VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

R. H. LEASBURG
VICE PRESIDENT
NUCLEAR OPERATIONS

September 1, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 506
NO/DJF:ms
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

NUREG 0612 - CONTROL OF HEAVY LOADS
SURRY POWER STATION
UNIT NOS. 1 AND 2

On August 12, 1982 a conference call was held between Vepco, the NRC and Franklin Research Center personnel. The following information is provided concerning areas requiring further action as a result of the conclusion and recommendations shown in the draft Technical Evaluation Report for Control of Heavy Loads at Surry Power Station and the August 12, 1982 conference call.

TER 2.1.2 SAFE LOAD PATHS {GUIDELINE, NUREG-0612, SEC. 5.1.1(1)}

Comment:

It was stated by the NRC that in order for Surry to fully comply with the guidelines, the licensee should clarify the procedure for handling deviations from safe load paths.

Response:

Vepco stated that this information is presently available to the NRC and that it could be found in Section 5 of Vepco's Nuclear Power Station Quality Assurance Manual and in Section 6 of the Surry Power Station Technical Specifications. The procedure for deviations to procedures requires review by station supervisory personnel with a followup review by the Station Nuclear Safety and Operating Committee.

TER 2.1.4 CRANE OPERATOR TRAINING {GUIDELINE 1, NUREG-0612, SEC. 5.1.1(3)}

Comment:

Under Section b. Evaluation of 2.1.4 it was stated that the licensee should ensure that a mechanism is in place to provide future crane operator training that meets the intent of ANSI B30.2-1976, Chapter 2-3.

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Response:

Surry Power Station presently has procedures for the initial training of crane operators. There are presently no provisions for retraining or future operator training beyond the initial training of the personnel.

By the next refueling outage at Surry Power Station procedures will be developed to provide future crane operator training that meets the intent of ANSI B-30.2-1976, Chapter 2-3.

TER 2.1.5 SPECIAL LIFTING DEVICES {GUIDELINE 4, NUREG-0612, SEC. 5.1.1(4)}

Comment:

The NRC stated that they would like the schedule dates that Vepco has for the completion of the Westinghouse analyses of special lifting devices used at Surry Power Station.

Response:

The dates for completion of the analyses as provided by Westinghouse and submittal to Vepco are as follows:

- January 1, 1983 - Preliminary Stress Report

- February 1, 1983 - Preliminary Evaluation

- March 1, 1983 - Final Report

After internal review Vepco would submit the results of this review by no later than March 31, 1983.

TER 2.1.6 LIFTING DEVICES (NOT SPECIALLY DESIGNED) {GUIDELINE 5, NUREG-0612 SEC. 5.1.1(5)}

Comment:

The NRC stated that an evaluation must be performed to determine the significance of dynamic loads on the slings used for lifting heavy loads. If there are significant dynamic loads slings shall be marked to indicate which slings are restricted in use to only certain cranes.

Response:

Vepco will evaluate the significance of dynamic loads on the slings used and provide the NRC with the information by October 15, 1982. If the results of the evaluation indicate the necessity to mark the slings, they will be marked prior to their next scheduled use.

TER 2.2.3 SPECIAL REVIEW FOR HEAVY LOADS HANDLED OVER THE CORE {INTERIM PROTECTION MEASURE 6, NUREG-0612, SEC. 5.3(6)}

Comment:

The NRC stated that the licensee should verify that the one-time visual inspections of load bearing components of cranes, slings, and special lifting devices to identify flaws or deficiencies that could lead to a component failure have been performed and the appropriate repairs/replacments have been completed.

Response:

Surry Power Station Preventative Maintenance Procedure MMP-P-CR-015 for periodic inspections of containment cranes and lifting equipment satisfies this requirement.

Per this procedure cranes, reactor head and internals lifting rigs are inspected prior to each refueling and at each containment maintenance period if they are to be used and have been idle for a period of more than six (6) months or the last inspection has been over one (1) year.

The reactor coolant pump motor lifting rig and wire rope slings are inspected prior to each refueling and at each containment maintenance period if they are to be used and the last inspection has been over one (1) month.

The one time inspection required will be performed prior to the next refueling in accordance with the above procedure.

If you have any questions or require further clarification concerning the above subjects, please advise.

Very truly yours,

R. H. Leasburg

cc: Mr. James P. O'Reilly, Regional Administrator
Region II
Atlanta, Georgia 30303