



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated August 30, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 66, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 day days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages provided to maintain document completeness.*

Remove

3/4 8-29

3/4 11-17

Insert

3/4 8-29

3/4 8-30*

3/4 11-17

3/4 11-18*

TABLE 3.8.4.1-1 (Continued)

PRIMARY CONTAINMENT PENE ION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

2. 480-VOLT MOLDED CASE CIRCUIT BREAKERS (Continued)

| CIRCUIT BREAKER NO. | LOCATION | TYPES | SYSTEMS OR EQUIPMENT POWERED |
|------------------------|----------|------------------------|--|
| 52-253064 | 10B253 | IM HFB150 TM HFB150 | Reactor Vessel Head Vent to Steam Line 1BB-HV-F005 |
| 52-263011 | 10B263 | IM HFB150 TM HFB150 | Reactor Vessel Head Vent Outboard Isolation 1BB-HV-F002 |
| 52-263012* | 10B263 | IM HFB150 TM HFB150 | Recirc Pump Motor Hoist 1BH201 Disconnect Switch 1BS204 |
| 52-263042* | 10B263 | IM HFB150 TM HFB150 | Main Steam Relief Valve Hoist 10H;02 Disconnect Switch 10S207 |
| 52-263054 | 10B263 | IM HFB150 TM HFB150 | RWCU Suction from Recirc Loop A 1BG-HV-F100 |
| 52-263081 | 10B263 | IM HFB150 TM HFB150 | RWCU Suction from RPV Drain Valve 1BG-HV-F101 |
| 52-263082 | 10B263 | IM HFB150 TM HFB150 | RWCU Suction Valve 1BG-HV-F102 |
| 52-263083 | 10B263 | IM HFB150 TM HFB150 | RWCU Suction from Recirc Loop B Valve 1BG-HV-F106 |
| 52-264053 | 10B264 | IM HFB150 TM HFB150 | Recirc Pump A Discharge Valve 1BB-HV-F031A |
| 52-264062 | 10B264 | IM HFB150 TM HFB150 | Feedwater Inlet B Shutoff Valve 1AE-HV-F011B |
| 52-264071 | 10B264 | IM HFB150 TM HFB150 | Reactor Recirc Pump 1AP201 Space Heater 1AS220 |
| 52-264072 | 10B264 | IM HFB150 TM HFB150 | Reactor Recirc Pump 1BP201 Space Heater 1BS220 |
| 52-264083 | 10B264 | IM HFB150 TM HFB150 | Recirc Pump A Suction Valve 1BB-HV-F023A |

* These breakers shall be administratively maintained open in OPERATIONAL CONDITIONS 1, 2 and 3 and are not required to be tested.

ELECTRICAL POWER SYSTEMS

MOTOR OPERATED VALVES - THERMAL OVERLOAD PROTECTION (BYPASSED)

LIMITING CONDITION FOR OPERATION

3.8.4.2 The thermal overload protection bypass circuit of each motor operated valve (MOV) shown in Table 3.8.4.2-1 shall be OPERABLE.

APPLICABILITY: Whenever the MOV is required to be OPERABLE.

ACTION:

With the thermal overload protection bypass circuit for one or more of the above required MOVs inoperable, restore the inoperable thermal overload protection bypass circuit(s) to OPERABLE status within 8 hours or declare the affected MOV(s) inoperable and apply the appropriate ACTION statement(s) for the affected system(s).

SURVEILLANCE REQUIREMENTS

4.8.4.2.1 The thermal overload protection bypass circuit for each of the above required MOVs shall be demonstrated OPERABLE:

- a. At least once per 18 months by the performance of a CHANNEL FUNCTIONAL TEST for:
 1. Those thermal overload protection devices which are normally in force during plant operation and bypassed only under accident conditions.
 2. A representative sample of at least 25% of those thermal overload protection devices which are bypassed continuously and temporarily placed in force only when the MOVs are undergoing periodic or maintenance testing, such that the bypass circuitry for each thermal overload protection device of this type is tested at least once per 6 years.
 3. A representative sample of at least 25% of those thermal overload protection devices which are in force during normal manual (momentary push button contact) MOV operation and bypassed during remote manual (push button held depressed) MOV operation, such that the bypass circuitry for each thermal overload protection device of this type is tested at least once per 6 years.
- b. Following maintenance on the motor starter.

4.8.4.2.2 The thermal overload protection for the above required MOVs which are continuously bypassed and temporarily placed in force only when the MOV is undergoing periodic or maintenance testing shall be verified to be continuously bypassed following such testing.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

LIMITING CONDITION FOR OPERATION

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3.11.2.7 The radioactivity rate of noble gases measured at the recombiner after-condenser discharge shall be limited to less than or equal to 3.30 E+5 microcuries/sec after 30 minute decay.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2* and 3*.

ACTION:

With the radioactivity rate of noble gases at the recombiner after-condenser discharge exceeding 3.30 E+5 microcuries/sec after 30 minute decay, restore the radioactivity rate to within its limit within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours.

SURVEILLANCE REQUIREMENTS

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4.11.2.7.1 The radioactivity rate of noble gases at the recombiner after-condenser discharge shall be continuously monitored in accordance with Specification 3.3.7.1.

4.11.2.7.2 The radioactivity rate of noble gases from the recombiner after-condenser discharge shall be determined to be within the limits of Specification 3.11.2.7 at the following frequencies by performing an isotopic analysis of a representative sample of gases taken near the discharge of the main condenser air ejector:

- a. At least once per 31 days.
- b. Within 4 hours following an increase, as indicated by the Offgas Pretreatment Radiation Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady-state fission gas release from the primary coolant.
- c. The provisions of Specification 4.0.4 are not applicable.

*When the main condenser air ejector is in operation.

RADIOACTIVE EFFLUENTS

VENTING OR PURGING

LIMITING CONDITION FOR OPERATION

3.11.2.8 VENTING or PURGING of the Mark I containment drywell shall be through either the reactor building ventilation system or the filtration, recirculation and ventilation system.*

APPLICABILITY: Whenever the containment is vented or purged.

ACTION:

- a. With the requirements of the above specification not satisfied, suspend all VENTING and PURGING of the drywell.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.8 The containment shall be determined to be aligned for VENTING or PURGING through the reactor building ventilation system, the filtration, recirculation and ventilation system, or the hardened torus vent within 4 hours prior to start of and at least once per 12 hours during VENTING or PURGING of the drywell.

* Following Type A Integrated Leakage Rate Testing, the Mark I containment drywell may be vented through the hardened torus vent.