



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 74  
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated May 11, 1978, as supplemented and supported by letters dated January 12 and 24, 1979, May 30, 1979, June 12, 1979, August 17, 1979, October 19 and 29, 1979, December 2, 1980, January 29, 1981, June 8, 1981, October 2, 1981 and January 20, 1982, complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, pursuant to the Atomic Safety and Licensing Board's (ASLB), "Final Initial Decision," dated August 17, 1982, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and a new Paragraph 3.N is added. Thus, Paragraph 3.B is hereby revised and Paragraph 3.N is hereby incorporated in Provisional Operating License No. DPR-19 to read as follows:

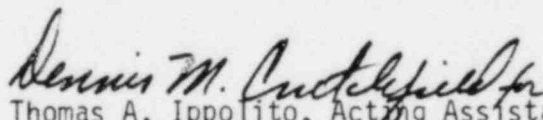
3.B Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3.N Spent Fuel Storage Racks\*

1. The licensee is authorized to install and use 33 high-density fuel storage racks in the spent fuel storage pool at Dresden Station, Unit 2.
  2. Fuel stored in the spent fuel pool shall have a U-235 loading less than or equal to 14.8 grams per axial centimeter.
  3. No loads heavier than the weight of a single spent fuel assembly and handling tool shall be carried over fuel stored in the spent fuel pool.
3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Thomas A. Ippolito, Acting Assistant Director  
for Safety Assessment  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 27, 1982

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\* Prior to the effective date of the ASLB's Final Initial Decision of August 17, 1982, the licensee had submitted to the Commission an updated FSAR for Dresden Station, Unit 2 to reflect the commitments set forth in the ASLB's Partial Initial Decision dated September 24, 1981, as referenced in the ASLB's Final Initial Decision, Section III.4.

ATTACHMENT TO LICENSE AMENDMENT NO. 74

PROVISIONAL OPERATING LICENSE NO. DPR-19

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Revise the Appendix "A" Technical Specifications by replacing the existing Page 157 with the attached Page 157.

## 5.0 DESIGN FEATURES

### 5.1 Site

Dresden Unit 2 is located at the Dresden Nuclear Power Station which consists of a tract of land of approximately 953 acres located in the north-east quarter of the Morris 15-minute quadrangle (as designated by the United States Geological Survey), Goose Lake Township, Grundy County, Illinois. The tract is situated in portions of Sections 25, 26, 27, 34, 35 and 36 of Township 34 North, Range 8 East of the Third Principal Meridian.

### 5.2 Reactor

- A. The core shall consist of not more than 724 fuel assemblies.
- B. The reactor core shall contain 177 cruciform-shaped control rods. The control material shall be boron carbide powder ( $B_4C$ ) compacted to approximately 70% of theoretical density.

### 5.3 Reactor Vessel

The reactor vessel shall be as described in Table 4.1.1 of the SAR. The applicable design codes shall be as described in Table 4.1.1 of the SAR.

### 5.4 Containment

- A. The principal design parameters and applicable design codes for the primary containment shall be as given in Table 5.2.1 of the SAR.
- B. The secondary containment shall be as described in Section 5.3.2 of the SAR and the applicable codes shall be as described in Section 12.1.1.3 of the SAR.
- C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with standards set forth in Section 5.3.2 of the SAR and the applicable codes shall be as described in Section 12.1.1.3 of the SAR.

### 5.5 Fuel Storage

- A. The new fuel storage facility shall be such that the  $K_{eff}$  dry is less than 0.90 and flooded is less than 0.95.
- B. The  $K_{eff}$  of the spent fuel storage pool shall be less than or equal to 0.95.

### 5.6 Seismic Design

The reactor building and all contained engineered safeguards are designed for the maximum credible earthquake ground motion with an acceleration of 20 per cent of gravity. Dynamic analysis was used to determine the earthquake acceleration, applicable to the various elevations in the reactor building.