

# Critical Experiments with 4.31 Wt% $^{235}\text{U}$ Enriched $\text{UO}_2$ Rods in Highly Borated Water Lattices

---

---

Prepared by B. M. Durst, S. R. Bierman, E. D. Clayton

Pacific Northwest Laboratory  
Operated by  
Battelle Memorial Institute

Prepared for  
U.S. Nuclear Regulatory  
Commission

## NOTICE

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, or any of their employees, makes any warranty, expressed or implied, or assumes any legal liability of responsibility for any third party's use, or the results of such use, of any information, apparatus, product or process disclosed in this report, or represents that its use by such third party would not infringe privately owned rights.

### Availability of Reference Materials Cited in NRC Publications

Most documents cited in NRC publications will be available from one of the following sources:

1. The NRC Public Document Room, 1717 H Street, N.W.  
Washington, DC 20555
2. The NRC/GPO Sales Program, U.S. Nuclear Regulatory Commission,  
Washington, DC 20555
3. The National Technical Information Service, Springfield, VA 22161

Although the listing that follows represents the majority of documents cited in NRC publications, it is not intended to be exhaustive.

Referenced documents available for inspection and copying for a fee from the NRC Public Document Room include NRC correspondence and internal NRC memoranda; NRC Office of Inspection and Enforcement bulletins, circulars, information notices, inspection and investigation notices; Licensee Event Reports; vendor reports and correspondence; Commission papers; and applicant and licensee documents and correspondence.

The following documents in the NUREG series are available for purchase from the NRC/GPO Sales Program: formal NRC staff and contractor reports, NRC-sponsored conference proceedings, and NRC booklets and brochures. Also available are Regulatory Guides, NRC regulations in the *Code of Federal Regulations*, and *Nuclear Regulatory Commission Issuances*.

Documents available from the National Technical Information Service include NUREG series reports and technical reports prepared by other federal agencies and reports prepared by the Atomic Energy Commission, forerunner agency to the Nuclear Regulatory Commission.

Documents available from public and special technical libraries include all open literature items, such as books, journal and periodical articles, and transactions. *Federal Register* notices, federal and state legislation, and congressional reports can usually be obtained from these libraries.

Documents such as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings are available for purchase from the organization sponsoring the publication cited.

Single copies of NRC draft reports are available free upon written request to the Division of Technical Information and Document Control, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at the NRC Library, 7920 Norfolk Avenue, Bethesda, Maryland, and are available there for reference use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from the American National Standards Institute, 1430 Broadway, New York, NY 10018.

---

---

# Critical Experiments with 4.31 Wt% $^{235}\text{U}$ Enriched $\text{UO}_2$ Rods in Highly Borated Water Lattices

---

---

Manuscript Completed: May 1982  
Date Published: August 1982

Prepared by  
B. M. Durst, S. R. Bierman, E. D. Clayton

Pacific Northwest Laboratory  
Richland, WA 99352

**Prepared for**  
**Division of Engineering Technology**  
**Office of Nuclear Regulatory Research**  
**U.S. Nuclear Regulatory Commission**  
**Washington, D.C. 20555**  
**NRC FIN B2094**

## ABSTRACT

A series of critical experiments were performed with 4.31 wt% <sup>235</sup>U enriched UO<sub>2</sub> fuel rods immersed in water containing various concentrations of boron ranging up to 2.55 g/l. The boron was added in the form of boric acid (H<sub>3</sub>BO<sub>3</sub>). Critical experiment data were obtained for two different lattice pitches wherein the water-to-uranium oxide volume ratios were 1.59 and 1.09. The experiments provide benchmarks on heavily borated systems for use in validating calculational techniques employed in analyzing fuel shipping casks and spent fuel storage systems that may utilize boron for criticality control.

#### ACKNOWLEDGEMENTS

The authors wish to acknowledge the contributions of Mr. J. H. Smith and Mr. J. H. Lauby, Senior Reactor Operators at the Critical Mass Laboratory, to the successful completion of these criticality experiments and of Lucille N. Terry in preparing this manuscript for publication.

## SUMMARY

The results from the seventh in a series of criticality experiments funded by the United States Nuclear Regulatory Commission are presented in this paper. The purpose of these experiments is to provide a data base for benchmarking computational methods used in the criticality analysis of fuel transportation and storage systems.

This experiment program explored the effect on criticality of adding large amounts of boric acid to undermoderated arrays of 4.31 wt%  $^{235}\text{U}$  enriched  $\text{UO}_2$  fuel rods in water. Boron concentrations ranging up to 2.55 g/l were achieved in two lattices, one having a square pitch of 1.89cm and the other 1.715cm (water-to-fuel volume ratios of 1.59 and 1.09 respectively).

It is apparent that if the concentration of boron in the water moderator of two different lattices be the same, the lattice with the larger spacing (and water-to-uranium ratio) will have a higher ratio of boron to uranium. In addition, as the spectrum is harder in the lattice of least water, the boron also can be expected to have a smaller effect on the criticality of that lattice.

It also is apparent that compacting a lattice of fuel rods from optimum spacing in water (reducing the separation between fuel rods in the assembly) can result in a larger critical size and number of fuel rods for criticality, and lead to a safer condition. The data reported here on heavily borated lattices show that it is possible for the reverse to occur, i.e., with the absorber present, compacting the lattice spacing can result in a smaller critical size or volume and number of fuel rods for criticality.

Adding a given amount of boron to the water moderator of either of the above two lattices results in a larger critical size and mass (number of rods for criticality). The boron-to-uranium atom ratio is smaller, however, in the smaller lattice for a given concentration of boron in the water. This implies (and the data on these two lattices show) that as the concentration of boron in the water of the two lattices is equally increased there will be a boron concentration point at which the smaller lattice has the smaller critical size, though in the absence of the boron this was not the case. If the concentration of boron in g/l is increased in the smaller lattice so as to maintain the same boron-to-uranium atom ratio in the two lattices this does not appear to be the case (see Figure 5). This point should be considered in the use of soluble absorbers for criticality prevention and control.

CONTENTS

ABSTRACT . . . . .	iii
ACKNOWLEDGEMENTS . . . . .	v
SUMMARY . . . . .	vii
LIST OF FIGURES . . . . .	xi
LIST OF TABLES . . . . .	xi
INTRODUCTION . . . . .	1
EXPERIMENTAL PROCEDURE . . . . .	2
RESULTS . . . . .	7
CONCLUSIONS . . . . .	11
REFERENCES . . . . .	12

FIGURES

1	Description of Fuel Rods . . . . .	3
2	Experimental Setup . . . . .	4
3	Photograph of Lattice Assembly . . . . .	5
4	Critical Rod Number as a Function of Boron Concentration . . . . .	9
5	Estimated Critical Number of Rods vs. B/UO <sub>2</sub> Wt. Ratio . . . . .	10

TABLES

I.	Water Impurities . . . . .	6
II.	Results of Experiments . . . . .	8



CRITICAL EXPERIMENTS WITH 4.31 WT%  $^{235}\text{U}$  ENRICHED  $\text{UO}_2$   
RODS IN HIGHLY BORATED WATER LATTICES

INTRODUCTION

A research program was begun in 1976 at the Pacific Northwest Laboratory Critical Mass Laboratory (CML) to provide experimental criticality data on conditions simulating light water reactor fuel shipping and storage configurations. The program, which is funded by the Nuclear Regulatory Commission, has provided data to date which supports criticality analyses of both fuel element handling and storage facilities.

Six previous experiment programs under this program have explored a wide variety of phenomena affecting the criticality safety of fuel rod storage, such as the effect of neutron absorber plates between interacting clusters of rods in water and the effect of heavy metal reflector walls on critical separation distances between fuel rod clusters.(1-7) This report details the results of the experiments which evaluate the effect of a soluble neutron absorber, boric acid, on the criticality of water moderated fuel rod arrays.

Boric acid is a widely used neutron poison in both shipping casks and storage pools. The poison is added as an additional precaution against accidental criticality. Although many experiments have been performed on water flooded arrays containing less than 1 g/l boron, no data exist for the high concentrations of boron which might be used for criticality control ( $\sim 2 - 5$  g/l B in solution). Large amounts of boric acid can significantly alter the neutronics of a system. For example, Marotta showed in calculations relating to the recriticality of the TMI-2 core, that the infinite multiplication factor,  $k_\infty$ , is higher for a tighter pitched system at the same boron concentrations.(8)

This should not be confused with the fact that adding a "neutron absorber" to any given lattice (providing this absorber does not of itself substantially moderate neutrons or displace the moderator that does) will always render that lattice assembly further subcritical.

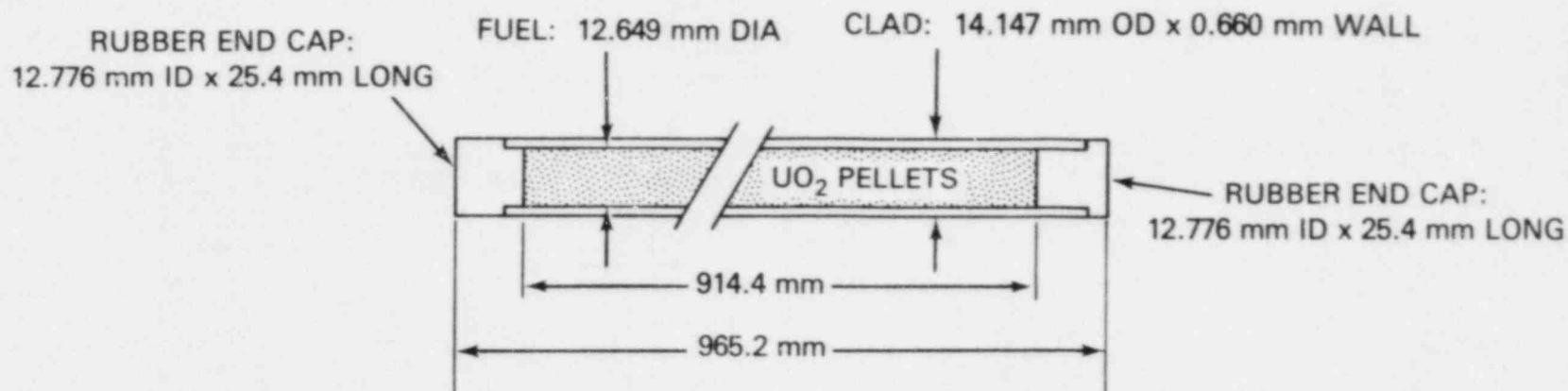
## EXPERIMENTAL PROCEDURE

The experiments utilized 4.31 wt%  $^{235}\text{U}$  enriched  $\text{UO}_2$  rods available at the Critical Mass Laboratory in square pitched, water moderated lattices, poisoned with varying amounts of boric acid ( $\text{H}_3\text{BO}_3$ ). Data were obtained for two different lattice pitches,  $1.890\text{cm} \pm .002$  and  $1.715\text{cm} \pm .005$ . This corresponds to water-to-fuel volume ratios of 1.59 and 1.09 respectively.

The fuel rods are described in Figure 1. The experimental setup is shown in Figures 2 and 3. Fuel rods were loaded into polypropylene lattice templates [density of  $0.904\text{ g/cm}^3$ ], securely fastened inside a plexiglas container. The container was surrounded on all four sides by a full water reflector. Beneath the container, a 15.2cm thick plexiglas slab elevated the fuel off the tank bottom. The soluble absorber was restricted to the water within the container, while the outside reflector region was left unpoisoned. An analysis of the water prior to boron additions is given in Table I.

Critical arrays were constructed using the critical approach method to obtain criticality. That is, arrays were built up in a rectangular fashion. With the array width remaining fixed at 40 or 44 rods, depending on pitch, the array length was gradually increased symmetrically. An experiment was performed at one boric acid concentration, poisoned water samples were taken and then additional boric acid was added prior to the next fuel loading. Water was progressively removed from inside the container as more fuel was added to keep the water level inside constant as shown in Figure 2.

## 4.31 WT% $^{235}\text{U}$ ENRICHED $\text{UO}_2$ RODS



CLADDING: 6061 ALUMINUM TUBING

LOADING:

ENRICHMENT -  $4.31 \pm 0.01$  WT%  $^{235}\text{U}$

FUEL DENSITY -  $94.9 \pm 0.55$ % OF THEORETICAL DENSITY

URANIUM ASSAY -  $88.055 \pm 0.261$  WT% OF TOTAL FUEL COMPOSITION

$\text{UO}_2$  -  $1203.38 \pm 4.12$  g/ROD

END CAP:

DENSITY -  $1.321$  g/cm<sup>3</sup>

COMPOSITION - C- $58 \pm 1$  WT%

H- $6.5 \pm 0.3$  WT%

Ca- $11.4 \pm 1.8$  WT%

S- $1.7 \pm 0.2$  WT%

O- $22.1$  WT% (BALANCE)

Si- $0.3 \pm 0.1$  WT%

Figure 1. Description of Fuel Rods

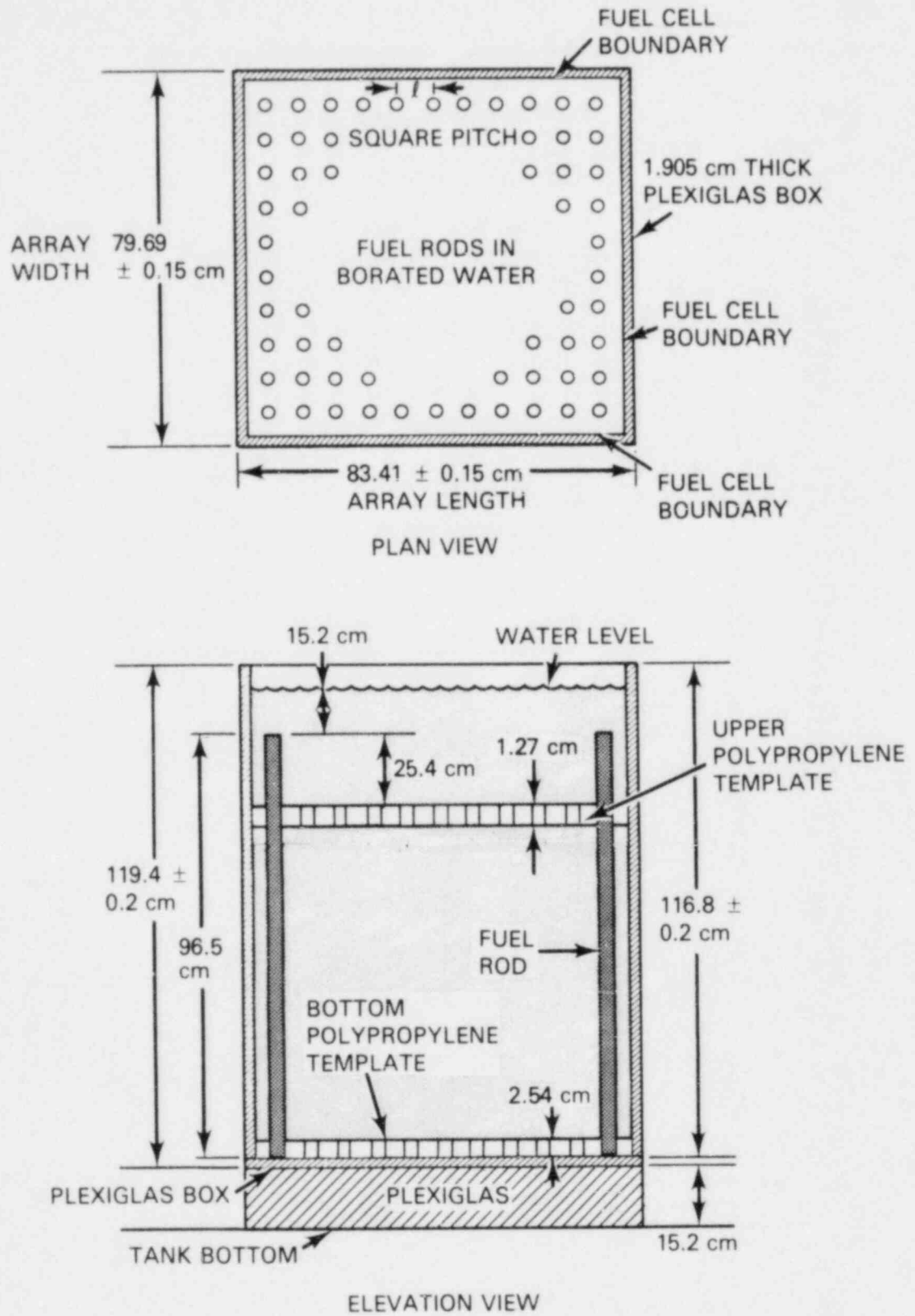


Figure 2. Experimental Setup

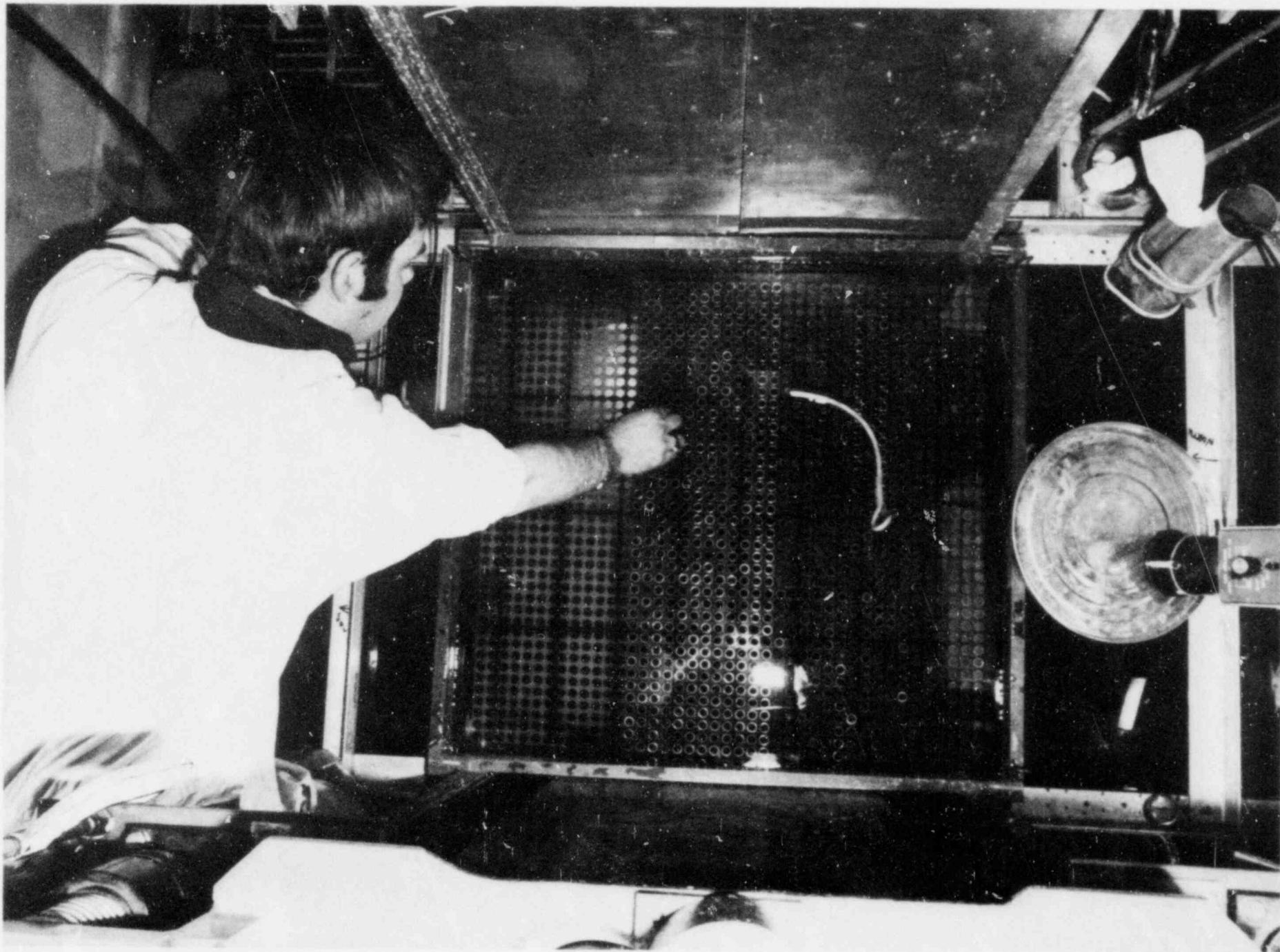


Figure 3. Photograph of Lattice Assembly

TABLE I

WATER IMPURITIES

CONCENTRATION  
g/m<sup>3</sup> (ppm)

COMPONENT

Cl	≤ 5
NO <sub>3</sub>	0.02
Cr <sup>+6</sup>	<0.01
Zn	16
Mn	<0.01
Pb	<0.005
F	0.18
Fe	24
Cu	<0.01
Cd	0.001
SO <sub>3</sub>	14.5
DISSOLVED SOLIDS	61 ± 3

## RESULTS

The results of the experiments are summarized in Table II and Figure 4. The largest critical array constructed for the 1.89cm - pitched lattice was a 40 x 30.92 array (~1237 rods), having a boron concentration of 2.55 g/l. For the 1.715cm lattice, the largest array constructed was 44 x 27.09 (~1192 fuel rods) also at a concentration of 2.55 g/l B. This amount of boron is substantial in terms of hold down as in both lattices the assemblies contained more than twice the number of rods required for criticality in the absence of the boron.

The lattice with the 1.715cm pitch had a water-to-fuel volume ratio of 1.09 and that of the 1.89cm lattice is 1.59. The smaller lattice was less well moderated and in the case of no boron required a larger number of rods for criticality (509 vs. 357). As noted above, however, for the case of 2.55 g/l B in the water, the critical rod number was 1192 for the 1.715cm lattice compared with 1237 for the 1.89cm lattice, i.e., the smaller lattice with the absorber had the smaller critical number, not larger. This may be misleading, but consideration must be given to the fact that if the boron concentration in the water moderator is the same, the lattice with the smaller water-to-fuel volume ratio also will have a smaller boron-to-uranium ratio, or less boron absorber per atom of uranium.

The experimental arrays were buckling corrected to square arrays to account for geometric differences and Figure 5 presents rod numbers plotted vs. the boron-to-uranium oxide weight ratios in the two lattices. With comparable amounts of absorber present, the smaller lattice does indeed have the larger critical number of rods.

The implication is important. It means that the boron concentration in terms of g/l B may have to be substantially increased in the less well moderated lattice to ensure the same degree of subcriticality for a given assembly size.

TABLE II

Experiment Number	Square Pitch (cm)	RESULTS OF EXPERIMENTS			
		Boron <sup>(a)</sup> Concentration g/l	Critical Array Width No. of Rods	Critical <sup>(b)(c)</sup> Array Length No. of Rods	Total Rods <sup>(d)</sup> for Criticality
SSC-4.3-000-173	1.890	0.0	40	8.92	357
174	1.890	0.49 ± 0.06	40	10.72	429
175	1.890	1.25 ± 0.21	40	14.05	562
176	1.890	2.15 ± 0.35	40	23.07	923
177	1.890	2.55 ± 0.07	40	30.92	1237
∞ 178	1.715	0.0	44	11.57	509
179	1.715	1.03 ± 0.05	44	16.75	737
180	1.715	1.82 ± 0.30	44	20.84	917
181	1.715	2.55 ± 0.21	44	27.09	1192

(a) Boron added in the form of  $H_3BO_3$ .

(b) This has the same interpretation as all experiments performed before in this program. A fraction of a row is a row which extends full width but is thinner in length. This is the best interpretation for computer modeling.

(c) The cell boundary of the first fuel row is next to the plexiglas wall of the box.

(d) The error in critical rod number is <0.5%.



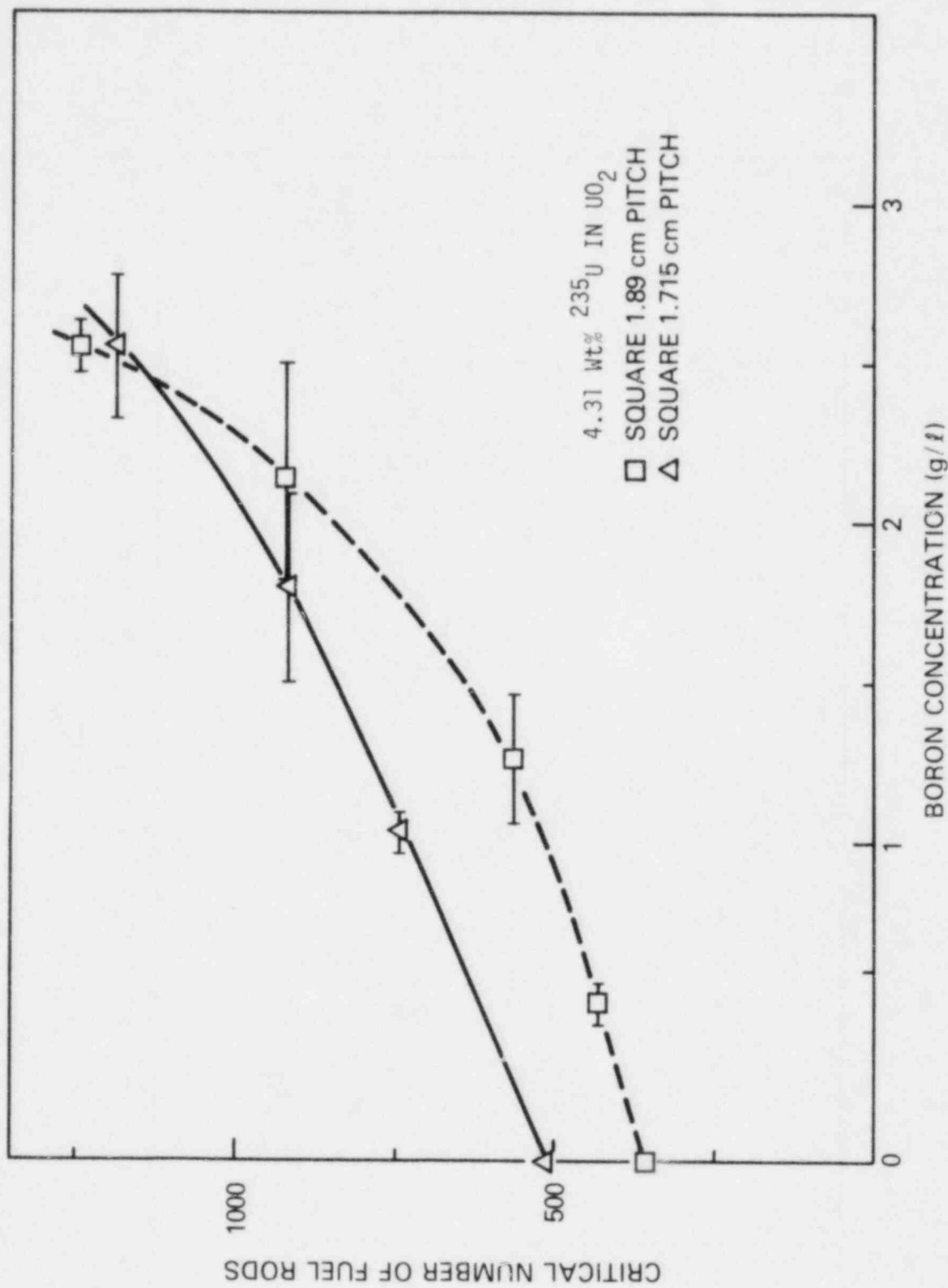


Figure 4. Critical Rod Number as a Function of Boron Concentration

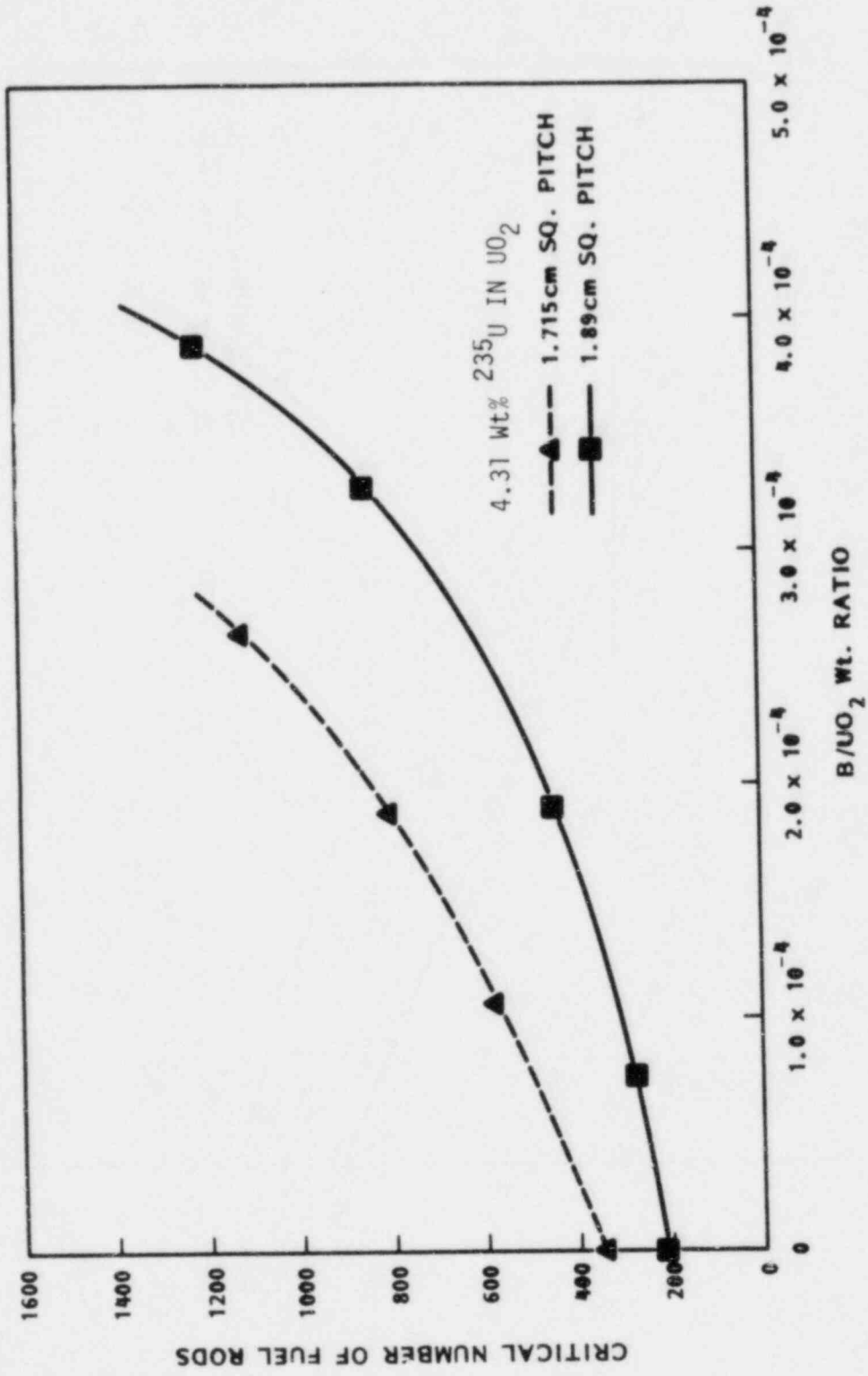


FIGURE 5. ESTIMATED CRITICAL NUMBER OF RODS vs B/UO<sub>2</sub> Wt. RATIO

## CONCLUSIONS

Experiments have been performed with LWR-type fuel rods for evaluating criticality in highly borated lattices under high reactivity hold down conditions with boron concentrations in the water ranging up to 2.55 g/l.

It is well known that compacting a lattice of fuel rods from optimum spacing in water (reducing the separation between fuel rods in the assembly) can result in a larger critical size and number of fuel rods, and lead to a safer condition. The data reported here on highly borated lattices show that it is possible for the reverse to occur, i.e., after adding boron to the lattice, compacting the lattice spacing might then result in a smaller critical volume and number of fuel rods for criticality.

The implication is important. It means that the boron concentration in terms of g/l B (but not necessarily the B/U atom ratio) may have to be substantially increased in the less well moderated lattice to ensure the same degree of subcriticality for a given assembly size.

## REFERENCES

1. S. R. Bierman, B. M. Durst and E. D. Clayton, Critical Separation Between Subcritical Clusters of 2.35 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water with Fixed Neutron Poisons, PNL-2438, Pacific Northwest Laboratory, 1977. Available from Technical Information Files, Pacific Northwest Laboratory, PO Box 999, Richland, WA 99352.
  2. S. R. Bierman, B. M. Durst and E. D. Clayton, Critical Separation Between Subcritical Clusters of 4.31 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water with Fixed Neutron Poisons, USNRC Report, NUREG/CR-0073, 1978. Available from National Technical Information Service, U. S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.\*
  3. S. R. Bierman, B. M. Durst and E. D. Clayton, Critical Experiments with Subcritical Clusters of 2.35 wt% and 4.31 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water with Uranium or Lead Reflecting Walls, USNRC Report, NUREG/CR-0796, Vol. 1, 1979. Available from National Technical Information Service, U. S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.\*
  4. S. R. Bierman and E. D. Clayton, Criticality Experiments with Subcritical Clusters of 2.35 wt% and 4.31 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water at a Water-to-Fuel Volume Ratio of 1.6, USNRC Report, NUREG/CR-1547, 1980. Available from National Technical Information Service, U. S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.\*
  5. S. R. Bierman and E. D. Clayton, Critical Experiments with Subcritical Clusters of 2.35 wt% and 4.31 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water with Steel Reflecting Walls, USNRC Report, NUREG/CR-1784, 1981. Available from National Technical Information Service, U. S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.\*
  6. B. M. Durst, "Effect of Heavy Metal Reflectors on Criticality of Water Flooded Fuel Rod Arrays," ANS Transactions, 35: 283, 1980. Available from American Nuclear Society, LaGrange Park, IL 60525.
  7. S. R. Bierman, B. M. Durst and E. D. Clayton, Critical Experiments with Subcritical Clusters of 2.35 wt% and 4.31 wt%  $^{235}\text{U}$  Enriched  $\text{UO}_2$  Rods in Water with Uranium or Lead Reflecting Walls, USNRC Report, NUREG/CR-0796, Vol. 2, 1981. Available from National Technical Information Service, U. S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.\*
  8. C. R. Marotta, Recriticality Potential of TMI-2 Core, ANS Transactions, 35: 272, 1980. Available from American Nuclear Society, LaGrange Park, IL 60525.
- \* Also available for purchase from the NRC/GPO Sales Program, U. S. Nuclear Regulatory Commission, Washington, DC 20555.

DISTRIBUTION

No. of  
Copies

OFFSITE

240	U. S. Nuclear Regulatory Commission Division of Technical Information and Document Control 7920 Norfolk Avenue Bethesda, MD 29014
2	DOE Technical Information Center
20	M. S. Weinstein Office of Nuclear Regulatory Research U. S. Nuclear Regulatory Commission MS 1130 SS Washington, DC 20555

ONSITE

50	<u>Pacific Northwest Laboratory</u>
	S. R. Bierman (43)
	Technical Information (5)
	Publishing Coordination (2)

<b>NRC FORM 335</b> (7-77)		<b>U.S. NUCLEAR REGULATORY COMMISSION</b> <b>BIBLIOGRAPHIC DATA SHEET</b>		<b>1. REPORT NUMBER (Assigned by DDC)</b> NUREG/CR-2709 PNL-4267	
<b>4. TITLE AND SUBTITLE (Add Volume No., if appropriate)</b> Critical Experiments with 4.31 Wt% <sup>235</sup> U Enriched UO <sub>2</sub> Rods in Highly Borated Water Lattices				<b>2. (Leave blank)</b>	
<b>7. AUTHOR(S)</b> B. M. Durst, S. R. Bierman and E. D. Clayton				<b>3. RECIPIENT'S ACCESSION NO.</b>	
<b>9. PERFORMING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code)</b> Pacific Northwest Laboratory Richland, WA 99352				<b>5. DATE REPORT COMPLETED</b> MONTH: May   YEAR: 1982	
<b>12. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code)</b> Division of Engineering Technology Office of Nuclear Regulatory Research U. S. Nuclear Regulatory Commission Washington, DC 20555				<b>6. (Leave blank)</b>	
<b>13. TYPE OF REPORT</b> Technical				<b>7. (Leave blank)</b>	
<b>15. SUPPLEMENTARY NOTES</b>				<b>8. (Leave blank)</b>	
<b>16. ABSTRACT (200 words or less)</b>  A series of critical experiments were performed with 4.31 wt% <sup>235</sup> U enriched UO <sub>2</sub> fuel rods immersed in water containing various concentrations of boron ranging up to 2.55 g/l. The boron was added in the form of boric acid (H <sub>3</sub> BO <sub>3</sub> ). Critical experiment data were obtained for two different lattice pitches wherein the water-to-uranium oxide volume ratios were 1.59 and 1.09. The experiments provide benchmarks on heavily borated systems for use in validating calculational techniques employed in analyzing fuel shipping casks and spent fuel storage systems that may utilize boron for criticality control.				<b>9. (Leave blank)</b>	
<b>17. KEY WORDS AND DOCUMENT ANALYSIS</b>		<b>17a. DESCRIPTORS</b>			
<b>17b. IDENTIFIERS/OPEN-ENDED TERMS</b>					
<b>18. AVAILABILITY STATEMENT</b>  Unlimited		<b>19. SECURITY CLASS (This report)</b> Unclassified		<b>21. NO. OF PAGES</b>	
		<b>20. SECURITY CLASS (This page)</b> Unclassified		<b>22. PRICE</b> \$	

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

OFFICIAL BUSINESS  
PENALTY FOR PRIVATE USE, \$300

FOURTH CLASS MAIL  
POSTAGE & FEES PAID  
USNRC  
WASH D C  
PERMIT No. 552

120555078877 1 ANRC  
US NRC  
ADM DIV OF TIDC  
POLICY & PUBLICATIONS MGT BR  
PDR NUREG COPY  
LA 212  
WASHINGTON DC 20555

NUREG/CR-2709  
CRITICAL EXPERIMENTS WITH 4.31 Wt% <sup>235</sup>U ENRICHED UO<sub>2</sub> RODS  
IN HIGHLY BORATED WATER LATTICES

AUGUST 1982