

OPERATING DATA REPORT

DOCKET NO. 050-0331

DATE 2-14-83

COMPLETED BY Mark Watson

TELEPHONE 319-851-7611

OPERATING STATUS

Notes

1. Unit Name Duane Arnold Energy Center
2. Reporting Period January, 1983
3. Licensed Thermal Power (Mwt): 1658
- \*4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since the Last Report, Give Reasons:  
\_\_\_\_\_  
\_\_\_\_\_
9. Power Level to Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>70128.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>744.0</u>	<u>51000.3</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>49679.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>886231.2</u>	<u>886231.2</u>	<u>61255751.2</u>
17. Gross Electrical Energy Generated (MWH)	<u>304347.0</u>	<u>304347.0</u>	<u>20519264.0</u>
18. Net Electrical Energy Generated (MWH)	<u>283698.2</u>	<u>283698.2</u>	<u>19197970.6</u>
19. Unit Service Factor	<u>100.0%</u>	<u>100.0%</u>	<u>70.8%</u>
20. Unit Availability Factor	<u>100.0%</u>	<u>100.0%</u>	<u>70.8%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>74.0%</u>	<u>74.0%</u>	<u>53.2%</u>
22. Unit Capacity Factor (Using DER Net)	<u>70.9%</u>	<u>70.9%</u>	<u>50.9%</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>17.6%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Refueling, February 11, 1983 8 weeks</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_

\*Turbine Rating: 565.7 Mwe  
Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331

UNIT Duane Arnold Energy Center

DATE 2-14-83

COMPLETED BY Mark Watson

TELEPHONE 319-851-7611

MONTH January, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	468
2	478
3	478
4	475
5	474
6	470
7	455
8	273
9	333
10	372
11	435
12	444
13	439
14	436
15	301
16	300

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	415
18	432
19	433
20	431
21	422
22	296
23	302
24	414
25	428
26	411
27	378
28	426
29	420
30	422
31	420

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

Docket No. 050-0331  
 Unit Name Duane Arnold Energy Center  
 Date 2-14-83  
 Completed by Mark Watson  
 Telephone 319-851-7611

REPORT MONTH January, 1983

No.	Date	Type <sup>1</sup>	Duration (hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	1-7-83	F	N/A	B	N/A	N/A	N/A	N/A	Power reduction to perform STP 43C001 and STP 43A002 "Scram insertion time test"
2	1-14-83	F	N/A	H	N/A	N/A	N/A	N/A	Load reduction by load dispatcher
3	1-21-83	F	N/A	H	N/A	N/A	N/A	N/A	Load reduction by load dispatcher

1  
 F: Forced  
 S: Scheduled

2  
 Reason:  
 A-Equipment Failure(Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error(Explain)  
 H-Other(Explain)

3  
 Method:  
 1-Manual  
 2-Manual Scram,  
 3-Automatic Scram,  
 4-Other(Explain)

4  
 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5  
 Exhibit 1-Same Source

REFUELING INFORMATION

1. Name of facility.
  - A. Duane Arnold Energy Center
2. Scheduled date for next refueling shutdown.
  - A. February 11, 1983
3. Scheduled date for restart following refueling.
  - A. April 9, 1983 (Preliminary)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes, the following changes will have to be included in the Technical Specifications:

A. MAPLHGR Tables	C. Torus Drained Refueling Operations
B. MCPR Table	D. Common Reference Level
5. Scheduled date(s) for submitting proposed licensing action and supporting information.

A. January 14, 1983	C. December 6, 1982 (RTS-146)
B. January 14, 1983	D. December 13, 1982 (RTS-141)
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
  - A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
  - A. a) 368 b) 448
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
  - A. 2050
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
  - A. 1998

Docket No. 050-0331  
Unit Duane Arnold Energy Center  
Date 2-14-83  
Completed by Mark Watson  
Telephone 319-851-7611

MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
1-4-83	Reactor Core Isolation Cooling (RCIC)	Governor Valve Flanges	Replaced old Flexatallic, torqued unit flange per spec.
1-25-83	Primary Containment H and V	1K-18A Drywell P compressor controller	Recalibrated per spec.
1-25-83	Reactor Protection System (RPS)	RPS low water level indicating switch LIS-4592A	Recalibrated per STP.

Docket No. 055-0331  
Unit Duane Arnold Energy Ctr  
Date 2-14-83  
Completed by Mark Watson  
Telephone 319-851-7611

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

1-1-83 Normal operation at 483 MWe.

1-4-83 During normal operation surveillance testing, steam was discovered escaping from HV-2406, the RCIC turbine hydraulic goverfor valve. The RCIC steam isolation valves MO-2400 and MO-2401 were closed to commence repairs, rendering the RCIC system inoperable.

RO Reprt 83-01

1-7-83 Power reduction to perform STP 43C001 and STP 43A002 "Scram Insertion Time Test".

1-14-83 Load reduction per load dispatcher.

1-21-83 Load reduction per load dispatcher.

1-24-83 During normal operation, the "drywell to torus low differential pressure" alarm was received.

RO Report 83-02

1-25-83 During normal operation surveillance testing, LIS-4592A (one of four Reactor Protection System Low Reactor Water Level Indicating Switches) tripped out of specification:

RO Report 83-03