15. Unit availability factor	<u>100.0</u>	<u>97.0</u>	<u>80.5</u>
16. Unit capacity factor (Using MDC)	<u>44.4</u>	<u>71.0</u>	<u>63.0</u>
17. Unit capacity factor (Using Des. MWe)	<u>43.3</u>	<u>69.2</u>	<u>61.4</u>
18. Unit forced outage rate	<u>0.0</u>	<u>1.5</u>	<u>6.7</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE September 1 1982

COMPLETED BY Randall Buss

TELEPHONE 309-654-2241x181

OPERATING STATUS

0000 080182

1. Reporting period: 2400 083182 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>4671.0</u>	<u>69522.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>744.0</u>	<u>4629.9</u>	<u>66871.1</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>1602567</u>	<u>10540455</u>	<u>138427538</u>
10. Gross electrical energy generated (MWH)	<u>496972</u>	<u>3354149</u>	<u>44060389</u>
11. Net electrical energy generated (MWH)	<u>473605</u>	<u>3193529</u>	<u>41318113</u>
12. Reactor service factor	<u>100.0</u>	<u>80.1</u>	<u>77.7</u>
13. Reactor availability factor	<u>100.0</u>	<u>80.1</u>	<u>81.1</u>
14. Unit service factor	<u>100.0</u>	<u>79.4</u>	<u>74.8</u>
15. Unit availability factor	<u>100.0</u>	<u>79.4</u>	<u>75.6</u>
16. Unit capacity factor (Using MDC)	<u>82.8</u>	<u>71.2</u>	<u>60.1</u>
17. Unit capacity factor (Using Des. MWe)	<u>80.7</u>	<u>69.4</u>	<u>58.5</u>
18. Unit forced outage rate	<u>0.0</u>	<u>19.8</u>	<u>9.3</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE September 1 1982

COMPLETED BY Randall Buss

TELEPHONE 309-654-2241x181

MONTH August 1982

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>370.5</u>
2.	<u>372.0</u>
3.	<u>378.9</u>
4.	<u>392.0</u>
5.	<u>390.3</u>
6.	<u>362.2</u>
7.	<u>382.3</u>
8.	<u>382.0</u>
9.	<u>364.9</u>
10.	<u>364.1</u>
11.	<u>363.8</u>
12.	<u>327.1</u>
13.	<u>340.5</u>
14.	<u>332.5</u>
15.	<u>338.1</u>
16.	<u>335.2</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>348.0</u>
18.	<u>333.7</u>
19.	<u>323.7</u>
20.	<u>361.8</u>
21.	<u>359.0</u>
22.	<u>235.1</u>
23.	<u>354.2</u>
24.	<u>353.3</u>
25.	<u>333.4</u>
26.	<u>339.1</u>
27.	<u>327.6</u>
28.	<u>221.1</u>
29.	<u>249.4</u>
30.	<u>333.4</u>
31.	<u>319.1</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE September 1 1982

COMPLETED BY Randall Buss

TELEPHONE 309-654-2241x181

MONTH August 1982

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY AVERAGE DAILY POWER LEVEL (MWe-Net)
1. <u>571.4</u>	17. <u>664.4</u>
2. <u>637.7</u>	18. <u>635.7</u>
3. <u>562.7</u>	19. <u>601.5</u>
4. <u>592.2</u>	20. <u>671.4</u>
5. <u>628.5</u>	21. <u>665.8</u>
6. <u>628.4</u>	22. <u>579.8</u>
7. <u>265.5</u>	23. <u>698.5</u>
8. <u>569.0</u>	24. <u>687.3</u>
9. <u>706.1</u>	25. <u>695.1</u>
10. <u>728.8</u>	26. <u>715.3</u>
11. <u>690.7</u>	27. <u>694.2</u>
12. <u>688.3</u>	28. <u>632.2</u>
13. <u>668.0</u>	29. <u>542.9</u>
14. <u>623.6</u>	30. <u>706.0</u>
15. <u>661.3</u>	31. <u>643.8</u>
16. <u>677.5</u>	

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254UNIT NAME Quad-Cities Unit 1COMPLETED BY Randall D BussDATE September 7, 1982REPORT MONTH AUGUST 1982TELEPHONE 309-654-2241
extension 181

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-56	820801	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-57	820802	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-58	820803	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-59	820809	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-60	820810	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-61	820812	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-62	820813	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-63	820814	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-64	820815	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-65	820816	S	0.0	F	5		EA	ZZZZZZ	" " " " "

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AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY Randall D Buss

DATE September 7, 1982

REPORT MONTH AUGUST 1982

TELEPHONE 309-654-2241
extension 181

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-66	820816	F	0.0	B	5		HH	PUMPXX	Reduced load to take the 1A Condensate Booster Pump out of service for repairs
82-67	820817	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-68	820818	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-69	820819	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-70	820822	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-71	820822	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-72	820825	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-73	820826	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-74	820827	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-75	820828	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-76	820828	S	0.0	F	5		EA	ZZZZZZ	" " " " "

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AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY Randall D Buss

DATE September 7, 1982

REPORT MONTH AUGUST 1982

TELEPHONE 309-654-2241
extension 181

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-77	820828	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-78	820829	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-79	820831	S	0.0	F	5		EA	ZZZZZZ	" " " " "

APPROVED
AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit 2COMPLETED BY Randall D BussDATE September 7, 1982REPORT MONTH AUGUST 1982TELEPHONE 309-654-2241
extension 181

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-51	820801	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-52	820802	F	0.0	H	5		WA	HTEXCH	Load reduced to lower cooling water temperature
82-53	820807	F	0.0	B	5		HF	PUMPXX	Reduced load to take the 2A Circulating Water Pump out of service for screen inspection
82-54	820813	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-55	820814	S	0.0	F	5		EA	ZZZZZZ	" " " " " "
82-56	820815	S	0.0	B	5		HA	XXX:XX	Reduced load to perform weekly Turbine tests
82-57	820815	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-58	820817	S	0.0	F	5		EA	ZZZZZZ	" " " " " "
82-59	820818	S	0.0	F	5		EA	ZZZZZZ	" " " " " "
82-60	820819	S	0.0	F	5		EA	ZZZZZZ	" " " " " "

APPROVED
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AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit 2COMPLETED BY Randall D BussDATE September 7, 1982REPORT MONTH AUGUST 1982TELEPHONE 309-654-2241
extension 181

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
82-61	820822	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Dispatcher due to low system demand
82-62	820828	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine tests
82-63	820828	S	0.0	F	5		EA	ZZZZZZ	Load reduction requested by Load Disptacher due to low system demand
82-64	820829	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-65	820829	S	0.0	F	5		EA	ZZZZZZ	" " " " "
82-66	820831	S	0.0	F	5		EA	ZZZZZZ	" " " " "

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AUG 16 1982

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information," dated January 18, 1978

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: 1 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: Sept 12, 1982
3. Scheduled date for restart following refueling: Dec 4, 1982
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
YES
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
JULY 26, 1982
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

IMPLEMENTATION OF THE ODYN TRANSIENT ANALYSIS CODE AND RESULTS
(MCPR SCRAM TIME DEPENDENCE)

7. The number of fuel assemblies.
a. Number of assemblies in core: 224 new/724 total
b. Number of assemblies in spent fuel pool: 1940 after the outage
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 2920
b. Planned increase in licensed storage: 4636 new/7556 total
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

LOSS OF FULL CORE DISCHARGE CAPABILITY - 3/84
LOSS OF RELOAD CORE DISCHARGE CAPABILITY - 2/86

A P P R O V E D

APR 20 1978

Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: 2 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: Feb 27, 1983
3. Scheduled date for restart following refueling: April 23, 1983
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
NO
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NONE
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE

7. The number of fuel assemblies.

- a. Number of assemblies in core: 192 new/724 total
b. Number of assemblies in spent fuel pool: 2132 after the outage

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

- a. Licensed storage capacity for spent fuel: 2920
b. Planned increase in licensed storage: 4636 new/7556 total

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

LOSS OF FULL CORE DISCHARGE CAPABILITY - 3/84
LOSS OF RELOAD CORE DISCHARGE CAPABILITY - 2/86

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VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traveling Incore Probe
TSC	-	Technical Support Center