UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 132
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated October 1, 1993, as revised January 6, 1994, and supplemented February 3, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 132 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

DBMatthews

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: March 17, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
VII	VII
XII	XII
3/4 6-3	3/4 6-3
3/4 6-4	3/4 6-4
3/4 6-7	3/4 6-7
3/4 6-24	3/4 6-24
B 3/4 6-2	B 3/4 6-2

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

BASES

SECTION	PAGE

PRIMARY CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Primary containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$, 1.2 percent by weight of the containment air per 24 hours at P_a, 57.5 psig, or

2. $\leq L_t$, 0.849 percent by weight of the containment air per 24 hours at a reduced pressure of P_t, 28.8 psig.

b. A combined leakage rate of:

1. $\leq 0.60\ L_a$ for all penetrations and valves, except for main steam isolation valves, subject to Type B and C tests when pressurized to P_a, and

2. $\leq 0.009\ L_a$ for the following penetrations*:

(a) Main steam condensate drain, penetration 8;

(b) Deleted

(c) Reactor water cleanup, penetration 14;

(d) Equipment drain sump discharge, penetration 18;

(e) Floor drain sump discharge, penetration 19; and

(f) Chemical drain sump discharge, penetration 55;

(g) Deleted

c. When tested at 28.8 psig**, 100 scf per hour for any one main steam isolation valve and a combined maximum pathway leakage rate of 250 scf per hour for all four main steam lines.

APPLICABILITY: When PRIMARY CONTAINMENT INTEGRITY is required per Specification 3.6.1.1.

*Potential bypass leakage paths.
**Exemption to Appendix J of 10 CFR 50.

LIMITING CONDITION FOR OPERATION (Continued)

ACTION:

With:

a. the measured overall integrated containment leakage rate exceeding 0.75 L_a or 0.75 L_t, as applicable, or

b. the measured combined leakage rate for all penetrations and valves, except main steam isolation valves, subject to Type B and C tests exceeding 0.60 L_a or with the measured combined leakage rate for all specified potential bypass leakage path penetrations exceeding 0.009 L_a, or

c. the main steam isolation valve measured leak rate exceeding 100 scf per hour for any one MSIV or a total maximum pathway leakage rate of > 250 scf per hour for all four main steam lines,

Restore:

a. the overall integrated leakage rate(s) to < 0.75 L_a or < 0.75 L_t as applicable, and

b. the combined leakage rate for all penetrations and valves, except main steam isolation valves, subject to Type B and C tests to ≤ 0.60 L_a and the combined leakage rate for the specified potential bypass leakage path penetrations to ≤ 0.009 L_a, and

c. the leakage rate to ≤ 11.5 scf per hour for any main steam isolation valve that exceeds 100 scf per hour, and restore the combined maximum pathway leakage rate to ≤ 250 scf per hour,

Prior to increasing the reactor coolant temperature above 212°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - (1972):

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a, 57.5 psig or at P_t, 28.8 psig during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

LIMITING CONDITION FOR OPERATION

3.6.1.4 Deleted

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

B. MANUAL ISOLATION VALVES[e]

1. Deleted

2. RHR return to recirculation loop isolation valves
   2E11-F015A, B

3. LOCA H_2 recombiner isolation valves
   2T49-F002 A, B
   2T49-F004 A, B

4. Core spray isolation valves
   2E21-F005A, B

5. Service air isolation valves
   2P51-F651
   2P51-F513

6. RBCCW supply and return isolation valves
   2P42-F051
   2P42-F052

[e]Includes power operated valves which do not isolate automatically.

3/4.6.1.4 MSIV LEAKAGE CONTROL SYSTEM

Deleted

3/4.6.1.5 PRIMARY CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the primary containment steel vessel will be maintained comparable to the original design standards for the life of the unit. Structural integrity is required to ensure that the vessel will withstand the maximum pressure of 57.5 psig in the event of a LOCA. A visual inspection in conjunction with Type A leakage tests is sufficient to demonstrate this capability.

3/4.6.1.6 PRIMARY CONTAINMENT INTERNAL PRESSURE

The limitations on primary containment internal pressure ensure that the containment peak pressure of 57.5 psig does not exceed the maximum allowable internal pressure of 62 psig during LOCA conditions or that the external pressure does not exceed the design maximum external pressure of 2 psig. The limit of 0.75 psig for initial positive containment pressure will limit the total pressure to 57.5 psig which is less than the maximum allowable internal pressure and is consistent with the accident analysis.

3/4.6.1.7 DRYWELL AVERAGE AIR TEMPERATURE

The limitation on drywell average air temperature ensures that the containment peak air temperature does not exceed the design temperature of 340°F during LOCA conditions and is consistent with the accident analysis.