



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

March 23, 1994

Mr. William Russell, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Additional Information Pertaining to
Request for EMERGENCY TECHNICAL SPECIFICATION AMENDMENT
for Byron and Braidwood Stations
Technical Specification Section 3/4.7.1
NRC Docket Nos. 50-454, 50-455 50-456 and 50-457

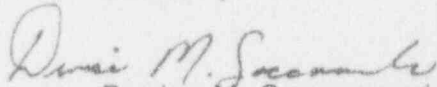
Reference: D. Saccomando Letter to W. Russell dated March 21, 1994,
transmitting request for Emergency Technical Specification Amendment

Dear Mr. Russell:

As requested by the Nuclear Regulatory Commission's (NRC), Commonwealth Edison Company (CECo) is supplying additional information pertaining to the request for Emergency Technical Specification which was transmitted with the reference letter. Specifically, the attachment addresses the analysis for Bryon/Braidwood Units 1 and 2 relaxation of main steam safety valves setpoint tolerance to +/- 3%.

If you have any additional comments or questions please contact me at
(708) 663-6484.

Sincerely,


Denise M. Saccomando
Nuclear Licensing Administrator

Attachment

cc: R. Assa, Braidwood Project Manager - NRR
G. Dick, Byron Project Manager - NRR
S. Dupont, Senior Resident Inspector - Braidwood
H. Peterson, Senior Resident Inspector - Byron
B. Clayton, Branch Chief - Region III
Office of Nuclear Facility Safety - IDNS

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Westinghouse
Electric Corporation

Energy Systems

Nuclear and Advanced
Technology Division

Box 355
Pittsburgh Pennsylvania 15230-0355

CAE-91-209
CCE-91-219
NS-OPLS-OPL-I-91-415
July 22, 1991

Mr. R. Pleniewicz, Station Manager
Byron Nuclear Station
Commonwealth Edison Company
P.O. Box 458
4450 North German Church Road
Byron, IL 61010

Commonwealth Edison Company
Byron/Braidwood Units 1 and 2
Relaxation of MSSV Setpoint Tolerance to +/-3% - Revised SECL

Reference: (1) Westinghouse letter CAE-90-315/CCE-90-310, dated 10-31-90.

Dear Mr. Pleniewicz:

Enclosed please find Revision 1 to safety evaluation SECL 90-469, previously transmitted via Reference 1. The revised safety evaluation clarifies the section of the ASME code associated with the main steam safety valves.

If you have any questions, do not hesitate to call.

Very truly yours,

Thomas F. Woodhouse for
B. S. Humphries, Manager
Commonwealth Edison Projects
Customer Projects Department

B. E. Rarig
Enclosure

cc: F. Lentine
D. Swartz
P. McHale
J. Feimster
K. Kofron
R. Waninski