## BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475
BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT CALVERT CLIFFS NUCLEAR POWER PLANT LUSBY, MARYLAND 20657

September 1, 1982

Mr. R. C. Haynes, Director U.S. Nuclear Regulatory Commission Region 1 631 Park Ave. King of Prussia, PA 19406

Dear Mr. Haynes,

In our letters to you dated August 26, 1982, on the thirty day report for LER 82-46/3L and LER 82-37/3L, the cover letters were inadvertently transposed. Attached you will find the corrected letters.

We regret any inconvenience this may have caused.

Very truly yours,

L. B. Russell

Plant Superintendent

LBR/mlk

cc: Director, Office of Management Information and Program Control

A. E. Lundvall, Jr.

J. A. Tiernan

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BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT CALVERT CLIFFS NUCLEAR POWER PLANT LUSBY, MARYLAND 20657

August 26, 1982

Mr. Ronald C. Haynes, Director U. S. Nuclear Regulatory Commission Region 1 631 Park Avenue King of Prussia, PA 19406 Docket No. 50-318 License No. DPR 69

Dear Mr. Haynes:

In accordance with Technical Specification 6.9.1.9.b please find the attached thirty day report for LER 82-37/3L.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. B. Russell

Plant Superintendent

LBR: LFB: bsg

cc: Director, Office of Management Information

and Program Control

Messrs: A. E. Lundvall, Jr.

J. A. Tiernan

DUPE 8209080294

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7-771 LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 0 0 0 0 CON'T (8)0 18 (7) 0 17 13 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During Surveillance Testing at 1415, the outer personnel air lock door 0 2 opening mechanism failed preventing the door from being closed 0 3 (T.S. 3.6.1.3). The inner air lock door was maintained shut during the 0 4 event, therefore, the safety of the public was not affected. The outer 0 5 airlock door was repaired and returned to service at 1800. 0 6 Similar events: 50-317/81-25 and 50-318/81-12. 0 7 0 8 COMP CAUSE SYSTEM CAUSE SUBCODE CODE CODE A (15 B (13) 0 9 REVISION OCCURRENCE CODE NO. REPORT NO. LER/RO 0 | 3 | 0 0 | 3 | 7 REPORT NUMBER PRIME COMP. COMPONENT NPRD-4 HOURS (22) MANUFACTURER FORM SUB SUBMITTED (18) F CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Airlock mechanism failed due to broken Cam Follower. Wear in the Geneva 1 0 Mechanism Plate Grooves is causing excessive play and bending forces on the Cam Followers. An overhaul of the Airlock Mechanism, replacing parts with modified ones made of stronger materials, will be performed during the fall 1982 refueling outage. 1 4 METHOD OF OTHER STATUS (30) B (31) 0 3 5 Surveillance Test CONTENT ACTIVITY LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35 N/A PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER 0 0 0 (37) PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 0 (40) LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION Z (42) NAC USE ONLY PUBLICITY DESCRIPTION (45) ISSUED N/A

S. Pavis/P. J.

301-269-4742/4871

PHONE -

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BALTIMORE. MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

August 26, 1982

Mr. Ronald C. Haynes, Director
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-317 License No. DPR 53

Dear Mr. Haynes:

In accordance with Technical Specification 6.9.1.9.b please find the attached thirty day report for LER 82-46/3L.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. B. Russell

Plant Superintendent

LBR: PJW: bsg

cc: Director, Office of Management Information and Program Control

A. E. Lundvall, Jr.

J. A. Tiernan

JUNE 82090802.09

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LER NO.\* 82-46/3L
DOCKET NO. 50-317
LICENSE NO. DPR 53
EVENT DATE 07-29-82
REPORT DATE 08-26-82
ATTACHMENT

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (CONT'D)

An improper Axial Power Distribution Calculator "Coefficient Setting" description and value in the setpoint file caused the ASI indication to be our of specification following maintenance on the RPS cabinet.

After the proper description was determined from system schematics, an investigation revealed the probable cause to be a typographical error during an earlier revision of the setpoint file. The "Coefficient Setting" description and value have been corrected in the setpoint file and all channels have been verified to be in specification.

The Unit 2 setpoint file also contained the improper "Coefficient Setting" description though the value was correct. The value was supplied by the NSSS vendor as part of the last core reload. Revision of the setpoint file at that time did not detect the incorrect description. The file has been revised to list the proper description.

System schematics of both units show the proper descriptions and values.

A memorandum from the Plant Superintendent to all Plant Supervisors regarding the importance of a proper technical review of all document changes has previously been promulgated to prevent recurrence of this type of review error.